



Public Service Electric and Gas Company 80 Park Plaza Newark, N.J. 07101 Phone 201/430-7000

July 9, 1981


Mr. Boyce H. Grier
Director of USNRC
Office of Inspection and Enforcement
Region 1
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

LICENSE NO. DPR-75
DOCKET NO. 50-311
REPORTABLE OCCURRENCE 81-41/03L

Pursuant to the requirements of Salem Generating Station Unit No. 2
Technical Specifications, Section 6.9.1.9.b, we are submitting Licensee
Event Report for Reportable Occurrence 81-41/03L. This report is
required within thirty (30) days of the occurrence.

Sincerely yours,


R. A. Uderitz
General Manager -
Nuclear Production

CC: Director, Office of Inspection
and Enforcement (30 copies)
Director, Office of Management
Information and Program Control
(3 copies)

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The Energy People

Report Number: 81-41/03L

Report Date: July 9, 1981

Occurrence Date: 6-12-81

Facility: Salem Generating Station, Unit 2
Public Service Electric and Gas Company
Hancocks Bridge, New Jersey 08038

IDENTIFICATION OF OCCURRENCE:

Residual Heat Removal Suction Relief Valve Actuation and Failure To Fully Reset. This report was initiated by Incident Report 81-189.

CONDITIONS PRIOR TO OCCURRENCE:

Mode 4 - Rx Power 0 - Unit Load 0 MWe

DESCRIPTION OF OCCURRENCE:

During Unit cooldown to Mode 5 while Reactor Coolant system degassing operations were in progress with 21 and 24 Reactor Coolant pumps, and while 21 and 22 Residual Heat Removal pumps were in service, and the Reactor Coolant system was at 300°F and 340 psig, the Residual Heat Removal system suction piping relief valve (2RH3) lifted and relieved to the Pressurizer Relief Tank at 0105 hours, June 12, 1981. Residual Heat Removal pumps 21 and 22 were removed from service and the isolation valves to the Residual Heat Removal system (2RH1 and 2RH2) were closed. The Reactor Coolant system depressurization transient was terminated at 0210 hours, the Residual Heat Removal system was returned to service and at that time 2RH3 appeared to be reseated. Reactor Coolant system pressure was increased to 335 psig. At 0355 hours, the Reactor Coolant system leakrate to the Pressurizer Tank was identified at 13 gpm. Action Statement 3.4.7.2.b was entered. Reactor Coolant system pressure was reduced to 280 psig to again reseal 2RH3, at 0549 hours, June 12, 1981, the Unit entered mode 5 and Action Statement 3.4.7.2.b was terminated.

This constituted operation in a degraded mode in accordance with Technical Specification 6.9.1.9.b.

DESIGNATION OF APPARENT CAUSE OF OCCURRENCE:

Equipment failure. Relief valve (2RH3) failed to reseal completely following actuation.

ANALYSIS OF OCCURRENCE:

Technical Specification 3.4.7.2.b requires that with any reactor coolant system leakage greater than the limits specified, excluding pressure boundary leakage from reactor coolant system pressure isolation valves, reduce the leakage rate to within limits within 4 hours or be in at least hot standby within the next 6 hours and in cold shutdown within the following 30 hours.

July 9, 1981

CORRECTIVE ACTION:

The lift setting on relief valve (2RH3) was adjusted to 375 psig and tested. Upon the return of the Unit to service (2RH3) performed satisfactory.

FAILURE DATA:

Crosby Valve and Gage Co.

Prepared By J. J. Espey

SORC Meeting No. 81-58



Manager - Salem Generating Station