



Public Service Electric and Gas Company P.O. Box 168 Hancocks Bridge, New Jersey 08038

Salem Generating Station

August 28, 1981

Mr. Boyce H. Grier
Director of USNRC
Office of Inspection and Enforcement
Region 1
631 Park Avenue
King of Prussia, Pennsylvania 19406



Dear Mr. Grier:

LICENSE NO. DPR-75
DOCKET NO. 50-311
REPORTABLE OCCURRENCE 81-84/03L

Pursuant to the requirements of Salem Generating Station Unit No. 2 Technical Specifications, Section 6.9.1.9.b, we are submitting Licensee Event Report for Reportable Occurrence 81-84/03L. This report is required within thirty (30) days of the occurrence.

Sincerely yours,

H. J. Midura
Manager -
Salem Generating Station

CC: R. A. Uderitz
General Manager - Nuclear Production
Director, Office of Inspection
and Enforcement (30 copies)
Director, Office of Management
Information and Program Control
(3 copies)

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Report Number: 81-84/03L
Report Date: 8-28-81
Occurrence Date: 8-10-81
Facility: Salem Generating Station, Unit 2
Public Service Electric & Gas Company
Hancocks Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

Identified Reactor Coolant System Leakage - Greater Than 10 GPM
This report is initiated by Incident Report 81-319.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 1 - Rx Power 70% - Unit Load 730 MWe

DESCRIPTION OF OCCURRENCE:

At approximately 0235 hours, August 10, 1981, a leak rate calculation indicated a Reactor Coolant System leak rate of 11.6 gpm to the reactor coolant drain tank. An unusual event was declared in accordance with EPI-1. At 0238 hours a shutdown was commenced at the rate of 1% per minute, and action statement 3.4.7.2.b was entered.

This constituted operation in a degraded mode in accordance with technical specification 6.9.1.9.b.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

The source of the Reactor Coolant System leakage was packing leakage on valve 22RC20.

ANALYSIS OF OCCURRENCE:

Technical specification 3.4.7.2.b requires:

With any identified Reactor Coolant System leakage greater than 10 gpm, reduce the leakage rate to within limits within 4 hours or be in at least hot standby within the next 6 hours and in cold shutdown within the following 30 hours.

CORRECTIVE ACTION:

The plant was taken off the line at 0400 hours and was in mode 3 at 0415 hours. Valve 22RC20 was repacked, and the leak rate was reduced to within specification. At 2045 hours, August 10, 1981, action statement 3.4.7.2.b was terminated.

FAILURE DATA:

Velan 3" Gate Valve
Type B10354B13M

Prepared By F. Dickey

SORC Meeting No. 81-83

H. J. Infeluna
Manager - Salem Generating Station