

## (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK: 

--	--	--	--	--	--	--

1

CON'T

0	1
7	8

REPORT SOURCE

L	6	0	5	0	-	0	3	2	4	7	0	7	0	3	8	1	8	0	7	2	8	8	1	9
60	61									68	69					74	75							80
DOCKET NUMBER											EVENT DATE							REPORT DATE						

02 | During a reactor startup following a reactor scram from 9% power on July 2, 1981,

03 | reactor coolant activity exceeded the technical specification limit of 0.2  $\mu\text{Ci/gm}$ .

0	4
---	---

 | I<sub>131</sub> dose equivalent for 10 hours, with the highest recorded activity at .2589  $\mu$ Ci/gm.

05 | This event did not affect the health or safety of the public.

0	6
---	---

08 | Technical Specifications 3.4.5, 6.9.1.9b

09		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP. SUBCODE		VALVE SUBCODE																			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24																		
		P	B	X		Z		Z	Z	Z	Z	Z	Z	Z		Z																			
LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.		ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER							
25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54						
17		8	1	0	6	3	0	3	L		0		X		Z		Z		Z		0	0	0	0	N		Y		Z		Z	9	9	9	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The high coolant activity was a result of the scram with a subsequent increase in coolant

11 fission product inventory originating from a leaking fuel element. Reactor power

1 2 was increased and the iodine activity returned to within specifications. The Unit No. 2

1 3 | fuel bundles will be sipped during the next refueling outage and the leaking bundles

1 4 | removed from the core.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
1	5	C	0 0 1	NA	A	Coolant Sampling			
ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE			
1	6	Z	Z	NA	NA				
PERSONNEL EXPOSURES		TYPE		DESCRIPTION					
1	7	0 0 0	Z	NA					
PERSONNEL INJURIES		DESCRIPTION							
1	8	0 0 0		NA					
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION					
1	9	Z		NA					
PUBLCITY		ISSUED		DESCRIPTION		NRC USE ONLY			
2	0	N		NA					

8108100173 810728  
PDR ADOCK 05000324  
S PDR

M. J. Pastva, Jr.

PHONE: 919-457-9521

## Off-Gas Analysis

Prior to this incident, the following Unit No. 2 steam jet air ejector (SJAE) off-gas level was measured:

<u>Date</u>	<u>Reactor Power %</u>	<u>Off-Gas <math>\mu</math>Ci/sec at SJAF</u>
6/18/81	96%	2.40943 E 4
6/19/81	96%	2.5227 E 4
6/30/81	86.4%	2.0104 E 4

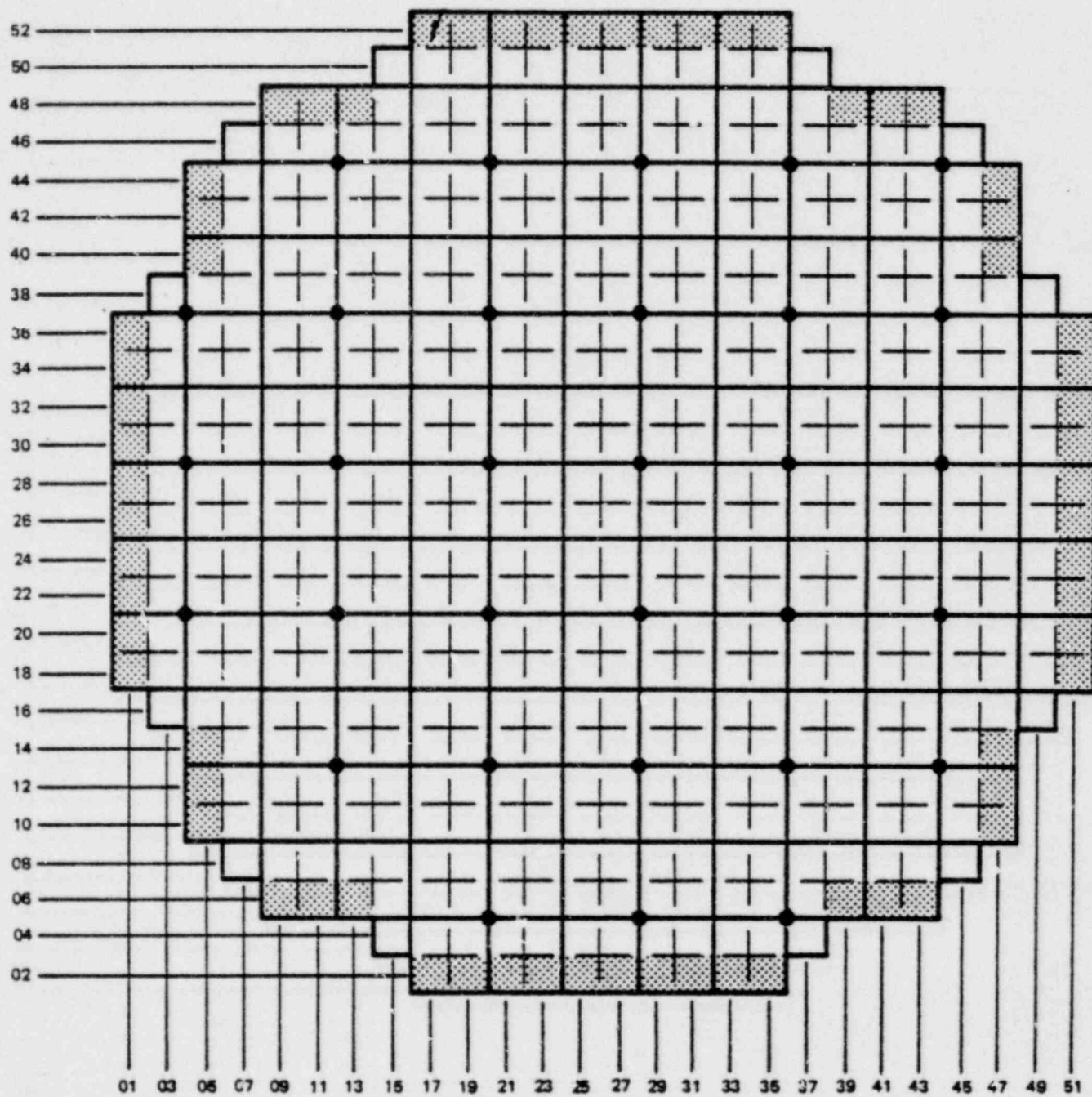
Following this incident, the following Unit No. 2 steam jet air ejector (SJAE) off-gas level was measured:

<u>Date</u>	<u>Reactor Power %</u>	<u>Off-Gas <math>\mu</math>Ci/sec at SJAE</u>
7/6/81	86.2%	1.874013 E 4
7/8/81	88.2%	2.41197 E 4
7/9/81	87.8%	2.29793 E 4

LER INFORMATION SHEET  
IODINE CONCENTRATIONS  $\mu\text{Ci/ML}$

Date	Time	% Power	I-131	I-132	I-133	I-134	I-135	I-131 Dose Equivalent
6/28/81	0835	20%	1.046E-1	9.662E-3	< MDA	7.958E-3	2.374E-3	1.051E-1
6/28/81	1250	20%	7.229E-2	9.33E-3	< MDA	2.505E-2	8.080E-3	7.328E-2
6/28/81	1750	49%	4.236E-2	1.115E-2	7.648E-3	4.617E-2	1.663E-2	4.626E-2
6/30/81	1045	84%	1.061E-3	1.142E-2	9.728E-3	4.469E-2	1.978E-2	5.807E-3
7/1/81	1355	91%	8.203E-4	1.348E-2	1.034E-2	4.480E-2	1.761E-2	5.637E-3
7/2/81	0153	Unit No. 2 Reactor Scram						
7/2/81	0639	0%	7/2/81 Unit No. 2 Reactor Startup @ 0639					
7/2/81	0724	0%	1.235E-3	1.587E-2	9.816E-3	2.152E-3	7.656E-3	5.149E-3
7/2/81	1124	Unit No. 2 Reactor Critical						
7/2/81	1145	Critical	9.669E-4	1.394E-2	6.262E-3	< MDA	3.080E-3	3.445E-3
7/2/81	1523	Unit No. 2 Reactor Power Level 9.0% Power Level						
7/2/81	2011	Unit No. 2 Reactor Scram approximately 9.0% Power						
7/2/81	2320	Unit No. 2 Reactor Startup @ 2320						
7/3/81	0115	0%	1.625E-1	2.839E-2	1.589E-1	1.572E-2	2.808E-2	2.100E-1
7/3/81	0326	4%	1.614E-1	2.398E-2	1.403E-1	< MDA	1.861E-2	2.028E-1
7/3/81	0526	10%	1.871E-1	2.618E-2	1.460E-1	< MDA	1.675E-2	2.300E-1
7/3/81	0835	10%	2.142E-1	2.627E-2	1.534E-1	< MDA	1.400E-2	2.589E-1
7/3/81	1110		1.575E-1	2.037E-2	1.170E-1	9.820E-3	1.270E-2	1.918E-1
7/3/81	1320	45%	1.323E-1	1.906E-2	8.660E-2	1.999E-2	1.137E-2	1.580E-1
7/3/81	1616	48%	4.627E-2	9.755E-3	3.705E-2	1.965E-2	8.149E-3	5.602E-2

B-101 gpm 65 Reject



Representation of 560-Bundle Core



## PRESENT R-Z EXPOSURE DISTRIBUTION

Unit 2 7-1-81

RNG	1	2	3	4	5	6	7	AVE
LVL								
12	6960.	5335.	4672.	4517.	4463.	3466.	4742.	4385.
11	11953.	9215.	8151.	7492.	7804.	6145.	8368.	7725.
10	15515.	11117.	10505.	10321.	10125.	8093.	10892.	10054.
9	16560.	12742.	11105.	11145.	10959.	8745.	11858.	10976.
8	17144.	12787.	11443.	11493.	11536.	9364.	12720.	11621.
7	17500.	14222.	11793.	12304.	11907.	10606.	13207.	11722.
6	18261.	15069.	12433.	12427.	12513.	10220.	14033.	12555.
5	18475.	15481.	12481.	13227.	12854.	10634.	14483.	13007.
4	18735.	15918.	13043.	13634.	13394.	11171.	15014.	13493.
3	18674.	15995.	13245.	13345.	13767.	11618.	15249.	13704.
2	17541.	14290.	11763.	12445.	12521.	10398.	13403.	12464.
1	12498.	8781.	7735.	7627.	7752.	6672.	8535.	774.
AVE	15226.	12743.	10704.	10771.	10909.	8476.	11710.	10714.

## PRESENT R-Z EXPOSURE DISTRIBUTION

Unit 2 7-18-81

RNG	1	2	3	4	5	6	AVE
LVL							
12	7062.	5526.	4838.	4665.	4603.	3588.	4815.
11	12223.	9473.	8420.	8147.	8039.	6373.	8498.
10	15831.	12242.	10848.	10634.	10424.	8341.	11061.
9	16891.	13286.	11467.	11466.	11221.	9019.	12077.
8	17690.	14148.	12026.	12225.	11960.	9648.	12982.
7	17867.	14598.	12171.	12538.	12224.	9874.	13391.
6	18623.	15457.	12843.	13270.	12915.	10564.	1424.
5	18825.	15865.	13090.	13559.	13168.	10924.	14675.
4	19024.	16254.	13436.	13863.	13718.	11475.	15213.
3	18889.	16242.	13604.	14170.	14102.	11938.	15447.
2	17682.	14430.	12252.	12721.	12822.	10974.	13862.
1	12549.	8867.	7410.	7741.	7946.	6824.	8623.
AVE	16096.	13032.	11033.	11257.	11092.	9130.	12072.