

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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REPORT SOURCE

L	6	0	5	0	0	0	3	4	6	7	0	1	1	5	8	3	8	0	6	0	7	8	3	9
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DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

(NP-33-83-03) On 1/15/83, 1/18/83, 4/9/83, 4/10/83, and 5/10/83, the Reactor Coolant

System (RCS) Dose Equivalent I-131 exceeded the Technical Specification 3.4.8 limit of

1.0 uCi/gm. The peak levels measured were: 2.67 uCi/gm on 1/15/83, 1.39 uCi/gm on

1/18/83 and 4/9/83, 1.95 uCi/gm on 4/10/83, and 1.16 uCi/gm on 5/10/83. There was no

danger to the health and safety of the public or station personnel. At no time did

the specific activity exceed the allowable limit of Tech Spec 3.4.8 which accommodates

possible iodine spiking phenomenon that may occur following a change in thermal power.)

SYSTEM CODE R C 11		CAUSE CODE E 12		CAUSE SUBCODE C 13		COMPONENT CODE F U E L E X 14				COMP. SUBCODE Z 15		VALVE SUBCODE Z 16	
ACTION TAKEN Z 18		FUTURE ACTION Z 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 22		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB. N 24	
EVENT YEAR 8 3 21 22		SEQUENTIAL REPORT NO. 0 0 24 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE X 30		REVISION NO. 2 32		PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER B 0 1 5 26	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The cause was a slight leakage of fission products through the fuel cladding. Some

leakage is normal following a reactor trip due to RCS temperature and pressure

changes. The I-131 level was monitored until it dropped below the Technical Specifi-

cation limit.

[illegible]

PERSONNEL EXPOSURES									
NUMBER				TYPE	DESCRIPTION				
1	7	0	0	0	37	Z	38	NA	39

PERSONNEL INJURIES						8306170429 830807	
NUMBER		DESCRIPTION		(41)			
1	2	0	0	0	(40)	NA	
						PDR ADOCK 05000346	
						S PDR	

7	8	9	11	12	80
LOSS OF OR DAMAGE TO FACILITY					43
TYPE DESCRIPTION					

1 9 Z 42 NA

PUBLCITY		ISSUED		DESCRIPTION		NRC USE ONLY	
2	0	N	44	NA			

DVRs <sup>8</sup>83-006, <sup>9</sup>007, <sup>10</sup>008, 037, 056

NAME OF PREPARER

Lynn Richter

PHONE: 419-259-5000, Ext. 369



June 7, 1983

Log No. K83-829  
File: RR 2 (NP-33-83-03)

Docket No. 50-346  
License No. NPF-3

Mr. James G. Keppler  
Regional Director, Region III  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

Enclosed are three copies of Revision 2 to Licensee Event Report 83-002, including revised supplemental information pages. The revisions to the report are indicated by a "2" in the left margin of each page.

Please destroy your previous copies of this report and replace with the attached revision.

Yours truly,

Terry D. Murray  
Station Superintendent  
Davis-Besse Nuclear Power Station

TDM/ljk

Enclosure

cc: Mr. Richard DeYoung, Director  
Office of Inspection and Enforcement  
Encl: 30 copies

Mr. Norman Haller, Director  
Office of Management and Program Analysis  
Encl: 3 copies

Mr. Tom Peebles  
NRC Resident Inspector  
Encl: 1 copy

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TOLEDO EDISON COMPANY  
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-33-83-03

2

DATE OF EVENT: January 15 and 18, 1983; April 9 and 10, 1983;  
May 10, 1983

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Reactor Coolant System (RCS) Dose Equivalent I-131 Exceeded Technical Specification Limits

Conditions Prior to Occurrence: The unit was in Mode 3, with Power (MWT) = 0 and Load (Gross MWE) = 0

Description of Occurrence: On January 15, 1983 after a reactor trip, the specific activity of the primary coolant exceeded the Technical Specification 3.4.8 limit of 1.0  $\mu\text{Ci/gm}$  Dose Equivalent I-131. The level peaked at 2.67  $\mu\text{Ci/gm}$  at 2004 hours on January 15, 1983.

A similar event occurred after a reactor trip on January 18, 1983 with the dose equivalent level peaking at 1.39  $\mu\text{Ci/gm}$  at 2301 hours.

On April 9, 1983, at 0420 hours, following a power reduction to repair a failed steam generator level transmitter, the Dose Equivalent I-131 exceeded the limit of Technical Specification 3.4.8. The level peaked at 1.39  $\mu\text{Ci/gm}$  at 0620 hours on April 10, 1983.

2

On May 10, 1983, following a reactor/turbine trip, the Dose Equivalent I-131 exceeded the limit of Technical Specification 3.4.8. The level peaked at 1.16  $\mu\text{Ci/gm}$  at 1323 hours on May 10, 1983.

The following information is supplied per reporting requirements:

Power History

On January 15, 1983, reactor power was approximately 100% until a reactor/turbine trip at 1700 hours.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
1/14/83	0000 hours	99.4%
	1600 hours	99.5%
1/15/83	1600 hours	98.7%
	1700 hours	TRIP

On January 18, 1983, reactor power was approximately 100% until a reactor/turbine trip at 1925 hours.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
1/16/83	1600 hours	84.3%
1/17/83	0000 hours	87.3%
	0800 hours	97.9%
	1600 hours	99.2%
1/18/83	0000 hours	99.3%
	0800 hours	99.4%
	1600 hours	99.6%
	1925 hours	TRIP

On April 8, 1983, a power reduction from approximately 100% was initiated for a short maintenance outage.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
4/8/83	2000	99.1%
4/9/83	0100	48.5%
	0700	6.4%
	1600	21.7%
	2400	42.0%
4/10/83	0300	79.8%
	0345	TRIP

On May 10, 1983, reactor power was approximately 90% until a reactor/turbine trip at 1028 hours.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
5/9/83	1200	90%
5/10/83	0000	90%
	0800	90%
	1028	TRIP

Fuel Burnup By Core Region: See attached computer printout.

Cleanup Flow History 48 hours prior to the first limiting exceeding sample, the average letdown flow during this time was:

January 13, 1983 - 60 gpm  
 January 14, 1983 - 61 gpm  
 January 15, 1983 - 86 gpm

January 16, 1983 - 109 gpm  
January 17, 1983 - 101 gpm  
January 18, 1983 - 60 gpm

April 7, 1983 - 60 gpm  
April 8, 1983 - 69 gpm  
April 9, 1983 - 93 gpm  
April 10, 1983 - 95 gpm

May 8, 1983 - 72 gpm  
May 9, 1983 - 72 gpm  
May 10, 1983 - 72 gpm

There was no degassing operation.

Table 1 lists the specific activity analysis, the time duration the specific activity exceeded the Technical Specification, and the maximum level reached for each occurrence.

Designation of Apparent Cause of Occurrence: The cause was a slight leakage of fission products through the fuel cladding. Some leakage is normal following a reactor trip due to RCS temperature and pressure changes.

Analysis of Occurrence: There was no danger to the health and safety of the public or station personnel.

At no time did the specific activity exceed the allowable limit of Technical Specification 3.4.8, which accommodates possible iodine spiking phenomenon that may occur following a change in thermal power as shown in Figure 3.4-1.

Corrective Action: Per the action statement requirements of Technical Specification 3.4.8, sampling of the primary coolant was performed at least once every four hours. Monitoring of the I-131 level continued until it dropped below the Technical Specification limit of 1.0  $\mu\text{Ci/gm}$ . Since iodine spikes are typical following a power change, no additional corrective action is applicable.

Failure Data: Previous occurrences of the RCS dose equivalent I-131 exceeding the Technical Specification limit have been reported in Licensee Event Reports NP-33-80-114 (80-088), NP-33-81-15 (81-016), NP-33-81-37 (81-031).

LER #83-002

TABLE 1

Date of Occurrence	Specific Activity Analysis			Approximate Time Duration Tech Spec Level Exceeded	Maximum Level $\mu\text{Ci/gm}$
	Date	Time	Activity ( $\mu\text{Ci/gm}$ ) Dose Equivalent I-131		
January 15, 1983	1/15/83	0810	0.17	21 hours	2.67
		2004	2.67		
		2155	2.54		
	1/16/83	0102	2.33		
		0400	2.40		
		0730	1.59		
		0930	1.58		
		1330	1.14		
		1730	0.73		
January 18, 1983	1/18/83	1354	0.14	8 hours	1.39
		2301	1.39		
	1/19/83	0256	1.11		
		0645	0.81		
April 9, 1983	4/8/83	1207	0.09	11 hours	1.39
	4/9/83	0420	1.06		
		0806	1.37		
		1145	1.39		
		1531	0.96		
April 10, 1983	4/10/83	0130	0.41	15 hours	1.95
		0620	1.95		
		1002	1.82		
		1341	1.31		
		1710	1.30		
		2110	0.89		
May 10, 1983	5/10/83	0811	0.06	8 hours	1.16
		1323	1.16		
		1600	0.95		
		2000	0.85		
	5/11/83	0820	0.33		



FUEL BURNUP BY CORE REGION FOR  
JANUARY 15, 1983 AND JANUARY 18, 1983 EVENTS

The fuel burnup is given for each fuel assembly as designated by the computer assigned assembly number.  
 For the location of the assemblies, see the attached core map. Units are given in  $\frac{\text{Megawatt Days}}{\text{Metric Ton}} \times 10^{-3}$ .

								FUEL ASSEMBLIES*
11194E+01	11194E+01	11194E+01	11194E+01	11194E+01	11194E+01	11194E+01	11194E+01	1 through 7
11194E+02	11194E+02	11194E+02	11194E+02	11194E+02	11194E+02	11194E+02	11194E+02	8 through 14
13450E+01	13450E+01	13450E+01	13450E+01	13450E+01	13450E+01	13450E+01	13450E+01	15 through 21
23979E+02	23979E+02	23979E+02	23979E+02	23979E+02	23979E+02	23979E+02	23979E+02	22 through 28
14377E+02	14377E+02	14377E+02	14377E+02	14377E+02	14377E+02	14377E+02	14377E+02	29 through 35
26059E+02	26059E+02	26059E+02	26059E+02	26059E+02	26059E+02	26059E+02	26059E+02	36 through 42
12306E+02	12306E+02	12306E+02	12306E+02	12306E+02	12306E+02	12306E+02	12306E+02	43 through 49
24681E+02	24681E+02	24681E+02	24681E+02	24681E+02	24681E+02	24681E+02	24681E+02	50 through 56
13653E+02	13653E+02	13653E+02	13653E+02	13653E+02	13653E+02	13653E+02	13653E+02	57 through 63
23477E+02	23477E+02	23477E+02	23477E+02	23477E+02	23477E+02	23477E+02	23477E+02	64 through 70
27439E+02	27439E+02	27439E+02	27439E+02	27439E+02	27439E+02	27439E+02	27439E+02	71 through 77
16703E+02	16703E+02	16703E+02	16703E+02	16703E+02	16703E+02	16703E+02	16703E+02	78 through 84
50180E+01	50180E+01	50180E+01	50180E+01	50180E+01	50180E+01	50180E+01	50180E+01	85 through 91
26070E+02	26070E+02	26070E+02	26070E+02	26070E+02	26070E+02	26070E+02	26070E+02	92 through 98
13524E+02	13524E+02	13524E+02	13524E+02	13524E+02	13524E+02	13524E+02	13524E+02	99 through 105
26987E+02	26987E+02	26987E+02	26987E+02	26987E+02	26987E+02	26987E+02	26987E+02	106 through 112
19929E+02	19929E+02	19929E+02	19929E+02	19929E+02	19929E+02	19929E+02	19929E+02	113 through 119
26153E+02	26153E+02	26153E+02	26153E+02	26153E+02	26153E+02	26153E+02	26153E+02	120 through 126
38557E+01	38557E+01	38557E+01	38557E+01	38557E+01	38557E+01	38557E+01	38557E+01	127 through 133
26938E+02	26938E+02	26938E+02	26938E+02	26938E+02	26938E+02	26938E+02	26938E+02	134 through 140
44076E+01	44076E+01	44076E+01	44076E+01	44076E+01	44076E+01	44076E+01	44076E+01	141 through 147
15423E+02	15423E+02	15423E+02	15423E+02	15423E+02	15423E+02	15423E+02	15423E+02	148 through 154
24797E+02	24797E+02	24797E+02	24797E+02	24797E+02	24797E+02	24797E+02	24797E+02	155 through 161
44202E+01	44202E+01	44202E+01	44202E+01	44202E+01	44202E+01	44202E+01	44202E+01	162 through 168
22196E+02	22196E+02	22196E+02	22196E+02	22196E+02	22196E+02	22196E+02	22196E+02	169 through 175
35331E+01	35331E+01	35331E+01	35331E+01	35331E+01	35331E+01	35331E+01	35331E+01	176 through 177

\*AS READ FROM LEFT TO RIGHT

FUEL BURNUP BY CORE REGION FOR  
APRIL 9, 1983 AND APRIL 10, 1983 EVENTS

The fuel burnup is given for each fuel assembly as designated by the computer assigned assembly number.  
For the location of the assemblies, see the attached core map. Units are given in  $\frac{\text{Megawatt Days}}{\text{Metric Ton}} \times 10^{-3}$

								FUEL ASSEMBLIES*
44936E+01	57712E+01	61204E+01	57816E+01	44983E+01	47002E+01	62491E+01		1 through 7
13825E+02	24556E+02	20118E+02	24577E+02	13846E+02	62568E+01	46772E+01		8 through 14
14713E+02	72375E+01	26855E+02	26354E+02	18153E+02	17396E+02	18151E+02		15 through 21
26274E+02	27256E+02	71974E+01	14675E+02	47347E+01	72681E+01	28439E+02		22 through 28
17058E+02	17350E+02	18727E+02	82091E+01	18736E+02	17362E+02	17048E+02		29 through 35
28404E+02	71681E+01	46475E+01	63162E+01	27311E+02	17281E+02	26992E+02		36 through 42
15293E+02	29397E+02	30291E+02	29426E+02	15294E+02	26988E+02	17246E+02		43 through 49
26851E+02	61934E+01	45356E+01	13502E+02	26025E+02	17535E+02	15060E+02		50 through 56
16763E+02	28583E+02	16523E+02	28587E+02	16764E+02	15046E+02	17502E+02		57 through 63
25777E+02	13386E+02	44434E+01	57979E+01	24898E+02	17628E+02	18349E+02		64 through 70
29679E+02	29402E+02	16332E+02	25019E+02	16342E+02	29437E+02	29779E+02		71 through 77
18341E+02	17580E+02	24849E+02	56999E+01	61106E+01	20082E+02	17341E+02		78 through 84
80721E+01	30159E+02	16454E+02	24963E+02	18336E+02	25018E+02	16495E+02		85 through 91
30239E+02	81254E+01	17337E+02	20037E+02	60362E+01	57401E+01	24829E+02		92 through 98
17527E+02	18289E+02	29756E+02	29373E+02	16287E+02	24966E+02	16304E+02		99 through 105
29412E+02	29815E+02	18349E+02	17571E+02	24823E+02	56923E+01	44577E+01		106 through 112
13373E+02	25665E+02	17444E+02	14955E+02	16671E+02	28517E+02	16454E+02		113 through 119
28498E+02	16730E+02	15031E+02	17518E+02	25773E+02	13367E+02	44283E+01		120 through 126
61737E+01	27403E+02	17149E+02	26882E+02	15190E+02	29310E+02	30204E+02		127 through 133
29186E+02	15187E+02	26919E+02	17183E+02	27172E+02	61634E+01	46353E+01		134 through 140
71281E+01	28340E+02	16992E+02	17318E+02	18659E+02	81061E+01	18628E+02		141 through 147
17278E+02	16983E+02	28363E+02	71310E+01	46315E+01	14663E+02	71529E+01		148 through 154
26980E+02	26271E+02	18128E+02	17356E+02	18105E+02	26259E+02	27341E+02		155 through 161
71503E+01	14641E+02	46628E+01	62239E+01	13791E+02	24550E+02	20071E+02		162 through 168
24538E+02	13803E+02	62159E+01	46581E+01	44822E+01	57618E+01	61114E+01		169 through 175
57592E+01	44776E+01							176 through 177

\*AS READ FROM LEFT TO RIGHT



FUEL BURNUP BY CORE REGION FOR  
MAY 10, 1983 EVENT

The fuel burnup is given for each fuel assembly as designated by the computer assigned assembly number.  
For the location of the assemblies, see the attached core map. Units are given in  $\frac{\text{Megawatt Days}}{\text{Metric Ton}} \times 10^{-3}$

FUEL ASSEMBLIES\*

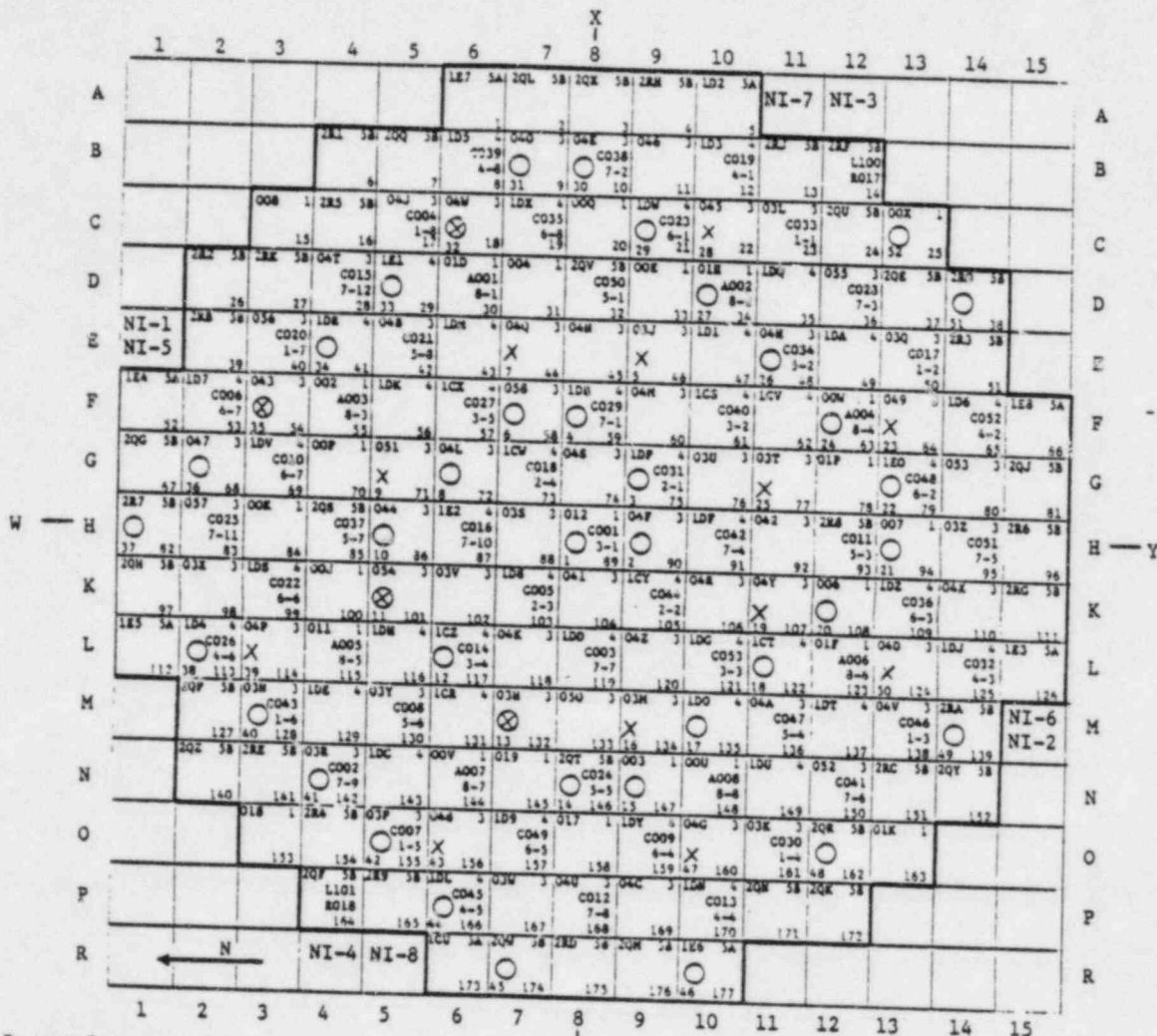
50807E+01	65150E+01	65043E+01	65292E+01	50900E+01	63046E+01	70320E+01	1 through 7
14686E+02	25330E+02	26591E+02	25508E+02	14716E+02	70488E+01	52878E+01	8 through 14
15145E+02	81454E+01	27074E+02	27139E+02	19035E+02	18099E+02	19038E+02	15 through 21
27052E+02	27984E+02	81135E+01	15109E+02	53404E+01	81772E+01	29218E+02	22 through 28
17969E+02	18018E+02	19474E+02	92474E+01	19486E+02	18035E+02	17967E+02	29 through 35
25187E+02	60814E+01	50549E+01	71010E+01	23037E+02	18219E+02	27872E+02	36 through 42
16306E+02	36171E+02	31018E+02	30202E+02	16306E+02	27874E+02	18186E+02	43 through 49
27578E+02	65787E+01	51227E+01	14315E+02	26821E+02	18243E+02	16026E+02	50 through 56
17727E+02	29355E+02	17386E+02	29356E+02	17733E+02	16013E+02	18213E+02	57 through 63
26552E+02	14198E+02	50293E+01	65391E+01	25644E+02	18437E+02	19038E+02	64 through 70
30429E+02	30211E+02	17222E+02	25509E+02	17236E+02	30252E+02	30550E+02	71 through 77
19034E+02	18391E+02	26505E+02	64397E+01	68884E+01	20849E+02	18036E+02	78 through 84
90957E+01	30873E+02	17309E+02	25849E+02	19122E+02	25907E+02	17356E+02	85 through 91
30962E+02	21574E+01	18355E+02	23804E+02	68126E+01	64793E+01	25573E+02	92 through 98
19334E+02	10976E+02	30927E+02	30182E+02	17176E+02	25854E+02	17196E+02	99 through 105
30228E+02	31568E+02	19642E+02	18381E+02	25568E+02	64307E+01	50423E+01	106 through 112
14181E+02	26426E+02	18149E+02	15919E+02	17632E+02	29283E+02	17314E+02	113 through 119
29265E+02	17697E+02	16003E+02	18229E+02	26545E+02	14176E+02	50117E+01	120 through 126
63544E+01	38125E+02	19084E+02	27761E+02	16194E+02	30081E+02	30927E+02	127 through 133
25944E+02	16195E+02	27002E+02	18122E+02	27995E+02	64442E+01	52394E+01	134 through 140
80350E+01	29118E+02	17994E+02	17987E+02	19495E+02	31411E+01	19374E+02	141 through 147
17948E+02	17893E+02	29141E+02	80382E+01	52355E+01	15094E+02	80626E+01	148 through 154
27703E+02	27061E+02	19011E+02	18058E+02	18388E+02	27043E+02	28063E+02	155 through 161
80594E+01	15072E+02	52698E+01	70107E+01	14657E+02	25327E+02	20844E+02	162 through 168
25316E+02	14663E+02	70015E+01	52644E+01	50712E+01	65073E+01	68952E+01	169 through 175
65941E+01	58625E+01						176 through 177

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TOLEDO EDISON COMPANY  
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-33-83-03  
PAGE 8

DAVIS-BESSE UNIT 1, CYCLE 3

March 18, 1982



Incore Detector  
○ = Total Core Monitor  
X = Symmetry Monitor  
⊗ = Total Core and Symmetry Monitor

Incore Detector String Number

Regenerative Sources at P4 and B12

Computer Assigned Assembly Number

Fuel Assembly ID  
All ID's preceded by "NJ0"

Fuel Assembly Batch ID, (No. in Core)  
1 (25) Former Batch A - once burned  
3 (60) Former Batch C - twice burned  
4 (44) Former Batch D - once burned  
5A (8) New Fuel - 3.04% enriched  
5B (40) New Fuel - 2.99% enriched

Control Component/Source ID and Type  
A = APSRA C = CRA  
R = Regenerative Neutron Source

Control Component Group/Rod

Prepared By Daniel B. Kelley Approved By D. J. Dike