

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE August 15, 1981
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: July, 1981
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

| | This Month | Yr.-to-Date | Cumulative |
|--|------------|-------------|------------|
| 11. Hours In Reporting Period | 744. | 5087. | 60600. |
| 12. Number Of Hours Reactor Was Critical | 0.0 | 0.0 | 31731.8 |
| 13. Reactor Reserve Shutdown Hours | 0.0 | 0.0 | 839.5 |
| 14. Hours Generator On-Line | 0.0 | 0.0 | 31180.9 |
| 15. Unit Reserve Shutdown Hours | 0.0 | 0.0 | 0.0 |
| 16. Gross Thermal Energy Generated (MWH) | 0.0 | 0.0 | 76531071. |
| 17. Gross Electrical Energy Generated (MWH) | 0. | 0. | 25484330. |
| 18. Net Electrical Energy Generated (MWH) | 0. | 0. | 23840053. |
| 19. Unit Service Factor | 0.0 | 0.0 | 51.5 |
| 20. Unit Availability Factor | 0.0 | 0.0 | 51.5 |
| 21. Unit Capacity Factor (Using MDC Net) | 0.0 | 0.0 | 50.1 |
| 22. Unit Capacity Factor (Using DER Net) | 0.0 | 0.0 | 48.0 |
| 23. Unit Forced Outage Rate | 100.0 | 100.0 | 41.5 |
| 24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): | | | |

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

| Forecast | Achieved |
|----------|----------|
| _____ | _____ |
| _____ | _____ |
| _____ | _____ |

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289

UNIT TMI-I

DATE August 15, 1981

COMPLETED BY C. W. Smyth

TELEPHONE (717) 948-8551

MONTH July, 1981

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

| | |
|----|----------|
| 1 | <u>0</u> |
| 2 | <u>0</u> |
| 3 | <u>0</u> |
| 4 | <u>0</u> |
| 5 | <u>0</u> |
| 6 | <u>0</u> |
| 7 | <u>0</u> |
| 8 | <u>0</u> |
| 9 | <u>0</u> |
| 10 | <u>0</u> |
| 11 | <u>0</u> |
| 12 | <u>0</u> |
| 13 | <u>0</u> |
| 14 | <u>0</u> |
| 15 | <u>0</u> |
| 16 | <u>0</u> |

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

| | |
|----|----------|
| 17 | <u>0</u> |
| 18 | <u>0</u> |
| 19 | <u>0</u> |
| 20 | <u>0</u> |
| 21 | <u>0</u> |
| 22 | <u>0</u> |
| 23 | <u>0</u> |
| 24 | <u>0</u> |
| 25 | <u>0</u> |
| 26 | <u>0</u> |
| 27 | <u>0</u> |
| 28 | <u>0</u> |
| 29 | <u>0</u> |
| 30 | <u>0</u> |
| 31 | <u>0</u> |

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-289

DOCKET NO.

TMI-1

UNIT NAME

August 15, 1981

DATE

C. W. Smyth

COMPLETED BY

TELEPHONE (717) 948-8551

REPORT MONTH July 1981

| No. | Date | Type ¹ | Duration (Hours) | Reason ² | Method of Shutting Down Reactor ³ | Licensee Event Report # | System Code ⁴ | Component Code ⁵ | Cause & Corrective Action to Prevent Recurrence |
|-----|--------|-------------------|------------------|---------------------|--|-------------------------|--------------------------|-----------------------------|---|
| 1 | 7/1/81 | F | 744 | D | 1 | | | | Regulatory Restraint Order |

F: Forced
S: Scheduled

Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit I - Same Source

OPERATIONS SUMMARY

The Unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System.

MAJOR SAFETY RELATED MAINTENANCE

During the month of July the Unit operated in the cold shutdown with restart modifications and Hot Functional Test preparations being performed. The following major maintenance activities were performed.

The Integrated Local Leak Rate Test (ILRT) was performed with satisfactory results. The testing sequence included:

- a) Reactor Building Inspection prior to commencing pressurization.
- b) Pressurize Reactor Building to 12 psi.
- c) Reactor Building Inspection at 12 psi.
- d) Pressurize Reactor Building to 50.2 psi.
- e) Stabilize pressure at 50.2 psi.
- f) Pressure drop test for twenty-four (24) hours.
- g) Superimposed leak check.
- h) Depressurize Reactor Building to one (1) psi.
- i) Hydrogen purge test.

Decay Heat River Water Pump (DR-P-1A) and motor were inspected. Tolerance readings were taken on pump parts, motor bearings were inspected, balancing was performed on the motor and pump bowl assemblies, the pump and motor were reassembled, and motor to pump was coupled. On-line balancing of the pump was performed with satisfactory results, and the pump was returned to service.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).