

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MAY 1983

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The condenser cooling method is a closed cycle spray canal, and the Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively, pursuant to Docket Numbers 50-254 and 50-265. The date of initial reactor criticalities for Units 1 and 2 respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit 1 and March 10, 1973 for Unit 2.

This report was compiled by Becky Brown and Alex Misak, telephone number 309-654-2241, extensions 127 and 194.

II. SUMMARY OF OPERATING EXPERIENCE

A. UNIT ONE

May 1-20: Unit One began the month operating at full power, and continued to operate at this level throughout the period, except for three occasions when the unit dropped load to perform weekly Turbine tests. At 1200 hours, on May 20, the unit dropped load 200 MWe/hour to 250 MWe in anticipation of unit shutdown for a weekend maintenance outage. While holding load at 250 MWe, control rod drive scram timing was performed. At 2340 hours, control rod insertion began for unit shutdown.

May 21-31: At 0105 hours, on May 21, the unit was shutdown to repair a leak on the continuous Reactor head vent line. At 1535 hours, on May 22, control rod withdrawal began, and at 1833 hours the Reactor was critical. During Reactor startup, with the Reactor at 60 psig, at 1937 hours, the "E" Main Steam Electromatic Relief Valve inadvertently opened. The Reactor was manually scrammed, per procedure. The relief valve pilot valve had stuck open, and was replaced. Repairs to the Electromatic Relief Valve were completed on May 23, and the Reactor was critical at 0355 hours.

At 1427 hours, the Generator was placed on-line, and a normal load increase was initiated.

On May 26, the unit began to experience a derating of approximately 60 MWe due to tube leaks in a "D" Feedwater Heater. The unit continued to operate at about 770 MWe for the remainder of the month, except for two occasions when the unit dropped load for weekly Turbine tests and control rod pattern changes.

B. UNIT TWO

Unit Two continued to be derated throughout the month due to End of Cycle Fuel Depletion. Load was held steady throughout the month until 0130 hours on May 30 when the unit dropped load to 400 MWe as requested by the Load Dispatcher due to low system demand. At 0635 hours, the unit increased load normally until, at 1430 hours, load was 630 MWe.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY
RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Unit One and Unit Two during the reporting period. This summary includes the following headings: Work Request Numbers, LER Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT ONE MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q26331	83-23/03L	1-203-3E Electro- matic	Pilot valve stuck open.	When the electromatic went open the situation was identified & the Reactor scrammed. Reactor was in Startup at 60 psig.	The pilot valve was replaced like-for-like.
Q26348	83-22/03L	1-1301-16 RCIC Inboard Steam Supply	A wire was loose on the indicating switch at the valve.	When the valve was given an OPEN signal, it would start to open then appear to close. Since it was an indication problem, the valve would have operated if required.	The wire was tightened and contacts cleaned.
Q26092	83-20-/03L	1-1601-33F Suppression Chamber to Dry- well Vacuum Breaker Indication Switch	The shaft of the vacuum breaker was binding slightly.	The Drywell-Torus Separation test was performed to determine that the valve was fully closed.	The valve was lubricated and the switches adjusted.

UNIT TWO MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q26272	83-8/03L	Southeast RHR Vault Submarine Door	The door's bottom latch arm was bent, preventing the operation of the handwheel.	The door would not seal. This made one loop of Containment Cooling mode of RHRS inoperable. The required surveillances were performed.	The latch was straightened.
Q24643		Rod Block Monitor Channel 7	The G8 integrated circuit on the null sequence board was failed.	Because Unit 2 was in a limiting control rod pattern, the channel 7 RBM was jumpered to provide a continuous rod block until the RBM could be repaired.	The G8 integrated circuit was replaced and the monitor was tested functionally.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

<u>UNIT ONE</u>		
<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
83-20/03L	5-10-83	1-1601-33F Vacuum Breaker Lost Division II Indication
83-21/01T	5-20-83	Continuous Reactor Head Vent Pinhole Leak 1-0214- 2"-B
83-22/03L	5-23-83	1-1301-16 Valve Out of Service for Repairs
83-23/03L	5-22-83	3E Electromatic Relief Valve Stuck Open
<u>UNIT TWO</u>		
83-7/01T	5-9-83	Control Rod Drives Inoperable
83-8/03L	5-20-83	"B" RHR Loop Inoperable

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE June 06 1983

COMPLETED BY Erich Weinfurter

TELEPHONE 309-654-2241X191

OPERATING STATUS

0000 050183

1. Reporting period: 2400 053183 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>694.2</u>	<u>3415.6</u>	<u>78586.8</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>682.6</u>	<u>3375.5</u>	<u>75462.2</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>1615467</u>	<u>8120041</u>	<u>154333032</u>
10. Gross electrical energy generated (MWH)	<u>530944</u>	<u>2670536</u>	<u>49792417</u>
11. Net electrical energy generated (MWH)	<u>500652</u>	<u>2515130</u>	<u>46344038</u>
12. Reactor service factor	<u>93.3</u>	<u>94.3</u>	<u>81.1</u>
13. Reactor availability factor	<u>93.3</u>	<u>94.3</u>	<u>84.6</u>
14. Unit service factor	<u>91.8</u>	<u>93.2</u>	<u>77.9</u>
15. Unit availability factor	<u>91.8</u>	<u>93.2</u>	<u>78.8</u>
16. Unit capacity factor (Using MDC)	<u>87.5</u>	<u>90.3</u>	<u>62.2</u>
17. Unit capacity factor (Using Des. MWe)	<u>85.3</u>	<u>88.0</u>	<u>60.6</u>
18. Unit forced outage rate	<u>1.2</u>	<u>2.1</u>	<u>6.5</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE June 06 1983

COMPLETED BY Erich Weinfurter

TELEPHONE 309-654-2241X191

OPERATING STATUS

0000 050183

1. Reporting period: 2400 053183 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>744.0</u>	<u>3359.0</u>	<u>75622.4</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>744.0</u>	<u>3329.7</u>	<u>72917.8</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>1489762</u>	<u>7311495</u>	<u>151902989</u>
10. Gross electrical energy generated (MWH)	<u>477002</u>	<u>2343436</u>	<u>48380971</u>
11. Net electrical energy generated (MWH)	<u>445056</u>	<u>2195616</u>	<u>45379183</u>
12. Reactor service factor	<u>100.0</u>	<u>92.7</u>	<u>78.8</u>
13. Reactor availability factor	<u>100.0</u>	<u>92.7</u>	<u>81.9</u>
14. Unit service factor	<u>100.0</u>	<u>91.9</u>	<u>76.0</u>
15. Unit availability factor	<u>100.0</u>	<u>91.9</u>	<u>76.7</u>
16. Unit capacity factor (Using MDC)	<u>77.8</u>	<u>78.8</u>	<u>61.5</u>
17. Unit capacity factor (Using Des. MWe)	<u>75.8</u>	<u>76.8</u>	<u>59.9</u>
18. Unit forced outage rate	<u>0.0</u>	<u>3.1</u>	<u>8.9</u>
19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):			
20. If shutdown at end of report period, estimated date of startup			<u>NA</u>

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

#UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE June 06 1983

COMPLETED BY Erich Weinfurter

TELEPHONE 309-654-2241X191

MONTH May 1983

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>697.3</u>
2.	<u>783.8</u>
3.	<u>785.8</u>
4.	<u>782.5</u>
5.	<u>783.3</u>
6.	<u>782.2</u>
7.	<u>774.7</u>
8.	<u>752.0</u>
9.	<u>781.1</u>
10.	<u>783.2</u>
11.	<u>779.3</u>
12.	<u>778.4</u>
13.	<u>776.6</u>
14.	<u>781.9</u>
15.	<u>749.8</u>
16.	<u>784.3</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>774.7</u>
18.	<u>779.2</u>
19.	<u>780.9</u>
20.	<u>530.2</u>
21.	<u>-7.1</u>
22.	<u>-11.4</u>
23.	<u>104.4</u>
24.	<u>526.0</u>
25.	<u>702.7</u>
26.	<u>721.6</u>
27.	<u>714.0</u>
28.	<u>734.3</u>
29.	<u>720.7</u>
30.	<u>723.6</u>
31.	<u>710.8</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE June 06 1983

COMPLETED BY Erich Weinfurter

TELEPHONE 309-654-2241X191

MONTH May 1983

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>612.0</u>
2.	<u>636.0</u>
3.	<u>640.2</u>
4.	<u>633.3</u>
5.	<u>630.1</u>
6.	<u>626.4</u>
7.	<u>623.1</u>
8.	<u>627.3</u>
9.	<u>623.6</u>
10.	<u>620.3</u>
11.	<u>615.3</u>
12.	<u>615.5</u>
13.	<u>599.7</u>
14.	<u>606.6</u>
15.	<u>603.3</u>
16.	<u>601.3</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>594.3</u>
18.	<u>595.0</u>
19.	<u>594.2</u>
20.	<u>585.5</u>
21.	<u>577.8</u>
22.	<u>575.5</u>
23.	<u>579.1</u>
24.	<u>580.0</u>
25.	<u>574.0</u>
26.	<u>574.5</u>
27.	<u>568.6</u>
28.	<u>568.0</u>
29.	<u>570.7</u>
30.	<u>528.5</u>
31.	<u>564.7</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be annotated to explain the apparent anomaly.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13

Revision 6

August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY Alex Misak, ext 194

DATE June 6, 1983

REPORT MONTH MAY 1983

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
83-28	830501	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine test
83-29	830507	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine test
83-30	830515	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine test
83-31	830520	S	41.47	B	2		CJ	PIPEXX	Unit shutdown for maintenance on continuous Reactor head vent line
83-32	830522	F	8.3	B	2		CC	VALVOP	Unit shutdown for Relief Valve problems
83-33	830529	S	0.0	B	5		HA	XXXXXX	Reduced load to perform weekly Turbine test
83-34	830531	F	0.0	A	5		RB	XXXXXX	Reduced load to change rod pattern

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AUG 16 1982

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13

Revision 6

August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit 2

COMPLETED BY Alex Misak, 194

DATE June 6, 1983

REPORT MONTH MAY 1983

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
83-35	830530	F	0.0	F	5				Load reduction requested by Load Dispatcher

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AUG 16 1982

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

<u>UNIT</u>	<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO. & TYPE ACTUATIONS</u>	<u>PLANT CONDITIONS</u>	<u>DESCRIPTION OF EVENTS</u>
1	5-23-83	1-203-3E	1 Manual	Rx Press 1000	Post-Maintenance - (Replaced pilot valve)

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specification 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with Reactor pressure greater than 800 psig.

RESULTS OF SCRAM TIMING MEASUREMENTS

PERFORMED ON UNIT 1 & 2 CONTROL

ROD DRIVES, FROM 1-1 TO 12-31-83

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				Max. Time For 90% Insertion	DESCRIPTION
		5	20	50	90		
		0.375	0.900	2.00	3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
5-23	89	0.28	0.65	1.41	2.49	2.79 (H-8)	Unit One Hot Scram Time "B" Sequence

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q1 Reload: 6 Cycle: 7
2. Scheduled date for next refueling shutdown: 9-6-82
3. Scheduled date for restart following refueling: 12-18-82
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment: Yes
5. Scheduled date(s) for submitting proposed licensing action and supporting information: 8-19-82: Tech. Spec. changes submitted to the NRC.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
- a) All 7x7 fuel assemblies will be removed from the core.
 - b) MAPLHGR curves for fuel types in the core are being extended to 40,000 MWD/ST.
 - c) MCPR limits will be determined by GE's ODYN computer code.
 - d) The vessel pressure safety limit is being modified to accommodate the potential for higher reactor pressures as calculated by ODYN.
7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 800
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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APR 20 1978

Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q2 Reload: 6 Cycle: 7
2. Scheduled date for next refueling shutdown: 9-5-83
3. Scheduled date for restart following refueling: 11-12-83
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
No, however, a change to the Technical Specifications is being submitted (see below).
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
June 14, 1983 (scheduled)
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
a) All new fuel assemblies will be of barrier design; MAPLHGR curves will be re-labeled to include the barrier designation.
b) The use of improved assumptions in the load reject without bypass analysis resulted in a much improved MCPR operating limit. Technical Specifications are being changed to provide this additional operating margin.
7. The number of fuel assemblies.
a. Number of assemblies in core: 724
b. Number of assemblies in spent fuel pool: 204
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 3897
b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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APR 20 1978

Q. C. O. S. R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center



Commonwealth Edison

Quad Cities Nuclear Power Station
22710 206 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

NJK-83-205

June 6, 1983

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, D. C. 20555
Attention: Document Control Desk

Gentlemen:

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of May 1983.

Very truly yours,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

L. J. Kalivianakis for
N. J. Kalivianakis
Station Superintendent

bb

Enclosure

IE 24
11