



Wisconsin Electric POWER COMPANY
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53201

September 1, 1981

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. NUCLEAR REGULATORY COMMISSION
Washington, D. C. 20555

Dear Mr. Denton:

DOCKET NOS. 50-266 AND 50-301
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 70
SNUBBER SURVEILLANCE, OVERPRESSURE MITIGATING SYSTEM,
CHECKS AND TESTS OF SURVEY INSTRUMENTS
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2



In accordance with Section 50.59 of 10 CFR Part 50, Wisconsin Electric Power Company (Licensee) hereby applies for amendments to Facility Operating Licenses DPR-24 and DPR-27 for the Point Beach Nuclear Plant Units 1 and 2, respectively. The purpose of these license amendments is to incorporate into the Technical Specifications, as requested by the Nuclear Regulatory Commission, additional restrictions concerning hydraulic snubber surveillance and to make other changes to the Specifications as discussed below.

On November 20, 1980, the NRC sent a letter to licensees regarding technical specification revisions for mechanical and hydraulic snubber surveillance. Wisconsin Electric responded in a letter dated February 19, 1981 and stated that the existing Point Beach Technical Specifications adequately addressed the Commission's recommendations for hydraulic snubber specifications and that mechanical snubbers are not installed on safety-related equipment at Point Beach. In Mr. Robert A. Clark's letter dated July 2, 1981, the NRC Staff reported on their review of this response and listed two unresolved items.

The first of these unresolved items concerns the exemption in the Point Beach Technical Specifications regarding functional testing of hydraulic snubbers of over 50,000 lbs. rated capacity. New NRC requirements have eliminated this exemption. Accordingly, we have modified Specification 15.4.13.2 to remove this exemption.

w/ check
\$1,600
Aug 31/1

September 1, 1981

The second unresolved item regards the requirement for monitoring of service life for hydraulic snubbers. In response to this concern we have added an item 4 to Specification 15.4.13 which requires that records of service life and maintenance history for each safety-related snubber be maintained and monitored. This Specification essentially duplicates the wording suggested in the standard technical specifications included with Mr. Clark's letter.

Included with this amendment request is a clarification to Specification 15.3.15, Overpressure Mitigating System. The purpose of Specification 15.3.15 is to insure the availability of a diverse means of relieving pressure during periods of water solid operations. As currently written, this Specification requires activation of the Overpressure Mitigating System whenever the reactor coolant system temperature is less than the minimum pressurization temperature for the inservice pressure test. This effectively precludes operation of the reactor coolant system during heatup and cooldown evaluations at pressures above 425 psig but below the permissible operating curve of the applicable heatup and cooldown limitations presented in Figures 15.3.1-1 through 15.3.1-4 even though a pressure absorbing volume or "bubble" is maintained in the pressurizer. Since we believe the intent of these Specifications, as stated in the objective on page 15.3.15-1, is to specify limiting conditions for operation when the reactor coolant system is operated without a pressure absorbing volume in the pressurizer, Licensee has proposed changes to this Specification to clarify this intent. These changes are identified by margin bars on the attached proposed Technical Specification pages.

We also are requesting an administrative change to Table 15.4.1-1, item 27, with this license amendment application. Item 27 specifies checks, calibration and test frequencies for the Emergency Plan radiation survey instruments. The existing Specification requires checks and tests of these instruments on a monthly basis. The guidance contained in NUREG-0654 on page 46 would allow these checks and tests to be conducted quarterly. Accordingly, we hereby propose that this Specification be changed to conform with the NUREG-0654 guidance. A proposed Technical Specification change with this change is attached.

As required by 10 CFR Section 170.27, Licensee has evaluated these proposed amendments and determined that the changes are administrative in nature and have no safety or environmental significance. The amendment to DPR-24 should, therefore, be classified as Class II and the duplicate changes requested for DPR-27 classified as a Class I amendment. We have, therefore, enclosed herewith a check in the amount of \$1,600 to cover the amendment fees required for the NRC review and approval of this application.

As required by the Commission's regulations, we are providing three signed originals and forty copies of this license amendment request and the attached proposed Technical Specification page changes. Should you

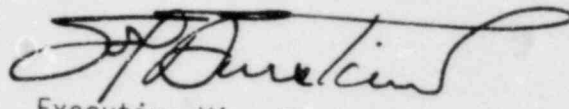
Mr. Harold R. Denton

-3-

September 1, 1981

have any questions concerning the contents of this application, please let us know.

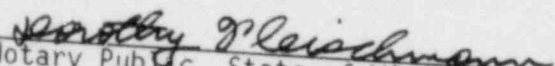
Very truly yours,


Executive Vice President

Sol Burstein

Enclosures

Subscribed and sworn to before me
This 1st day of September, 1981.


Notary Public, State of Wisconsin

My Commission expires July 1, 1984

Copy to NRC Resident Inspector

