

LICENSEE EVENT REPORT

CONTROL BLOCK:

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 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	I	L	D	R	S	2	2	0	0	-	0	0	0	0	0	0	3	4	1	1	1	1	4			5		
7	8	LICENSEE CODE						14	15	LICENSE NUMBER									25	26	LICENSE TYPE					30	37	CAT	38

CON'T

REPORT SOURCE L 6 0 5 0 0 0 2 3 7 7 0 6 0 3 8 3 3 0 6 3 0 8 3 9

7 8 61 68 69 74 75 90

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During normal operation a review of X area temperature showed an increase of
0 3 | approximately 25 degrees. A visual inspection of the area showed a test tap
0 4 | downstream of 2C MSIV leaking. The unit was shutdown for repairs. There was no
0 5 | effect on public health and safety as all leakage was contained within secondary
0 6 | containment. This is the first occurrence of a test connection pipe cracking
0 7 | in the main steam piping.
0 8 |

SYSTEM CODE C C 11		CAUSE CODE E 12		CAUSE SUBCODE B 13		COMPONENT CODE P I P E X X 14				COMP. SUBCODE A 15		VALVE SUBCODE Z 16					
EVENT YEAR 8 3 21 22		SEQUENTIAL REPORT NO. 0 4 4 24 25 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO. 0 32									
ACTION TAKEN B 18		FUTURE ACTION Z 19		EFFECT ON PLANT A 20		SHUTDOWN METHOD A 21		HOURS 0 0 2 1 22 40		ATTACHMENT SUBMITTED N 23		NPRC-4 FORM SUB. N 24		PRIME COMP. SUPPLIER Z 25		COMPONENT MANUFACTURER Z 9 9 9 26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The cause of the failure was fatigue due to improper support of the 3/4

1 1 | inch test connection resulting in a vibration induced stress crack in an

1 2 | elbow fillet weld. Because the test tap was no longer used the line was cut

1 3 | and capped to remove the moment arm effect of the unsupported test valves and

1 4 | connecting nipples.

FACILITY STATUS		% POWER		OTHER STATUS (30)		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION (32)			
1	5	E	(28)	0	9	8	(29)	N/A	A	(31)	Operator Observation

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)

1 6 7 8 9 10 11 N/A 12 13 14

LOCATION OF RELEASE (36)

15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	37	Z	38	N/A	39

PERSONNEL INJURIES		DESCRIPTION (41)	
NUMBER			
1	2	0	0
0	0	0	(40) N/A

		8		9		11		12		
		LOSS OF OR DAMAGE TO FACILITY						(43)		
		TYPE		DESCRIPTION						
1	9	Z	(42)	N/A						

8 9 10
PUBLICITY
ISSUED DESCRIPTION (45)
2 0 N (44) N/A
8307110115 830630
PDR ADOCK 05000237
S PDR
NRC USE ONLY
30

NRC USE ONLY

NAME OF PREPARER Don Maxwell

PHONE: (815) 942-2920 x489



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

June 30, 1983

DJS Ltr #83-650

James G. Keppler, Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #83-44/03L-0, Docket #050-237 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(d), abnormal degradation of systems other than those specified in item B.1.e. above designed to contain radioactive material resulting from the fission process.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC, Document Management Branch
File/NRC

JUL 5 1983

TE22
11



Commonwealth Edison

DEVIATION REPORT

DVR NO. 12 - 2 - 83 - 83
STA UNIT YEAR NO.

PART 1 TITLE OF DEVIATION

OCCURRED

Failure of 3/4" Test Connection on 2C Main Steam Line

6/3/83

0130

DATE

TIME

SYSTEM AFFECTED 250

PLANT STATUS AT TIME OF EVENT

Main Steam

MODE Run

PWR(MWT) 2500

LOAD(MWE) 818

TESTING

☐ YES ☒ NO

DESCRIPTION OF EVENT

Review of X area temperatures showed approximately 25°F increase

during 6/2/83. An upward trend was noted in the reactor building vent

stack radiation monitor reading. Subsequent leak inspection disclosed

excessive steam leakage into the X area, from an unknown source.

10 CFR50.72 NRC RED PHONE
NOTIFICATION MADE☐ YES ☒ NO

EQUIPMENT FAILURE D28344

☒ YES ☐ NO WORK REQUEST NO.

RESPONSIBLE SUPERVISOR Michael Wright

DATE 6/3/83

PART 2 OPERATING ENGINEER'S COMMENTS

An orderly load reduction was initiated to allow an X area inspection, which disclosed a crack in the test tap on 2C steam line, downstream of the outboard MSIV on C steam line. (Out side of pri. cont. boundary).

☐ EVENT OF PUBLIC INTEREST☐ TECH. SPEC. VIOLATION☐ NON REPORTABLE OCCURRENCE☐ 14 DAY REPORTABLE/T.S.☒ 30 DAY REPORTABLE/T.S. 6.6.B.2.d☐ ANNUAL/SPECL REPORT REQ'D

A.I.R. #

L.E.R. # 83-44/03L-0

☐ 24-HOUR NRC NOTIFICATION REQ'D (Courtesy)

TELEPH Stan Stacek

6/3/83

1300

REGION III

DATE

TIME

TELEGM/TELECOPY N/A

REGION III

DATE

TIME

☐ CECO CORPORATE NOTIFICATION MADE
IF ABOVE NOTIFICATION IS PER 10CFR21☐ 5-DAY WRITTEN REPORT REQ'D PER 10CFR21

Telecopy

TELETYPE

Dennis P. Galle

6/3/83

1433

CECO CORPORATE OFFICER

DATE

TIME

PRELIMINARY REPORT
COMPLETED AND REVIEWED

Michael Wright

6/3/83

OPERATING ENGINEER

DATE

INVESTIGATED REPORT & RESOLUTION
ACCEPTED BY STATION REVIEWJ. Brunner
6/30/83John M. Almes
6/30/83RESOLUTION APPROVED AND
AUTHORIZED FOR DISTRIBUTIONDouglas J. Coet
STATION SUPERINTENDENT6/30/83
DATE