

July 29, 1981  
EF2 - 54,193

Mr. L. L. Kintner  
Division of Project Management  
Office of Nuclear Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Kintner:

Reference: Enrico Fermi Atomic Power Plant, Unit 2  
NRC Docket No. 50-341

Subject: 10CFR50, Appendix G & H  
Information Transmittal

Please find attached five (5) copies of information requested in Questions 121.16 through 121.27. The information addresses the ferritic materials used in pressure retaining components of the RCPB within General Electric's scope of supply as well as additional information concerning the materials surveillance program.

This is essentially the same as that Emery Expressed to your attention on July 24, 1981, with the following exceptions:

1. The response to Appendix H questions has been reorganized.
2. A response to Questions 121.24 f and g is included
3. The response to Questions 121.26 is given in the response to Question 121.24c which now provides a statement that the program will be updated to include a total of 108 Charpy V-notch specimens.

Sincerely,



W. F. Colbert  
Technical Director  
Fermi 2 Project

S108030285 810729  
PDR ADOCK 05000341  
A PDR

WFC/AAS:jl  
Attachments

cc: Mr. B. Little

Boo!  
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Fermi 2 Reactor Vessel  
Beltline Plate and Weld  
Information

1. Available Charpy V-notch and drop-weight NDT toughness data are presented in Tables 1, 2 and 4 for Fermi 2 beltline plates and welds. Table 2 gives supplementary transverse Charpy results which were determined for one of the Fermi 2 surveillance plates. Table 3 shows a typical Test Certificate for a Fermi 2 beltline plate.
2. The beltline layout is shown in Figure 1. This gives plate heat numbers and locations, as well as weld seam locations and identifications.
3. Copper and phosphorous values, to estimate the effects of radiation on toughness, are presented in Table 5. It can be seen in Table 5 that the analysis for Cu and P was not done for the final weld wire/flux combination weld deposit for one set of longitudinal weld seams.
4. Estimated starting (unirradiated)  $RT_{NDT}$  values are given in Table 5. They are estimated by using the data in Tables 1 and 4 in accordance with GE procedure Y1006A006 which meets the intent of ASME Code paragraph NB-2300. This procedure is explained in paragraph 5.2.4.2.2 (Attachment A) of the Fermi 2 FSAR, Amendment 23. The data base for this procedure is further clarified in response to Zimmer (ZPS-1) Q 121.15.

For the Fermi 2 beltline plates, longitudinal Charpy V-notch transition curve data are available (Table 1). Thus, the 50 ft-lb transition temperature can be determined by interpolation of these values, and by adding 30°F to the result to correct for orientation effects. These Charpy transition temperatures can then be used with the corresponding NDT data (Table 1) to determine  $RT_{NDT}$  in accordance with ASME NB-2300.

For the beltline welds all Charpy values (Table 4) are in excess of 50 ft-lb at +10°F, except for one value of 47 ft-lb. The 50 ft-lb transition temperature was taken as +10°F for those welds exceeding 50 ft-lb. The 50 ft-lb temperature for the weld with the 47 ft-lb value was estimated as +16°F by adding the correction factor of 2°F/ft-lb (Y1006A006). Since NDT data are not available for these welds, an assumption of -50°F for NDT was made. Justification for this assumption is given in Item 2. This NDT value was used with the Charpy transition temperatures to determine  $RT_{NDT}$  in accordance with ASME NB-2300.

5. Estimated end-of-life (EOL)  $RT_{NDT}$  values (for  $\frac{1}{2}$  thickness location from the vessel inside diameter) are given in Table 5. These values are slightly lower than those previously reported in Amendment 23 of the FSAK, because of a correction to the predicted fluence. The estimations are in accordance with NRC Regulatory Guide 1.99 Rev. 1. Where Cu and P content analyses were not available for the

deposited wire/flux combination, the maximum RT<sub>NDT</sub> shift ( $\Delta RT_{NDT}$ ) is conservatively assumed in accordance with Reg. Guide 1.99 Rev. 1.

6. Charpy V-notch upper shelf toughness was not a requirement when the Fermi 2 vessel was manufactured. Thus, such data is not available for the Fermi 2 beltline welds, but is available for the plates as shown in Tables 1 and 2.

A very conservative assumption of 65% factor on longitudinal upper shelf can be applied to the results of Table 1 in order to estimate transverse orientation upper shelf. (Table 2 shows that a higher factor may be justified.) The factor of 65% (from MTEB 5-2) would result in a longitudinal requirement of 115 ft-lb in order to meet the 10CFR50 Appendix G value of 75 ft-lb upper shelf. This value is met by all plates in Table 1 except C4564-1, which very narrowly misses. However, since the Cu content of this plate is only 0.09% (Table 5) a reduction of upper shelf of only 10% at EOL is conservatively predicted by Reg. Guide 1.99 Rev. 1. Combining these 2 conservative factors of 65% and 10% results in an initial longitudinal upper shelf value of only 85 ft-lb to meet the goal of 50 ft-lb transverse upper shelf at EOL. This value of 85 ft-lb, as calculated in the following equation, is exceeded by plate C4564-1.

$$50 = .65(L) - (.10) [.65 (L)]$$

(where L is the longitudinal upper shelf value at start of life)

As seen in Table 4, upper shelf toughness values are not available for Fermi 2 welds. However, all Charpy results at the test temperature of +10°F are in excess of 75 ft-lb except for one weld material (Heat 12008/Lot 3833). It is expected that further testing at higher temperatures would have revealed an upper shelf in excess of 75 ft-lb for this material also. Evidence in this respect is presented in Tables 6 through 15 which show weld procedures and upper shelf toughness results for similar submerged arc weld materials. All upper shelf (~ 100% shear results in Tables 12 through 15) are in excess of 75 ft-lb. These welds are considered to be representative of the Fermi 2 weld in question (seams 2-307 A, B, C) since the welding processes (generally tandem wire submerged arc for the bulk of the weld), post weld heat treatment, and weld materials are similar (as shown in Tables 8 through 11). Particular attention should be given to the LaSalle 1 results, since these welds were made by the same vendor (Combustion Engineering) and with the exact same weld procedure (Tables 8 and 9) as for the Fermi 2 weld. The LaSalle 1 surveillance program weld material 1P3571/3958 in Table 12 gave values less than 75 ft-lb at +10°F, but further testing at +200°F revealed an upper shelf of 110 ft-lb.

7. Drop-weight NDT values for the Fermi 2 weld materials were not determined by testing. However, evidence for a conservative assumption of -50°F is found in Table 12, based on the LaSalle 1 result. All values of NDT are -50°F or lower. Further results in this respect are also shown in Tables 13, 14 and 15 (CBIN welds) and verify NDT

values of -50°F and lower, except for one case. This case (1P6484/0156 for Laguna Verde 2) is considered to be non-representative of Fermi 2, because of the relatively low Charpy test value (17 ft-lb) at +10 and 0°F for this material.

8. The RT<sup>NDT</sup> values for weld heat affected zones (HAZ) are assumed the same as for the base material. Weld procedure qualification test requirements for HAZ toughness indicate this assumption is valid. This is also supported by the following technical publications, which conclude that the HAZ toughness for these materials is actually superior to that of the base material: (a) T. U. Marston and W. Server, "Assessment of Weld Heat-Affected Zones in a Reactor Vessel Material" Journal of Engineering Materials and Technology, July 1978, Vol. 100, page 267, (b) D. A. Canonico, "Significance of Reheat Cracks to the Integrity of Pressure Vessels for Light-Water Reactors," Supplement to the Welding Journal, May 1979, page 137-S.
9. Refer to Fermi 2 FSAR Table 5.2-9, Amendment 23 for justifications regarding toughness testing calibration and qualification of testing personnel.
10. Weld material toughness test coupons were made with the exact same weld filler metal and procedure as in the actual vessel weld. However, these weld deposits were not necessarily made on the exact same heat of base plate as in the vessel. Base plate of the same specification was employed for this purpose. This small difference in base plate would not effect the testing of the weld metal since the Charpy specimen would be in the weld metal. Toughness testing of the exact base plates in the vessel was done separately.
11. Cross-Reference of paragraphs for resolution of open items:

<u>EF-2 Question</u>	<u>This Submittal</u>	<u>10CFR Part 50 Appendix G</u>
121.16(b)	8	I.B
121.17	7	III.A
121.18	1-6	III.C.1, IV.B
121.19	10	III.C.2
121.22	6	IV.B
--	3	III.B.4



## FERMI 2 REACTOR VESSEL

NON-BELTLINE INFORMATION

1. Limiting  $RT_{NDT}$  values which affect vessel testing and operation are shown in the FSAR (paragraphs shown on Attachment A). A sentence has been added in paragraph 5.2.4.2.2 of Attachment A to further clarify that these are the  $RT_{NDT}$  values for the limiting vessel locations and materials that affect testing and operating limits. The other materials in the vessel, which meet specific toughness requirements, do not affect the pressure-temperature curves.
2. The estimation procedures for these  $RT_{NDT}$  values are in accordance with GE procedure Y1006A006, and are also explained in paragraph 5.2.4.2.2 of the FSAR. As with the beltline, the data base for this procedure is further clarified in response to Zimmer (ZPS-1) Q121.15. A more specific explanation follows:
  - a) Non-beltline Plates - Both longitudinal Charpy values over the full temperature range and NDT values are available.  $RT_{NDT}$  was evaluated the same as for the beltline plates. The limiting plate (highest  $RT_{NDT}$ ) is in the bottom head (Heat No. C4504-2) with an NDT of  $+10^{\circ}\text{F}$  and lowest Charpy values of 40 ft-lb. at  $+40^{\circ}\text{F}$  and 76 ft-lb at  $+110^{\circ}\text{F}$ . Linear interpolation estimates the longitudinal 50 ft-lb. temperature as  $+60^{\circ}\text{F}$ . Adding  $30^{\circ}\text{F}$  for orientation correction and subtracting  $60^{\circ}\text{F}$  (NB-2300) gives an  $RT_{NDT}$  of  $+30^{\circ}\text{F}$ .
  - b) Vessel and head closure flange materials had NDT values of  $+10^{\circ}\text{F}$  (or possibly lower - no break at  $+20^{\circ}\text{F}$ ) and lowest Charpy values of 95 ft-lb. at  $+40^{\circ}\text{F}$  and 167 ft-lb. at  $+40^{\circ}\text{F}$ . The correction of  $30^{\circ}\text{F}$  was added to  $+40^{\circ}\text{F}$  for orientation and  $60^{\circ}\text{F}$  was subtracted to give an  $RT_{NDT}$  of  $+10^{\circ}\text{F}$ .
  - c) Feedwater nozzles had a maximum specified NDT value of  $+40^{\circ}\text{F}$ , and Quality Assurance records show no deviations in this respect. The lowest Charpy value for these forgings is 38 ft-lb. at  $+10^{\circ}\text{F}$ . Adding  $2^{\circ}\text{F/ft-lb.}$  gives an estimated 50 ft-lb. temperature of  $+34^{\circ}\text{F}$ . Adding  $30^{\circ}\text{F}$  for orientation and subtracting  $60^{\circ}\text{F}$  (NB-2300) gives an  $RT_{NDT}$  of  $+4^{\circ}\text{F}$ , as determined by Charpy. Thus, the  $RT_{NDT}$  is set equal to the NDT value of  $+40^{\circ}\text{F}$ .
  - d) Closure Studs - The lowest Charpy values at  $+10^{\circ}\text{F}$  are 50 ft-lb. and 33 mils lateral expansion. Thus, in accordance with NB-2300 the lowest service temperature is  $+10^{\circ}\text{F}$ .

e) Non-beltline Welds - The purchase specification required Charpy tests at +10°F or drop-weight NDT of +10°F or lower. Quality Assurance records show no deviations in this respect. Charpy requirements at +10°F were for 30 ft-lb. average with no single value less than 25 ft-lb. Assuming 25 ft-lb. at +10°F as the limiting case, and adding 2°F/ft-lb. gives an estimated 50 ft-lb. temperature of +60°F. Subtracting 60°F (NB-2300) gives an RT<sub>NDT</sub> of 0°F. Data presented in support of the beltline welds indicate NDT values much below 0°F. Thus, the RT<sub>NDT</sub> value is taken as 0°F.

3. Refer also to paragraphs 7 through 10 of the Beltline section of this submittal, since they also apply to non-beltline materials and testing.
4. Cross-reference of paragraphs for resolution of open items:

<u>EF-2 Question</u>	<u>This Submittal</u>	<u>10CFR Part 50 Appendix C</u>
121.16 (a)	2.e	I.B
121.16 (b)	3	I.B
121.17	2	III.A
121.18	1, 2	IV.A.1
121.21	2.d	IV.A.3
	3	III.B.4

FERMI 2 MAIN STEAM  
PIPING AND FERRITIC VALVES  
(MSIV AND SRV)

1. The Fermi 2 main steam piping was procured to the USAS B31.7, Class I, 1969 Code, which did not require toughness testing. However, data are supplied in Tables 16 through 21 to show that the Fermi 2 NSSS supply steam pipe materials would possess adequate toughness. This is concluded from available toughness information for Fermi 2 in Tables 16, 17, and 18, and from the fact that similar materials (as shown in Tables 19, 20, and 21) have data showing adequate toughness per more current 10CFR50 Appendix G Main Steam Pipe requirements.

No toughness results are available for the 26" pipe. However, the material is pipe fabricated from A516 Grade 70 plate which is a tough carbon steel melted to fine-grain practice for low temperature service. Charpy V-notch data for this material in Tables 17 and 19 verify this toughness. Furthermore, Charpy keyhole data are available at -50°F for the Fermi 2 26" elbows fabricated from A516 Grade 70. A Charpy transition curve shift of about 60°F increase should give an estimation of Charpy V-notch results for these elbows (Reference: W. S. Pellini, ASTM Spec. Tech. Publ. 158 page 222, 1954). Thus, they should have adequate toughness at about +10°F even better toughness at the more current test temperature of (Table 19). This transition temperature shift and argument also should apply to the sweepolet Charpy keyhole results in Table 16.

Note that the material and pipe suppliers in Table 19 are the same as for the Fermi 2 26" pipe (Table 16).

2. Fermi 2 Safety Relief Valves (SRV) are in compliance with 10CFR50 Appendix G since they are exempted by the ASME Code from toughness testing because of their 6-inch size.
3. Fermi 2 Main Steam Isolation Valves (MSIV) were exempt from toughness testing at the time of purchase. They do not see significant pressures at temperatures below that of steam.

Typical information is given in Table 22 for Fermi 2 MSIV's. Toughness data on similar materials for MSIV's on other projects, where toughness testing was done, is attached on Tables 23 and 24. In fact, Table 23 gives A216 WCB base metal, weld metal, and HAZ toughness results from the Weld Procedure Qualification used for Fermi 2. In some cases (Table 24), the materials and valves vendor are the same as for Fermi 2. These data demonstrate the capability of the Fermi 2 MSIV materials to meet current toughness requirements.

Further evidence of toughness for SA-105 forgings (MSIV bonnet, or cover, material) can be found in the July 1978 issue of Metal Progress, pages 35-39. This article shows Charpy V-notch toughness in excess of 25 mils at +40°F and NDT values no greater than -10°F for SA-105 material normalized at 1565°F for 4 hr. and air cooled after forging.

4. Cross-reference of paragraphs for resolution of open items:

<u>EF-2</u> <u>Question</u>	<u>This</u> <u>Submittal</u>	<u>10CFR Part 50</u> <u>Appendix G</u>
121.20	1, 2, 3	IV.A.3

Compliance With Appendix H, 10 CFR Part 50

Question 121.23

Response:

A sketch of the beltline of the reactor vessel showing the location of all of the beltline plates and welds is shown in Figure 1. The azimuth angle giving the location of the capsules is given in the response to Question 121-24(c).

Question 121.24

Response:

The weld material has a Cu content of 0.32 wt. % (Table 5) and is very close to being the limiting material in the vessel beltline. (It may actually be limiting since the Cu for the limiting material in seams 2-307 is not known, but is probably lower than 0.32.) The plate materials are very close to being the limiting beltline plates (only 8 to 10°F lower EOL RT<sub>NDT</sub> than the limiting plate).

The surveillance specimens were not taken from alongside the ASME NE-2300 specimens. This is not considered critical since they are just as representative of the material in the vessel as the NB-2300 specimens. This requirement has been dropped from the current proposed revision (Nov. 1980).

- (a&b) Fermi 2 surveillance specimen plate and weld materials are identified, with properties and predicted radiation effects, in Tables 1 through 5. The weld procedure is given in Tables 6 and 7 and represents weld seam 15-308.
- (c) The actual specimens in each capsule are the following:

	<u>Tensile</u>	<u>Present Program Charpy V-notch</u>
Capsule 1 (Azimuth 300°)	2 BM Long 2 WM 2 HAZ	8 BM, Long 8 WM 8 HAZ
Capsule 2 (Azimuth 120°)	3 BM Long 3 WM 2 HAZ	8 BM, Long 8 WM 8 HAZ
Capsule 3 (Azimuth 30°)	3 BM Long 2 WM 3 HAZ	12 BM, Trans. 12 WM 12 HAZ



The specimens indicated above are as the program is presently constituted. Capsules 1 and 2 will be updated to include 12 each of Charpy V-notch specimens of base metal (longitudinal), weld metal and heat-affected zone.

- (d) Location given in response to Question 121.24(c).

The attachment method of the capsules is in accordance with GE Drawing 922D218. The assembly is attached to mounting brackets (upper and lower) and a bolt at approximately the center of the assembly can be adjusted to secure the holder firmly against the top and bottom brackets.

- (e) The lead factor is the ratio of the flux greater than 1 MeV at the surveillance sample, divided by the flux greater than 1 MeV at the point of greatest flux in the vessel. For Fermi 2 this value is 1.4. This lead factor has arbitrarily been reduced by a factor of 2 in order to improve the probability that vessel fluxes estimated from surveillance data will be underestimated. The lead factor then becomes 0.7.

#### Note

The lead factor is the relationship between the measured flux/fluence at the surveillance sample and the peak flux/fluence at 1/4 depth into the vessel wall. This relationship has two variations. One variation is the radial variation from a position inside the reactor pressure vessel wall to a radial position at 1/4 thickness of the vessel wall. The second variation is the variation of the flux as a function of angle from a position adjacent to the surveillance sample to the position of the peak flux.

The peak fluence at 1/4 t was calculated using a one-dimensional program and applying a peaking factor to adjust for the maximum point in the angular direction. In addition to the peaking factor, a safety factor is applied to the analysis to insure that the calculated peak is a maximum. Attached is an updated sheet for Table 4.3-2 for the FSAR Chapter 4 that provides the current 251-764 neutron fluence calculations including the data at 1/4 t in the vessel.

Not all of the analysis required is available to define the fluence at the surveillance sample. The radial value can be selected from the one-dimensional analysis. However, the angular variation from the surveillance sample at 30° to the peak is not well defined.

- (f) The materials surveillance capsules will be loaded prior to fuel loading.
- (g) The material surveillance program assumes a 40-year life and 80% capacity factor, thus the capsules withdrawal will be:

Withdrawal

Capsule #1	8 full-power years
Capsule #2	24 full-power years
Capsule #3	Standby

Due to uncertainty in capacity factor, the calendar withdrawal schedule cannot be stated with any confidence.

Question 121.25

Response:

See response to Question 121.24.

Question 121.26

Response:

See response to Question 121.24(c)

Question 121.27

Response:

Each capsule also includes a Fe, Ni, and Cu flux wire. A separate neutron dosimeter is attached at Azimuth 30° and contains 3 Cu and 3 Fe flux wires, at Capsule 3.

AAS/br  
7/29/81

Table 1

FERMI 2, BELTLINE PLATE TOUGHNESS DATA  
(SA-533 GRADE B, CLASS 1 - LUKENS)

## CHARPY V-NOTCH TOUGHNESS

	Plate Heat No.	Dropweight Temp	Orientation (L or T)	Charpy Temp	Energy (ft-lbs)	Lat. Expansion Mils	% Shear
Lower Intermediate Shell	C4504-1	-20°F	L	-80°F	11, 10	7, 7	0, 0
				-40°F	30, 36, 23	21, 26, 17	10, 10, 10
				+10°F	60, 45, 59	44, 32, 42	25, 15, 25
				+40°F	86, 74, 63	59, 52, 45	40, 30, 30
				+110°F	104, 95	70, 72	95, 90
				+160°F	113, 116	85, 83	100, 100
	B8614-1 (Also in surveillance program)	-20°F	L	-80°F	5, 10	5, 7	0, 0
				-40°F	43, 35, 27	32, 20, 21	5, 5, 5
				+10°F	62, 64, 56	41, 45, 40	20, 25, 20
				+40°F	86, 75, 70	62, 54, 50	40, 35, 30
				+110°F	112, 110	81, 79	95, 90
				+160°F	125, 135	86, 90	100, 100
	C4574-2 (Also in surveillance program)	-30°F	L	-80°F	8, 16	6, 13	0, 0
				-40°F	34, 32, 27	25, 24, 20	10, 10, 5
				+10°F	48, 49, 60	36, 37, 43	15, 15, 20
				+40°F	76, 63, 69	56, 47, 51	30, 20, 25
				+110°F	98, 103	72, 76	95, 95
				+160°F	121, 119	85, 82	100, 100

Table 1 (Continued)  
FERMI 2, BELTLINE PLATE TOUGHNESS DATA

	Plate Heat No.	Dropweight NDT	Orientation (L or T)	Charpy Temp	Energy (ft-lbs)	Lat. Expansion Mils	% Shear
Lower Intermediate Shell	C4568-2	-30°F	L	-80°F	10, 18	5, 13	0, 0
				-40°F	30, 38, 30	22, 27, 23	5, 5, 5
				+10°F	46, 67, 63	37, 54, 47	15, 30, 25
				+40°F	76, 85, 61	58, 61, 45	40, 50, 35
				+110°F	106, 102	75, 72	95, 95
				+160°F	116, 122	89, 87	100, 100
Lower Shell	C4540-2	-10°F	L	-80°F	7, 9	6, 8	0, 0
				-40°F	30, 44, 30	23, 34, 25	5, 10, 5
				+10°F	64, 76, 74	49, 58, 56	30, 30, 30
				+40°F	87, 84, 97	69, 63, 72	40, 40, 50
				+110°F	115, 119	85, 84	85, 85
				+160°F	144, 146	90, 92	100, 100
	C4560-1	-10°F	L	-80°F	14, 11	12, 8	0, 0
				-40°F	59, 53, 38	45, 51, 30	20, 20, 15
				+10°F	85, 79, 99	62, 60, 72	30, 30, 35
				+40°F	90, 121, 109	69, 78, 74	50, 65, 60
				+110°F	160, 144	88, 88	100, 90
				+160°F	158, 153	90, 88	100, 100
	C4554-1	-10°F	L	-80°F	13, 25	8, 18	0, 1
				-40°F	35, 40, 42	27, 30, 32	20, 20, 20
				+10°F	59, 65, 68	45, 49, 51	30, 30, 30
				+40°F	79, 77, 87	59, 61, 64	35, 35, 35
				+110°F	131, 118	88, 89	100, 90
				+160°F	137, 127	90, 87	100, 100

METALLURGY LABORATORY  
CHARPY IMPACT TEST DATA

Table 2  
TRANSVERSE RESULTS  
(SURVEILLANCE PLATE)

Requestor: John Copeland Responsible Engineer: \_\_\_\_\_  
Charge No.: P7830 Date: 5/18/79  
DRF No.: \_\_\_\_\_ EWA No.: EA 302-01  
Material Condition: SA533B plate HT # C4574-2

Specimen Identification	Bath Medium	Test Temperature °F	Energy Absorbed Ft-Lb	Lateral Expansion Mils	Remarks
63 B	MeOH	-20°	22.0	17.5	1 %
63 U	"	10°	32.0	22.5	5 %
63 L	"	10°	35.0	27.5	5 %
63 J	"	40°	50.0	35.5	10 %
63 P	"	40°	52.5	41.5	10 %
63 D	AIR	65°	64.0	47.0	30 %
63 A	"	65°	55.0	42.5	30 %
63 M	H <sub>2</sub> O	102°	75.0	60.5	50 %
63 C	"	119°	108	75.0	100 %
63 Y	"	119°	88	66.0	55 %
63 T	"	201°	112.5	83.5	100 %
63 E	"	202°	108.5	79.0	100 %

TINUS OISEN Model 74 Calibrated per TP-509.0294  
DATE LAST CALIBRATED - 7/26/78  
Centered per 2.1.7.2 Rev 0 Temp Control - Calibrated Thermocouple

Test Procedure No. CMS 2.1.7.2 Rev 0

Specimen Size 1 cm x 1 cm

S/N Tester 119073

Specimen Orientation TRANSVERSE

Calibration File No. 509-1

S/N Lateral Expansion gage 16602

Performed by LSI

Level 3



Table 3  
FERMI 2 TYPICAL BELTLINE PLATE (SURVEILLANCE PLATE C4574-2)

CHASER:  10 Combustion Engr Inc. Chattanooga Div. Attn: Jack Michael, Purch Dept	<b>LUKENS STEEL COMPANY</b> <small>COATESVILLE, PA. 19330</small> <b>TEST CERTIFICATE</b>			DATE: <b>7-9-68</b> CONSIGNEE:	FILE NO. <b>1771</b>  <div style="text-align: right; font-family: cursive;">           V-70711            Nuclear 70063            V-76063         </div>
	MILL ORDER NO. <b>25720-2</b>	CUSTOMER P.O. <b>48-37377</b>	NB 62958 FJ 7208		

SPECIFICATIONS  
 Combustion Spec P3F12(C) 9/25/67 (SA-533-65 Gr. B Class 1) ASME Sec 3 Class A FEX 20000 Req. # 2557  
 O.K. / O.K.

CHEMICAL ANALYSIS																
MELT NO	C	Mn	P	S	CU	SI	Fe	Ca	Mg	V	Ti	Al	B			
C4578	23	1.33	010	015		24	51		56					F.O.P.	V.I.P.	Steel
Slab 1	20	1.35	012	015		24	53		57							
C4568	23	1.32	011	016		24	61		56							
Slab 2	20	1.29	012	016		23	60		54							
C4574	22	1.36	013	016		24	55		54							
Slab 2	22	1.34	014	016		25	55		52							

PHYSICAL PROPERTIES										DESCRIPTION
MELT NO	SLAB NO	TENSILE	YIELD	ELONG	REDUCED	IMPACTS				
C4578	1	728	960 939	22	G-3704-2					1-291-7/16 x 133-1/2 x 7-3/8"
C4568	2	766	969 985	24	G-3705-3					1-231-5/16 x 172-3/4 x 7-3/8"
C4574	2	693	925 917	24	G-3705-2					1-
<div style="font-family: cursive; font-size: 1.2em;">2667</div>										
Tests heated 1550-1650°F. held 4 hrs max and program cooled per C.E. cooling rate for 7-3/8" Gauge plate. Then tempered 1200-1300°F. held 4 hrs max and air cooled.										
Tests stress relieved 1125-1175°F. held 40 hrs and furnace cooled within a rate of 4 hrs min to 600°F. Plates furnished in as rolled temper.										

We hereby certify the above figures are correct as contained in the records of the company.

SUPERVISOR TESTED: \_\_\_\_\_

Table 3 (continued)

G. E. St. Cin

COMBUSTION ENGINEERING, INC.

METALLURGICAL RESEARCH AND DEVELOPMENT, DEPT

MATERIALS CERTIFICATION REPORT

MATERIAL SPECIFICATION P3F12(c)

CONTRACT NO. 2667

VENDOR Lukens Steel Company

JOB NO. V-70637-002

HEAT NO. C 4574-2

CODE NO. G-3705-2

MATERIAL DESCRIPTION 231-5/16" x 172-3/4" x 7-3/8" Lower Intermediate Shell

## MILL CHEMICAL ANALYSIS

TYPE	C	Mn	P	S	Si	Ni	Cr	Mo	Cu	Cb	Ca
	.22	1.34	.014	.015	.26	1.55		.52			

## MECHANICAL TESTS

TEST NO.	GAUGE	TEST TEMPERATURE °F	YIELD STRENGTH, KSI	ULTIMATE TENSILE STRENGTH, KSI	ELONG. IN 2"	REDUCTION OF AREA %
VLT-A	.505	RT	71.6	95.0	25.0	70.0
VLT-B	.505	RT	71.6	94.0	25.0	64.0

## IMPACT AND/OR FRACTURE TESTS

TYPE	TEMP. °F	VALUES	VALUES	TEMP. °F	VALUES	NDT
Charpy V Notch		Ft/Lbs	%Shear	Hit	Drop Weights	
				Lat. Exp.		
	-80	8.0	0	6		
	-80	16.0	0	13	-40	1-F
	-40	34.0	10	25	-30	1-F
	-40	32.0	10	24	-20	2-NF
	-40	27.0	5	20	0	1-NF
	+10	48.0	15	36		
	+10	49.0	15	37		
	+10	50.0	20	43		
	+40	76.0	30	56		
	+40	63.0	20	47		
	+40	59.0	25	51		
	+110	98.0	95	72		
	+110	103.0	95	76		
	+160	121.0	100	85		
	+160	119.0	100	82		

## ADDITIONAL DATA INCLUDING HEAT TREATMENT:

(a) 1550-1650°F 4 hours water quenched.

(b) 1225° ± 25°F 4 hours.

(c) 1150°F ± 25°F 40 hours furnace cooled to 600°F.

The CVN impact specimens were taken parallel to the major rolling direction of the plate at the 1/4T level, and were notched perpendicular to the plate surface.

The tensile specimens were taken in accordance with ASTM A-20-68.

The above tests were witnessed by G. E. Representative, S. G. Hall.

Form 1-100

cc: P. Webb (2)✓

J. Brasfield

T. B. Burton

T. H. Cullen

K. E. Lorentz, Jr.

We hereby certify that the foregoing data is a true copy of the data furnished us by the producing mill, and that resulting from tests performed in the Combustion Engineering Laboratories.

COMBUSTION ENGINEERING, INC.

BY

S. G. Hall

DATE May 19, 1969

Table 4

FERMI 2 BELTLINE WELD TOUGHNESS DATA, POST WELD 1150°F FOR 40 HR. TYPICAL,  
SUBMERGED ARC WELDING - B-4 MODIFIED WIRE WITH LINDE FLUX

Weld Seam	Type	Heat #	Lot # or Flux #	Drop-Weight NDT °F	Charpy Toughness			
					Charpy Temp °F	Charpy Energy ft-lbs	Lat. Expansion Mils	% Shear
2-307 A, B, C	B-4 Mod.	13253	3833	(Linde 1092)	NA	+10	79, 79, 82	NA
		12008	3833		NA	+10	62, 47, 62	NA
15-308 A, B, C, D	B-4 Mod.	*33A277	*3878	(Linde 124)	NA	+10	83, 94, 87	NA
1-313	B-4 Mod.	10137	3999	(Linde 0091)	NA	+10	101, 108, 107	NA

NA = Not Available

\*This material is also in the surveillance program.

Table 5

FERMI 2BELTLINE RADIATION  $\Delta RT_{NDT}$  & EOL (END-OF-LIFE)  $RT_{NDT}$ Peak EOL Fluence =  $1.1 \times 10^{18}$  n/cm<sup>2</sup> ( $\frac{1}{2}$ T wall)

## A. Plates - Beltline

HEAT NO.	WT. % Cu	WT. % P	Y1006A006 START $RT_{NDT}$ (°F)	REG. GUIDE 1.99 EXTRAP. $\Delta RT_{NDT}$ (°F)	EOL $RT_{NDT}$ (°F)
C4564-1	.09	.010	-12	20	8
B8614-1*	.12	.011	-20	32	12
C4574-2*	.10	.014	-16	30	14
C4568-2	.12	.012	-12	33	21
C4540-2	.08	.010	-10	17	7
C4560-1	.11	.010	-10	27	17
C4554-1	.12	.011	-10	32	22 Limiting Plate

## B. Welds - Beltline

SEAM	HEAT/LOT	WT. % Cu	WT. % P	Y1006A006 START $RT_{NDT}$ (°F)	REG. GUIDE 1.99 EXTRAP. $\Delta RT_{NDT}$ (°F)	EOL $RT_{NDT}$ (°F)
2-307A,B,C	13253/3833	(.07	.013)**	-50	(110)**	60
	12008/3833	.13	.010)	-44	(110)	66 Limiting Weld
15-308A,B,C,D	33A277/3878*	.32	.016	-50	106	56
1-313	10137/3999	.23	.016	-50	76	26

\* This material is also in the surveillance program.

\*\* Bare wire analysis only; as deposited wire/flux combination analysis not done.  
Therefore, maximum  $\Delta RT_{NDT}$  is assumed.

Table 6

## BELTLINE WELD PROCEDURE FOR FERMI 2 SURVEILLANCE PROGRAM

COMBUSTION ENGINEERING, INC.  
NUCLEAR QUALITY ENGINEERING  
SURVEILLANCE PROGRAM TEST REPORT

Customer	<u>General Electric Company</u>	Contract	<u>2667</u>
Material	<u>SA-533 Gr. B , Cl. 1</u>	Job No.	<u>V-70711</u>
Dwg. No.	<u>E-232-902</u>	Seam No.	<u>15-308</u>
Detail Weld Procedure No.	<u>SAA-4-0</u>	TK.	<u>7-3/8"</u>
Code No.	<u>G-3705-1 and G-3705-2</u>		
Filler Metal (Type, Ht. and Size)	<u>St , 33A277 , 1/8"</u>		
Flux (Type and Lot)	<u>Linde 124 Lot # 3878</u>		
Post Weld Heat Treatment:	Temp. <u>1150°F ± 25°</u>	Hours	<u>40-3/4</u>

Weld Dept. or Shop Nuclear ShopWelders Symbols WP - YY - MU - TV - TN

## Non-Destructive Tests

MT M&P 2.4.2.4(d) Add. 1(a) , 3(a)PT                     RT M&P 2.4.1.3(b) Add. 1(a) , 2(a) , 4(a)UT                     

We certify that the statements in this Report are correct as contained in the Records of the company.

COMBUSTION ENGINEERING, INC.

By: C. E. WhiteDate: 5/8/72



Table 7  
DETAIL WELD PROCEDURE FOR FERMI 2 SURVEILLANCE PROGRAM

COMBUSTION ENGINEERING, INC.  
NUCLEAR COMPONENTS DEPARTMENT  
CHATTANOOGA, TENNESSEE

CONTRACT NO.:  
DRAWING NO.:  
WELD NO.:  
REFERENCES: M&P 6.1.1.2(c),  
M&P 4.3.8.5(b)  
SAA-33-29  
QSAA-11A(1), QMA-11A(1)F4  
Non-destructive Testing:

P.T.  
M.T.  
R.T.  
U.T.

WELDING CONDITIONS:

Electrode Type & Size

Filler Metal Type & Size

Flux Type & Size

\*Welding Current & Polarity

\*Arc Voltage

\*Travel Speed (in/min.)

Shield Gas Type & Flow

Gas Cup Size

Gas Cup to Work Distance

Other

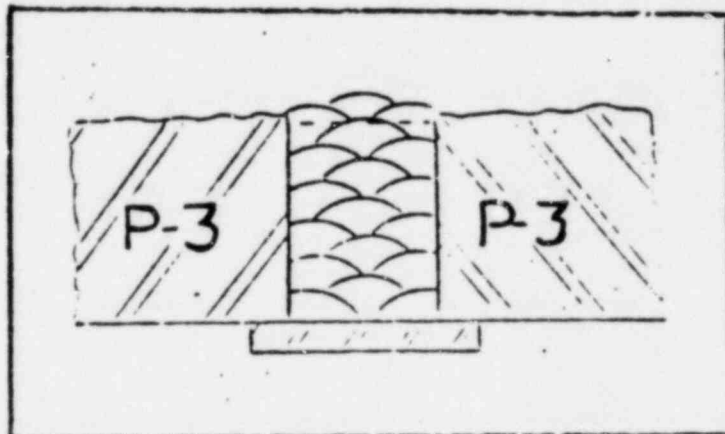
\*± 10% of Value or Range

WELDING POSITION: Flat (Vertical Progression)

Preheat: 250 °F. Hold Unspecified. Until P.W.H.T.  
Interpass: 500 °F.

Post-weld heat treatment: 1150 °± 25 °F hold one hour/inch  
thickness of weld  
Intermediate P.W.H.T. 1100 °± 50 °F, hold 15 minutes

DETAIL WELDING PROCEDURE  
NO.: SAA-4 Rev. 0  
DATE:



1/8"Ø MIL. B-4

Linde 124, 20 X 150

550 AC

33

12-13

Table 8  
DETAIL WELD PROCEDURE FOR FERMI 2 BELTLINE SEAMS 2-307A,B,C

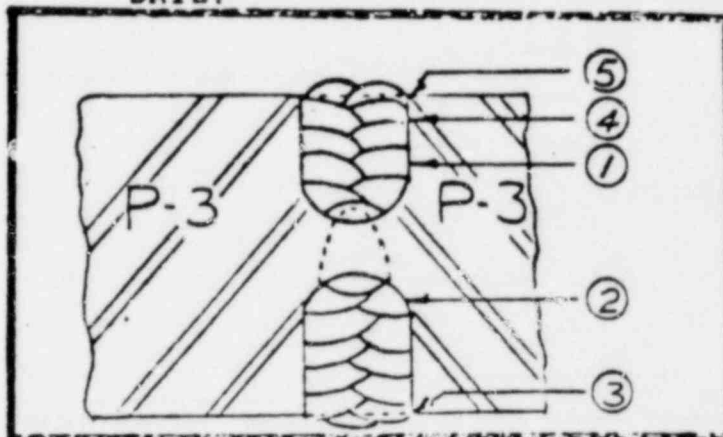
**COMBUSTION ENGINEERING, INC.**  
**NUCLEAR COMPONENTS DEPARTMENT**  
**CHATTANOOGA, TENNESSEE**

CONTRACT NO.:  
DRAWING NO.:  
WELD NO.:  
REFERENCES: M&P 6.1.1.2(c),  
M&P 4.3.8.5(b), SAA-33-27

DETAIL WELDING PROCEDURE  
NO.: TSAA-2(A) Rev. 0  
DATE:

Non-Destructive Testing:

P.T.  
M.T.  
R.T.  
U.T.



WELDING CONDITIONS:

Electrode Type & Size

See attached sheet.

Filler Metal Type & Size

Flux Type & Size

\*Welding Current & Polarity

\*Arc Voltage

\*Travel Speed (in/min.)

Shield Gas Type & Flow

Gas Cup Size

Gas Cup to Work Distance

Other

\*+ 10% of Value or Range

WELDING POSITIONS: Flat

Preheat: 250 °F. Hold

~~DO NOT WELD~~

Until P.W.H.T.

Interpass: 500 °F.

Post-weld heat treatment: 1150 °+ 25 °F hold one hour/inch  
thickness of weld.

Intermediate P.W.H.T. 1100 °+ 50 °F, hold 15 minutes.

Table 8 continued

## DETAIL WELDING PROCEDURE

No.: TSAA-2(A) Rev.0

Sheet: 2 of 2

<u>WELDING SEQUENCE</u>	<u>TRAVEL</u>	<u>AMPS*</u>	<u>VOLTS*</u>
*1st Pass - O.D. 3/16"Ø Mil.B4 Mod.	13 IPM	700 AC	31
NOTE: Use copper backing bar		Single Arc	
1st Increment - O.D. 3/16"Ø Mil.B4 Mod.	13 IPM	650 AC	31
		Single Arc	
(1) O.D. to 1½" Level 3/16"Ø Mil. B4 Mod.	13 IPM	650 AC	31
		Single Arc	
(2) I.D. to 1" Level 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC	31
		Tandem Arc	
(3) *Remainder - I.D. 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC	31
		Tandem Arc	
(4) O.D. to 3" Level 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC	31
		Tandem Arc	
(5) *Remainder O.D. 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC	31
		Tandem Arc	
Backweld if required			
1/4"Ø E-8018 C-3 (Flat Only)		325-375 DC-RP	25
or			
3/16"Ø E-8018 C-3		210-260 DC-RP	25

\*Flux Linde 1092 65 x 200

COMBUSTION ENGINEERING, INC.  
NUCLEAR COMPONENTS DEPARTMENT  
Chattanooga, Tennessee

CONTRACT NO.:  
DRAWING NO.:  
WELD NO.:  
REFERENCES: M&P 6.1.1.2(c),  
M&P 4.3.8.5(b), SAA-33-27  
QSAA-11A(3), QMA-11A(1)F4

Non-Destructive Testing:

P.T.  
M.T.  
R.T.  
U.T.

WELDING CONDITIONS:

Electrode Type & Size

Filler Metal Type & Size

Flux Type & Size

\*Welding Current & Polarity

\*Arc Voltage

\*Travel Speed (in/min.)

Shield Gas Type & Flow

Gas Cup Size

Gas Cup to Work Distance

Other

\*  $\pm 10\%$  of Value or Range

WELDING POSITIONS: Flat

Preheat: 250 °F. Hold  
Interpass: 500 °F.

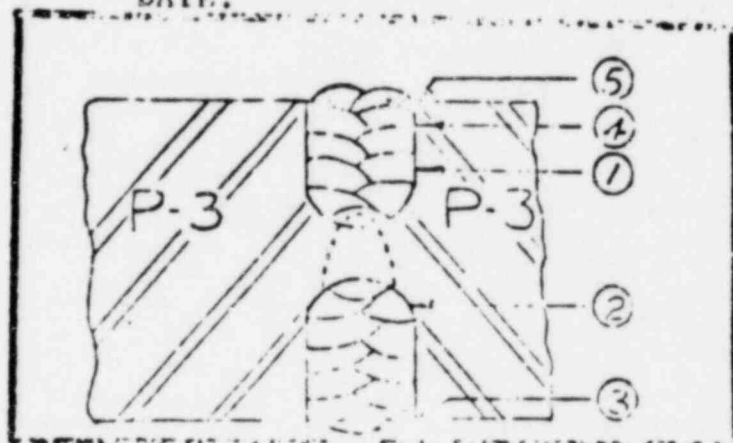
REWORKING:

Until P.W.H.T.

Post-weld heat treatment: 1150  $\pm$  25 °F hold one hour/inch  
thickness of weld.

Intermediate P.W.H.T. 1100  $\pm$  50 °F, hold 15 minutes.

DETAIL WELDING PROCEDURE  
NO.: TSAA-2(A) Rev. 1  
DATE:



See attached sheet.

Table 9 continued

## DETAIL WELDING PROCEDURE

No.: TSAA-2(A) Rev. 1

Sheet: 2 of 2

<u>WELDING SEQUENCE</u>	<u>TRAVEL</u>	<u>AMPS*</u>	<u>VOLTS*</u>
*1st Pass - O.D. 3/16"Ø Mil.B4 Mod.	13 IPM	550 AC Single Arc	31
1st Increment - O.D. 3/16"Ø Mil.B4 Mod.	13 IPM	650 AC Single Arc	31
(1) O.D. to 1½" Level 3/16"Ø Mil. B4 Mod.	13 IPM	650 AC Single Arc	31
(2) I.D. to 1" Level 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC Tandem Arc	31
(3) *Remainder - I.D. 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC Tandem Arc	31
(4) O.D. to 3" Level 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC Tandem Arc	31
(5) *Remainder O.D. 3/16"Ø Mil. B4 Mod.	22 IPM	600/550 AC Tandem Arc	31
Root or Backweld 1/4"Ø E-8018 C-3 (Flat Only)		325-375 DC-RP	25
or 3/16"Ø E-8018 C-3		210-260 DC-RP	25

\*Flux Linde 1092 65 x 200



WELD PROCEDURE  
SPECIFICATION

Low Alloy SMA & SA  
Grooves & buildup

PROCEDURE NUMBER 105-227-5486  
 PAGE NO. 1 OF 3  
 DATE 2-27-69  
 REVISION NO. 8 (7-2)-74 P. 5

PREHEAT REQUIREMENTS:

Minimum preheat of 300°F shall be applied uniformly to the full thickness of the weld joint and adjacent base material for a minimum distance of "T" or "6", whichever is least, where "T" is the material thickness.

Maintain 300°F min. preheat temp. until start of postweld heat treatment except for longitudinal and circumferential shell and head seams, preheat may be dropped to 250°F min. 8 hours after completion of welding. All runoff tabs and flux dams must be removed prior to dropping preheat below 300°F.

INTERPASS TEMPERATURE REQUIREMENTS:

The interpass temperature shall not exceed 500°F maximum.

FILLER METAL:

### Submerged Arc

Specification - N.A.  
Classification - N.A.  
Analysis - AI (except Ni 0.50 to  
1.25)  
Disability - F6  
Trade Name - CBI INHM (1% Nickel)  
or equal

### ELECTRICAL CHARACTERISTICS:

SMA - DC RP

Submerged Arc  
Tandem Wire  
Lead Wire - DC??  
Trail Wire - AC  
Single Wire - DC??

BACKUP GAS - MON. 3

FLUX - Linde 224

CUSTOMER APPROVAL

000000

[illegible]

Table 11  
DETAIL WELD PROCEDURE FOR ZIMMER 1 & LA SALLE 2 SURVEILLANCE PROGRAM



CONTRACT NO.	BY	DATE
62-5521	PCJ	11/9/76

## SPECIFICATION

Low Alloy SMA & SA  
Grooves & Buildup

CUSTOMER General Electric Company  
PRODUCT NUCLEAR VESSELS (Class A)  
DESCRIPTION Shielded Metal Arc and Submerged  
Arc Welding of ASME P12B Subgroup 1 Material

PROCEDURE NUMBER WPS 323-2F06  
PAGE NO 1 of 1  
DATE 2-17-69  
REVISION NO 4 (9-21-70) PJ

## REFERENCE SPECIFICATIONS

General WPS 800 Latest Revision  
General WPS 820 Latest Revision

## PROCEDURE QUALIFICATION

NO	POSITION	THICKNESS RANGE
963(TW)	F(Sub Arc) F,V,H(SMA)	4 1/2" to 9.9"
1261(CW)	F(Sub Arc) F,V(SMA)	2 3/4" to 8"

## POST HEAT TREATMENT -

Procedure qualified with 50 hrs. at 1150°F +25°/-50°F.

Post weld heat treatment of the weldment shall be in accordance with a CB&I approved procedure.

## BASE METAL -

ASME SA-533 Gr B Class 1 or  
SA-508 Class 2  
ASME Group No. P12B Subgroup 1

## FILLER METAL - ASME

See Adjacent Column

## ELECTRICAL CHARACTERISTICS -

See Adjacent Column

SHIELDING GAS - None

BACKUP GAS - None

FLUX - Linde 124

## PREHEAT REQUIREMENTS:

Minimum preheat of 300°F shall be applied uniformly to the full thickness of the weld joint and adjacent base material for a minimum distance of "T" or 6", whichever is least, where "T" is the material thickness.

Maintain preheat temperature until start of post weld heat treatment.

## INTERPASS TEMPERATURE REQUIREMENTS

The interpass temperature shall not exceed 500°F maximum.

## FILLER METAL:

## Submerged Arc

Specification - N.A.

Classification - N.A.

Analysis - A3 (except Ni 0.50 to 1.25)

Usability - F6

Trade Name - Adcom 1NM (1% Nickel or equal)

## Shielded Metal Arc

Specification - SA-316

Classification - E3018-G

Analysis - A3 (except Ni 0.50 to 1.25)

Usability - F4

Trade Name - Alloy Rods E3018NM

## ELECTRICAL CHARACTERISTICS:

SMA - DCRP

Submerged Arc  
Tandem Wire

Lead Wire - DCRP

Trail Wire - AC

Single Wire - DCRP

VPF # 2812-93-2

Table 12  
 MILB-4 ELECTRODE, LINDE 1092 FLUX  
 SUBMERGED ARC  
 VESSEL WELD TOUGHNESS DATA  
 (LaSalle 1 - Combustion Engineering)

Heat No./ Lot No.	NDT (°F)	Charpy Temp (°F)	Single or Tandem Wire (S or T)	Energy (ft-lbs)	Lateral Expansion (mils)	% Shear
21935/3889		+10		97, 90, 83	NA	NA
12008/3889		+10		97, 90, 83	NA	NA
30544/3947		+10		82, 66, 80 92, 91, 92	NA	NA
12008/3947		+10		92, 91, 92	NA	NA
305424/3889		+10		82, 87, 92	NA	NA
1P3571/3958*		+10	S	40, 46, 46	NA	NA
			T	79, 68, 64		
		+200	T	111, 110, 109	77, 78, 79.5	99, 99, 99
4P6519/0145	-60	+10		106, 109, 116	NA	NA
4P6519/0842	-80	+10		110, 79, 126	NA	80, 70, 90
4P6519/0653	-60	0		88, 94, 96	NA	60, 70, 70
		+60		121, 121, 120	NA	100, 100, 100
		+212		125, 133, 133	NA	100, 100, 100
10137/3999		+10		101, 108, 107	NA	NA
6324637/3499		+10		101, 108, 103	NA	NA
5P5622/0831	-80	+10		108, 112, 109	NA	NA
2P5755/0831	-70	+10		109, 104, 114	NA	NA
6329637/3458		+10		103, 65, 88	NA	NA
51874/3458		+10		89, 64, 87	NA	NA

NA = Not Available

\*This material (T) is in LaSalle 1 & Shoreham surveillance program.

Table 13

INMM ELECTRODE (TRADE NAME - RACO)  
 LINDE 124 FLUX, SUBMERGED ARC  
 POST WELD 1150°F for 50 HR TYPICAL

Plant C (Laguna Verde 2 - CBIN)

Heat No./ Flux No.	NDT (°F)	Charpy Temp (°F)	Single or Tandem Wire (S or T)	Energy (ft-lbs)	Lateral Expansion (mils)	% Shear
5P7397/0156	-50	-70		25, 21	18, 15	5, 5
		-50		42, 27, 19	33, 25, 20	10, 15, 10
		+10		64, 67, 55	53, 53, 52	30, 35, 40
		+10		64, 70	53, 54	40, 45
		+40		91, 84, 85	78, 68, 79	85, 90, 95
		+212		103, 92, 94	59, 66, 59	100, 100, 100
3P4966/0342	-80	-80		51, 27, 9	45, 25, 12	5, 5, 5
		-20		71, 66, 54	57, 57, 45	30, 25, 20
		+10		85, 84, 71	68, 72, 61	70, 80, 65
		+10		83, 76	67, 64	65, 55
		+40		87, 91	71, 60	75, 80
		+70		100, 101, 97	82, 89, 71	90, 95, 90
		+212		108, 111, 108	66, 84, 86	100, 100, 100
4P7465/0751	-60	-80		27, 14	21, 12	5, 0
		-70		48, 43, 26	42, 36, 22	15, 15, 5
		0		63, 57, 68	54, 45, 63	30, 25, 35
		+10		56, 58, 90	62, 62, 86	30, 25, 45
		+10		87, 55	83, 42	40, 30
		+40		67, 97	71, 90	45, 50
		+212		118, 102, 112	88, 71, 72	100, 100, 100
1P6484/0156	-20	-80		5, 8	6, 11	5, 5
		-60		22, 16, 12	23, 13, 10	10, 10, 10
		0		17, 36, 30	20, 27, 28	25, 20, 25
		+10		30, 38, 17,	25, 38, 12,	15, 15, 15,
				34, 38	28, 30	15, 20
		+30		34, 46, 42	29, 37, 45	25, 50, 35
		+40		72, 60, 72	54, 47, 49	50, 45, 50
5P5657/0931	-60	+212		93, 81, 83	65, 66, 69	100, 100, 100
		-80		39, 39	27, 37	5, 5
		-60		19, 20, 32	18, 22, 28	10, 10, 10
		0		51, 55, 58	50, 50, 63	30, 30, 55
		+10		69, 69, 66	61, 65, 59	50, 50, 40
		+10		62, 57	60, 63	60, 40
		+40		77, 66	73, 72	70, 80
		+212		88, 91, 85	86, 75, 83	100, 100, 100

Table 14

INMM ELECTRODE (TRADE NAME - TECHALLOY)  
 LINDE 124 FLUX, SUBMERGED ARC  
 POST WELD 1150°F FOR 50 HRS TYPICAL

Plant A (Zimmer RPV, CBIN)

Heat No./ Flux Lot	NDT (°F)	Charpy Temp (°F)	Single or Tandem Wire (S or T)	Energy (ft-lbs)			Expansion (mils)			% Shear		
KNE03/0171	-80	-130	S	7,	6		7,	7		5,	5	
		-80		34,	18,	22	32,	16,	21	40,	35,	40
		-20		68,	70,	62	61,	57,	56	80,	70,	75
		+10		75,	72		64,	64		90,	90	
		+40		94,	82		81,	71		100,	95	
		+212		94,	92,	86	76,	80,	80	100,	100,	100
	-100	-130	T	7,	5		6,	5		5,	5	
		-100		25,	16		24,	19		10,	10	
		-80		24,	22,	25	21,	19,	25	25,	20,	30
		-20		48,	49,	54	44,	42,	46	45,	45,	60
		-10		59,	54,	54	48,	49,	46	60,	45,	60
		+10		78,	67		65,	56		95,	80	
		+40		80,	79		68,	68		95,	95	
		+212		86,	89,	87	87,	86,	85	100,	100,	100

Table 15

ENMM ELECTRODE (TRADE NAME - RACO)  
 LINDE 124 FLUX, SUBMERGED ARC  
 POST WELD 1150°F FOR 50 HR TYPICAL

Plant B (La Salle 2 RPV, CBIN)

Heat No./ Flux Lot	NDT (°F)	Charpy Temp. (°F)	Single or Tandem Wire (S or T)	Energy (ft-lbs)	Lateral Expansion (mils)	% Shear
5P7397/ 0342	-70	-70	T	22, 16, 36	22, 18, 28	5, 5, 5
		-10		58, 68, 61	54, 50, 47	25, 20, 20
		+10		76, 73, 75	60, 65, 60	30, 45, 50
		+10		75, 69	58, 56	35, 35
		+40		91, 84	75, 63	80, 85
		+70		79, 75, 77	73, 63, 74	90, 95, 95
		+212		84, 81, 87	69, 67, 75	100, 100, 100
		-70	S	20, 34, 28	16, 32, 22	5, 5, 5
		-10		54, 50, 59	47, 47, 53	25, 20, 20
		+10		65, 59, 69	60, 56, 65	50, 25, 75
		+10		70, 75	56, 61	45, 55
		+40		71, 78	65, 68	75, 90
		+70		92, 101, 94	82, 65, 69	95, 95, 100
		+212		100, 95, 96	88, 58, 82	100, 100, 100

Table 16

## FERMI 2 REACTOR COOLANT PRESSURE BOUNDARY NSSS SUPPLY MAIN STEAM PIPE DATA

Mfr.	Material Supplier	Material	Component	O.D. Size	Min. Wall	Heat No.	Lot No.	Wt. %					TS ksi	YS ksi	Grain Size	Charpy Data* ft-lb
								C	Mn	P	S	Si				
NABCO	Lukens	A155 Class 1 Grade 3CF70 (from A516-69 Gr. 70) (Pipe stress relieved 1175°F, 2-1/2 hr.)	Pipe	26"	1.088"	B2875	---	.21	.99	.009	.021	.24	77.5	47	7-8	NA
Taylor Forge	Bethlehem	A420 WPL1-W (from SA-516 Gr. 70)	Elbows	26"	1.140"	ECNU 802B10449	---	.22	1.07	.016	.026	.26	76.4	48.4	Fine Grain	40-29-27
			Elbows	26"	1.140"	ECPY 802C05829	---	.22	1.01	.009	.025	.25	78.2	53.7		38-33-39
			Elbows	26"	0.950"	ECNT 801B08420	---	.22	1.11	.009	.020	.24	75.4	50.4		21-56-42
			Elbows	26"	1.140"	ECPV 802C09120	---	.23	1.04	.017	.030	.25	77.2	52.8		45-41-32
			Elbows	26"	0.950"	ECNW 802C05820	---	.22	1.01	.009	.025	.25	73.2	51.6		50-40-54
	Crucible	A350 Gr. LF1	Expander Flange	6"x8"	Sch.160	EMMY 3108903	---	.26	.83	.006	.025	.21	74.0	46.5	7	33-15-16
(Materials normalized 1650°F, welds stress relieved 1175°F)																
Bonney Forge	Sharon	A350 Gr. LF1	Sweepolets	26"x8"	1.088"	219839	Q1Q57/ 307M	.26	.80	.010	.010	.22	83.3	56.1	Fine Grain	26-25-32
						218543	Q1Q20/ 693.166943	.29	.81	.010	.021	.23	84.5	56.3		13-16-17
						218306	Q1Q10/ 695J	.29	.74	.010	.013	.23	85.0	56.9		19-20-19
						210608	C772	.24	.69	.009	.012	.23	75.3	49.4		23-23-23
	Bethlehem	SA-105 Gr. 2	Socket/Weld	26"x2"	0.950"	662C499	F873	.30	.75	.010	.023	.22	88.4	57.6		>15 (Spec.)
(Materials normalized 1650°F, weld stress relieved 1175°F)																

\*Charpy Keyhole at -50°F  
NA = Not Available





Table 18

## FERMI 2 MAIN STEAM NSSS PIPE WELD FILLER METAL

1. AWS A5.1, E7018 - Meets charpy v-notch minimum requirement of 20ft-lb. at -20 °F.
2. AWS A5.17, EH14 - RACO 123/402B0451 heat no. used for elbows reports charpy v-notch values at -50 °F of 47-46-45 ft-lb.
3. AWS A5.18, E70S2- Charpy v-notch minimum requirement of 20 ft-lb. at -20 °F.

TVA #17 MAIN STEAM PIPE TOUGHNESS  
(SA-516 Gr. 70, LUKENS/NABCO)

# NATIONAL ANNEALING BOX CO.

WASHINGTON, PENNA.

## MATERIAL RECORD

INSPECTION

**APPROVED**  
AP & E  
QC DEPT

DATE 8/22/77  
SIGNED [Signature]  
ny/25/77

PURCHASER ASSOCIATED POWER & Fuel Corp  
TYPE EQUIPMENT SA 516 Gr 70 Anneal Section C-1  
SA 516 Gr 70 Anneal Section C-2  
YEAR BUILT 1977 (32-1962-7)  
DWG. NO. NA-2311-1 REV I  
SERIAL NO. 1467-1 C1K  
PUR. ORDER NO. 12114-4  
INSPECTED BY Ed. H. H. S.

## LABORATORY TEST REPORT

INSPECTED BY				CHEMICAL				PHYSICAL				
PART NO	MANUFACTURER OF MATERIAL	MILY. SIAL. SERIAL NO.	SIAL. GP. TEST NO.	C.	MIN	P	S	SIL.	ULT. STRENGTH	EL. LIMIT	ELONGATION	FRAC. TURE
1-1	Ph. C. 200	C. 6220	3A	23.107	005	014	26		78100	44400	27	OK
1-2	Ph. C. 200	C. 6220	1B	23.107	005	014	26		81500	(TEST)		OK
2-1	Ph. C. 200	C. 6220	2A	23.107	007	016	22		77200	65600	26	OK
2-2	Ph. C. 200	C. 6220	2B	23.107	007	016	22		81500	(TEST)		OK
3-1	Ph. C. 200	C. 6220	3A	23.107	007	016	22					
3-2	Ph. C. 200	C. 6220	3B	23.107	007	016	22					
4-1	Ph. C. 200	C. 6220	4A	23.107	007	016	22					
4-2	Ph. C. 200	C. 6220	4B	23.107	007	016	22					
5-1	Ph. C. 200	C. 6220	5A	23.107	007	016	22					
5-2	Ph. C. 200	C. 6220	5B	23.107	007	016	22					
6-1	Ph. C. 200	C. 6220	6A	23.107	007	016	22					
6-2	Ph. C. 200	C. 6220	6B	23.107	007	016	22					
7-1	Ph. C. 200	C. 6220	7A	23.107	007	016	22					
7-2	Ph. C. 200	C. 6220	7B	23.107	007	016	22					
8-1	Ph. C. 200	C. 6220	8A	23.107	007	016	22					
8-2	Ph. C. 200	C. 6220	8B	23.107	007	016	22					
9-1	Ph. C. 200	C. 6220	9A	23.107	007	016	22					
9-2	Ph. C. 200	C. 6220	9B	23.107	007	016	22					
10-1	Ph. C. 200	C. 6220	10A	23.107	007	016	22					
10-2	Ph. C. 200	C. 6220	10B	23.107	007	016	22					
11-1	Ph. C. 200	C. 6220	11A	23.107	007	016	22					
11-2	Ph. C. 200	C. 6220	11B	23.107	007	016	22					
12-1	Ph. C. 200	C. 6220	12A	23.107	007	016	22					
12-2	Ph. C. 200	C. 6220	12B	23.107	007	016	22					
13-1	Ph. C. 200	C. 6220	13A	23.107	007	016	22					
13-2	Ph. C. 200	C. 6220	13B	23.107	007	016	22					
14-1	Ph. C. 200	C. 6220	14A	23.107	007	016	22					
14-2	Ph. C. 200	C. 6220	14B	23.107	007	016	22					
15-1	Ph. C. 200	C. 6220	15A	23.107	007	016	22					
15-2	Ph. C. 200	C. 6220	15B	23.107	007	016	22					
16-1	Ph. C. 200	C. 6220	16A	23.107	007	016	22					
16-2	Ph. C. 200	C. 6220	16B	23.107	007	016	22					
17-1	Ph. C. 200	C. 6220	17A	23.107	007	016	22					
17-2	Ph. C. 200	C. 6220	17B	23.107	007	016	22					
18-1	Ph. C. 200	C. 6220	18A	23.107	007	016	22					
18-2	Ph. C. 200	C. 6220	18B	23.107	007	016	22					
19-1	Ph. C. 200	C. 6220	19A	23.107	007	016	22					
19-2	Ph. C. 200	C. 6220	19B	23.107	007	016	22					
20-1	Ph. C. 200	C. 6220	20A	23.107	007	016	22					
20-2	Ph. C. 200	C. 6220	20B	23.107	007	016	22					
21-1	Ph. C. 200	C. 6220	21A	23.107	007	016	22					
21-2	Ph. C. 200	C. 6220	21B	23.107	007	016	22					
22-1	Ph. C. 200	C. 6220	22A	23.107	007	016	22					
22-2	Ph. C. 200	C. 6220	22B	23.107	007	016	22					
23-1	Ph. C. 200	C. 6220	23A	23.107	007	016	22					
23-2	Ph. C. 200	C. 6220	23B	23.107	007	016	22					
24-1	Ph. C. 200	C. 6220	24A	23.107	007	016	22					
24-2	Ph. C. 200	C. 6220	24B	23.107	007	016	22					
25-1	Ph. C. 200	C. 6220	25A	23.107	007	016	22					
25-2	Ph. C. 200	C. 6220	25B	23.107	007	016	22					
26-1	Ph. C. 200	C. 6220	26A	23.107	007	016	22					
26-2	Ph. C. 200	C. 6220	26B	23.107	007	016	22					
27-1	Ph. C. 200	C. 6220	27A	23.107	007	016	22					
27-2	Ph. C. 200	C. 6220	27B	23.107	007	016	22					
28-1	Ph. C. 200	C. 6220	28A	23.107	007	016	22					
28-2	Ph. C. 200	C. 6220	28B	23.107	007	016	22					
29-1	Ph. C. 200	C. 6220	29A	23.107	007	016	22					
29-2	Ph. C. 200	C. 6220	29B	23.107	007	016	22					
30-1	Ph. C. 200	C. 6220	30A	23.107	007	016	22					
30-2	Ph. C. 200	C. 6220	30B	23.107	007	016	22					
31-1	Ph. C. 200	C. 6220	31A	23.107	007	016	22					
31-2	Ph. C. 200	C. 6220	31B	23.107	007	016	22					
32-1	Ph. C. 200	C. 6220	32A	23.107	007	016	22					
32-2	Ph. C. 200	C. 6220	32B	23.107	007	016	22					
33-1	Ph. C. 200	C. 6220	33A	23.107	007	016	22					
33-2	Ph. C. 200	C. 6220	33B	23.107	007	016	22					
34-1	Ph. C. 200	C. 6220	34A	23.107	007	016	22					
34-2	Ph. C. 200	C. 6220	34B	23.107	007	016	22					
35-1	Ph. C. 200	C. 6220	35A	23.107	007	016	22					
35-2	Ph. C. 200	C. 6220	35B	23.107	007	016	22					
36-1	Ph. C. 200	C. 6220	36A	23.107	007	016	22					
36-2	Ph. C. 200	C. 6220	36B	23.107	007	016	22					
37-1	Ph. C. 200	C. 6220	37A	23.107	007	016	22					
37-2	Ph. C. 200	C. 6220	37B	23.107	007	016	22					
38-1	Ph. C. 200	C. 6220	38A	23.107	007	016	22					
38-2	Ph. C. 200	C. 6220	38B	23.107	007	016	22					
39-1	Ph. C. 200	C. 6220	39A	23.107	007	016	22					
39-2	Ph. C. 200	C. 6220	39B	23.107	007	016	22					
40-1	Ph. C. 200	C. 6220	40A	23.107	007	016	22					
40-2	Ph. C. 200	C. 6220	40B	23.107	007	016	22					
41-1	Ph. C. 200	C. 6220	41A	23.107	007	016	22					
41-2	Ph. C. 200	C. 6220	41B	23.107	007	016	22					
42-1	Ph. C. 200	C. 6220	42A	23.107	007	016	22					
42-2	Ph. C. 200	C. 6220	42B	23.107	007	016	22					
43-1	Ph. C. 200	C. 6220	43A	23.107	007	016	22					
43-2	Ph. C. 200	C. 6220	43B	23.107	007	016	22					
44-1	Ph. C. 200	C. 6220	44A	23.107	007	016	22					
44-2	Ph. C. 200	C. 6220	44B	23.107	007	016	22					
45-1	Ph. C. 200	C. 6220	45A	23.107	007	016	22					
45-2	Ph. C. 200	C. 6220	45B	23.107	007	016	22					
46-1	Ph. C. 200	C. 6220	46A	23.107	007	016	22					
46-2	Ph. C. 200	C. 6220	46B	23.107	007	016	22					
47-1	Ph. C. 200	C. 6220	47A	23.107	007	016	22					
47-2	Ph. C. 200	C. 6220	47B	23.107	007	016	22					
48-1	Ph. C. 200	C. 6220	48A	23.107	007	016	22					
48-2	Ph. C. 200	C. 6220	48B	23.107	007	016	22					
49-1	Ph. C. 200	C. 6220	49A	23.107	007	016	22					
49-2	Ph. C. 200	C. 6220	49B	23.107	007	016	22					
50-1	Ph. C. 200	C. 6220	50A	23.107	007	016	22					
50-2	Ph. C. 200	C. 6220	50B	23.107	007	016	22					
51-1	Ph. C. 200	C. 6220	51A	23.107	007	016	22					
51-2	Ph. C. 200	C. 6220	51B	23.107	007	016	22					
52-1	Ph. C. 200	C. 6220	52A	23.107	007	016	22					
52-2	Ph. C. 200	C. 6220	52B	23.107	007	016	22					
53-1	Ph. C. 200	C. 6220	53A	23.107	007	016	22					
53-2	Ph. C. 200	C. 6220	53B	23.107	007	016	22					
54-1	Ph. C. 200	C. 6220	54A	23.107	007	016	22					
54-2	Ph. C. 200	C. 6220	54B	23.107	007	016	22					
55-1	Ph. C. 200	C. 6220	55A	23.107	007	016	22					
55-2	Ph. C. 200	C. 6220	55B	23.107	007	016	22					
56-1	Ph. C. 200	C. 6220	56A	23.107	007	016	22					
56-2	Ph. C. 200	C. 6220	56B	23.107	007	016	22					
57-1	Ph. C. 200	C. 6220	57A	23.107	007	016	22					
57-2	Ph. C. 200	C. 6220	57B	23.107	007	016						

SUSQUEHANNA I A106 GR. B  
MAIN STEAM NSSS SUPPLY TIGHTNESS DATA

Table 20

Pipe Size	Sch.	Thickness	Mfg.	Heat	C	Mn	P	S	SI	TS K	YS K	Ft.-Lbs.	Mile Lbs. Yr.	*F
23.5 ID	1.013	0.26	Camerton	4410	0.26	1.01	0.011	0.015	0.26	78.9	51.6	116	122	105
23.5 ID	1.013	0.26	Camerton	4410	0.26	1.01	0.011	0.015	0.26	78.9	51.6	115	120	81
23.5 ID	1.013	0.26	Camerton	4411	0.26	1.01	0.020	0.026	0.23	79.6	56.9	96	98	96
23.5 ID	1.013	0.26	Camerton	4411	0.26	1.01	0.020	0.026	0.23	79.6	56.9	93	97	92
8"	0.906	0.26	Phoenix	21342	0.26	0.90	0.012	0.027	0.23	80	40	32	45	40
26" Reducer	1.013	0.26	Tube Turm	S9351	0.26	0.97	0.020	0.035	0.23	80.9	56.3	165	166	210
26" Weld E11	1.013	0.25	Tube Turm	S9087	0.25	0.85	0.007	0.026	0.18	76.6	42.1	158	139	162
26" Weld E11	1.013	0.21	Tube Turm	S2155	0.21	0.87	0.008	0.020	0.17	67.9	40.8	11	42.5	52.0
26" Weld E11	1.013	0.27	Tube Turm	S9923	0.27	0.92	0.020	0.028	0.21	82	55.6	54	68	67
26" Weld E11	1.013	0.30	Tube Turm	S9825	0.30	0.92	0.020	0.020	0.20	80.9	45.5	59	50	65
26" Weld E11	1.013	0.26	Tube Turm	S9926	0.26	0.99	0.026	0.018	0.25	81.9	50.6	60	57	59
26" Weld E11	1.013	0.26	Tube Turm	S8369	0.26	0.95	0.009	0.025	0.16	72.5	38.4	69	66	73
26" Tee	1.013	0.27	Tube Turm	S8863	0.27	0.93	0.015	0.026	0.23	79	41.5	66	52	66

Charge Data

Table 21  
 ZIMMER 1 A106 GR. B  
 MAIN STEAM NSSS SUPPLY TOUGHNESS DATA

Pipe Size	Sch.	Thk. In.	Mfg.	Heat	C	Mo	P	S	SI	T <sub>1</sub>	T <sub>2</sub>	Charpy Data							
												FT-LBS.	Lat. Imp.	91%	°F				
24"		0.945	USS	151171	0.25	0.91	0.013	0.019	0.18	75	75	61	76	90	118	66	76	80	70
24"		0.945	USS	851188	0.25	0.93	0.016	0.021	0.16	73	73	60	40	62	60	50	58	55	70
24"		0.945	USS	151116	0.25	0.95	0.018	0.021	0.20	73	73	62	46	68	50	55	62	42	70
24"		0.945	USS	851118	0.25	0.95	0.016	0.021	0.16	75	75	62	33	32	32	36	36	36	70
24"		0.945	USS	851173	0.25	0.92	0.025	0.025	0.22	73	73	60	65	75	70	65	66	66	70
FITTERS A136 (A106B)																			
24 1R F11 (Elbow)		0.945	Tube Turns	151822	0.25	0.93	0.010	0.022	0.17	70	70	60	43	50	56	63	55	58	70
24 1R F11		0.945	Tube Turns	122573	0.25	0.93	0.005	0.021	0.16	76	76	61	61	56	56	67	55	49	70
24 1R F11		0.945	Tube Turns	151824	0.25	0.98	0.011	0.025	0.19	73	73	66	75	75	83	69	66	61	70
24 1R F11		0.945	Tube Turns	160566	0.25	0.95	0.013	0.016	0.23	76	76	58	96	103	105	76	79	75	70

Table 22

Project	<u>Fermi 2</u>					
Valve	<u>MSIV</u>					
Component	<u>Cover</u>					
Applicable Code	<u>1968 Pump &amp; Valve Code (ASME)</u>					
Valve Vendor	<u>Atwood &amp; Morrill Co.</u>					
Material Vendor	<u>Cann &amp; Saul Steel Co.</u>					
Material Specification	<u>ASTM A105 Grade 2</u>					
Heat No.	<u>219222 (Typical)</u>					
Chemical Composition (Wt. %)	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
	.30	.68	.19	.009	.014	NA (Typical)
Grain Size (ASTM No.)	<u>NA</u>					
Heat Treatment	<u>1650 °F (12 hr.) Air Cool</u>					
Charpy V - Notch Impact Toughness						
Test Temperature:	} NA					
Ft-lb.						
Mils						
% Shear						

NA - Not Available

Table 22

Project	<u>Fermi 2</u>					
Valve	<u>MSIV</u>					
Component	<u>Body</u>					
Applicable Code	<u>1968 Pump &amp; Valve Code (ASME)</u>					
Valve Vendor	<u>Atwood &amp; Morrill Co.,</u>					
Material Vendor	<u>Quaker Alloy Casting Co.</u>					
Material Specification	ASTM A216 WCB					
Heat No.	F7080 (Typical)					
Chemical Composition (Wt. %)	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
	0.27	0.79	0.39	0.019	0.012	NA (Typical)
Grain Size (ASTM No.)	NA					
Heat Treatment	1700 °F (4 hr. 30 min.) Air Cool +1320/1340°F (4 hr. 10 min.) Air Cool +1245/1260°F (4 hr. 20 min.) Air Cool +1100/1160°F (3 hr.) Air Cool +1150°F (4 hr. 15 min.) Air Cool +1150°F (3 hr. 10 min.) Air Cool					
Charpy V - Notch Impact Toughness						
Test Temperature:						
Ft-lb.	} NA					
Mils						
% Shear						
Weld Filler Metal	AWS A5.1-69 Type E7018 Tested by charpy v-notch at -20 °F to meet the requirement of at least 20 ft-lb.					

NA - Not Available



# RECOMMENDED FORM Q-1 MANUFACTURER'S RECORD OF WELDING PROCEDURE QUALIFICATION TESTS

Specification No. QAP-L9, Rev. D, Mod. Date 10-26-71  
 Welding Process Shielded Arc Manual or Machine Manual  
 Material Specification A216 WCB to A216 WCB of P-No. 1 to P-No. 1  
 Thickness (if pipe, diameter and wall thickness) 1/2 Inches  
 Thickness Range this test qualifies 3/16 Inch to 6 Inches  
 Filler Metal Group No. F- 1 FLUX OR ATMOSPHERE  
 Weld Metal Analysis No. A- 1 Flux Trade Name or Composition \_\_\_\_\_  
 Describe Filler Metal if not included in Table Q-11.2 Inert Gas Composition \_\_\_\_\_  
 or QN-11.2 Trade Name \_\_\_\_\_ Flow Rate \_\_\_\_\_  
 For oxyacetylene welding—State if Filler Metal is silicon or aluminum killed. Is Backing Strip used? Yes  
 Preheat Temperature Range 500°F Minimum  
 Interpass Temperature Range 500°F Minimum  
 Postheat Treatment 1100°F Minimum  
Air Cool  
**WELDING PROCEDURE**  
 Single or Multiple Pass Multiple  
 Single or Multiple Arc Single  
 Position of Groove Vertical - Upward (See Para. & Figs. Q-2 & Q-3, or QN-2 & QN-3)  
 (Flat, horizontal, vertical, or overhead; if vertical, state whether upward or downward)

**FOR INFORMATION ONLY**  
 Filler Wire—Diameter 1/8" - 5/32" - 3/16" - 1/4" **WELDING TECHNIQUES**  
 Trade Name Atom Arc 7015 Joint Dimensions Accord with 1971 ASME Code  
 Type of Backing Carbon Steel amps \* volts \* inches per min. \*  
 Forehand or Backhand Forehand Current Direct Polarity Reverse

## REDUCED SECTION TENSILE TEST (Figs. Q-6 and QN-6)

Specimen No.	Dimensions		Area	Ultimate Total Load, lb.	Ultimate Unit Stress, psi	Character of Failure and Location
	Width	Thickness				
SEE PAGE 2 FOR TENSILE RESULTS						
SEE PAGE 2 FOR TENSILE RESULTS						

## GUIDED BEND TESTS (Figs. Q-7.1, Q-7.2, QN-7.1, QN-7.2, QN-7.3)

Type and Figure No.	Result	Type and Figure No.	Result
Q-7.1	Satisfactory	Q-7.1	Satisfactory
Q-7.1	Satisfactory	Q-7.1	Satisfactory

Results of Filler Weld Tests, Fig. Q-8(a) \_\_\_\_\_  
 Welder's Name Earl Zollers Clock No. 275 Stamp No. EZ  
 Who by virtue of these tests meets welder performance requirements.  
 Test Conducted by Quaker Alloy Casting Co. Laboratory—Test No. EZ 62071  
 per Mark Linds

We certify that the statements in this record are correct and that the test welds were prepared, welded and tested in accordance with the requirements of Section IX of the ASME Code.

Signed QUAKER ALLOY CASTING CO.  
 (Manufacturer)

Date 10-26-71

By John J. [Signature]

(Detail of record of tests are illustrative only and may be modified to conform to the type and number of tests required by the Code. Recommended Form Q-1 is available for purchase at ASME Headquarters.)

NOTE: Any essential variables in addition to those above shall be recorded.

Re-typed March 20, 1973

\*See Paragraph 11.5 of QAP-L9

55

Table 23  
 FERMI 2 MSIV BODY WELD PROCEDURE  
 QUALIFICATION TOUGHNESS RESULTS

QAP-49, D, Mod.  
Manual Shielded Arc

Table 23

Specimen No.	Diameter	Area	Ultimate Total Load lb.	Ultimate Unit Stress, psi	Location of Failure
1	.505	.2	15100	75500	Weld Metal
2	.505	.2	15000	75000	Weld Metal
3	.505	.2	14900	74500	Weld Metal
4	.505	.2	15200	76000	Weld Metal
5	.505	.2	14800	74000	Weld Metal
6	.505	.2	15100	75500	Weld Metal
7	.505	.2	14900	74500	Weld Metal
8	.505	.2	14800	74000	Weld Metal

Charpy Impact  
"V" Notch @ Minus 20°F

## Base Metal -

Foot pounds	34-31-34
Lateral Expansion	24-21-22
Percent Ductile-Fracture	20-20-20

## Weld Metal -

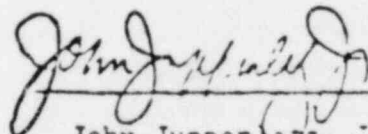
Foot pounds	60-72-80
Lateral Expansion	40-52-66
Percent Ductile Fracture	40-40-50

## Heat Affected Zone

Foot pounds	51-45-57
Lateral Expansion	23-21-28
Percent Ductile Fracture	40-40-40

## Non-Destructive Examination of Completed Weld

- |    |                               |   |            |
|----|-------------------------------|---|------------|
| 1. | Radiographic Examination      | - | Acceptable |
| 2. | Magnetic Particle Examination | - | Acceptable |
| 3. | Visual Examination            | - | Acceptable |


John Juppenhart, Jr.  
Quaker Alloy Casting Co.

3-20-73

Table 24

Project Clinton 1  
 Valve MSIV  
 Component Body  
 Applicable Code ASME Sect. III, 1974  
 Valve Vendor Atwood & Morrill Co.  
 Material Vendor Quaker Alley Casting Co.

Material Specification ASME SA216 Grade WCB

Heat No. F7516

	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.25	0.78	0.53	0.018	0.013	NA

Grain Size (ASTM No.) NA

Heat Treatment 1690/1710°F (6 hr. 5 min) Air Cool  
 + Temper 1350/1360°F (6 hrs) Air Cool  
 + Post Weld 1200°F (6 hr, 5 min) Air Cool

Charpy V - Notch Impact Toughness

Test Temperature: +60°F

Ft-lb. 30,24,34

Mils 37,27,33

% Shear 40,40,40

NA - Not Available

TABLE 24

Project	<u>Clinton 1</u>					
Valve	<u>MSIV</u>					
Component	<u>Cover (Bonnet)</u>					
Applicable Code	<u>ASME Sect. III, 1974</u>					
Valve Vendor	<u>Atwood &amp; Morrill Co.,</u>					
Material Vendor	<u>Cann &amp; Saul Steel Co.</u>					
Material Specification	ASME SA105 QT					
Heat No.	214934					
	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.28	0.76	0.22	0.017	0.023	NA
Grain Size (ASTM No.)	N/A					
Heat Treatment	1600°F (12 hr) Quench, Water + 1175°F (12 hr) Furnance Cool					
Charpy V - Notch Impact Toughness						
Test Temperature:	+ 60°F					
Ft-lb.	62,60,55					
Mils	48,45,50					
% Shear	30,30,30					

NA - Not Available

Table 24

Project	<u>Grand Gulf 1</u>					
Valve	<u>MSIV</u>					
Component	<u>Body</u>					
Applicable Code	<u>ASME Sect. III, 1974</u>					
Valve Vendor	<u>Atwood &amp; Morrill, Co.,</u>					
Material Vendor	<u>Quaker Alloy Casting Co.</u>					
Material Specification	ASME SA216 Grade WCB					
Heat No.	F6406					
Chemical Composition (Wt. %)	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
	0.23	0.89	0.53	0.019	0.012	NA
Grain Size (ASTM No.)	NA					
Heat Treatment	1680/1710°F (5 hrs, 30 min) Air Cool					
	+ Temper 1350°F (5 hr, 30 min) Air Cool					
	+ Post Weld 1200°F (6 hr) Air Cool					
Charpy V - Notch Impact Toughness						
Test Temperature:	+60°F					
Ft-lb.	32, 31, 34					
Mils	33, 32, 31					
% Shear	40, 40, 40					

NA - Not Available

Table 24

Project	<u>Grand Gulf 1</u>					
Valve	<u>MSIV</u>					
Component	<u>Cover (Bonnet)</u>					
Applicable Code	<u>ASME Sect. III, 1974</u>					
Valve Vendor	<u>Atwood &amp; Morrill Co.,</u>					
Material Vendor	<u>Cann &amp; Saul Steel Co.</u>					
Material Specification	SA-105 (QT)					
Heat No.	632202					
	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.26	0.94	0.20	0.023	0.015	NA
Grain Size (ASTM No.)	NA					
Heat Treatment	1550°F (12 hr) quench in water + 1175°F (12 hr) furnace cool					
Charpy V - Notch Impact Toughness:						
Test Temperature:	+60°F					
Ft-lb.	66,74,65					
Mils	58,64,54					
% Shear	20,20,20					

NA - Not Available

Table 24

Project	<u>Riverbend 1</u>
Valve	<u>MSIV</u>
Component	<u>Cover (Bonnet)</u>
Applicable Code	<u>ASME Sect. III, 1974</u>
Valve Vendor	<u>Atwood &amp; Morrill Co.,</u>
Material Vendor	<u>Cann &amp; Saul Steel Co.</u>

Material Specification	ASME SA105 QT
------------------------	---------------

Heat No.	216149
----------	--------

	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.30	0.88	0.16	0.006	0.014	NA

Grain Size (ASTM No.)	NA
-----------------------	----

Heat Treatment	1550°F (12 hr) Quench in water + 1225°F (12 hr) Furnance cool
----------------	--

Charpy V- Notch Impact Toughness

Test Temperature:	+60°F
-------------------	-------

Ft-lb.	62,64,60
--------	----------

Mils	56,54,52
------	----------

% Shear	20,20,20
---------	----------

NA - Not Available



Table 24

Project Riverbend 1  
 Valve MSIV  
 Component Body  
 Applicable Code ASME Sect. III, 1974  
 Valve Vendor Atwood & Morrill Co.,  
 Material Vendor Atwood & Morrill, Ltd.

Material Specification SA216 Grade WCB

Heat No. 35

	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.24	0.82	0.46	0.022	0.013	NA

Grain Size (ASTM No.) NA

Heat Treatment 1650°F - 1800°F (8 hrs.) air cool to 400°F  
 + temper 1150°/1250°F (8 hrs) air cool  
 + post weld 1095°/1195°F (18 hrs) furnace cool  
 to 800°F (100°F/hr) air cool

Charpy V - Notch Impact Toughness

Test Temperature: +60°F

Ft-lb. 31.5, 37.5, 39.5

Mils 33, 41, 40

% Shear 10, 10, 10

NA - Not Available

Table 24

Project	<u>Laguna Verde 1</u>					
Valve	<u>MSIV</u>					
Component	<u>Body</u>					
Applicable Code	<u>ASME Sect. III, 1971 with Summer 1973 Addenda</u>					
Valve Vendor	<u>Rockwell International</u>					
Material Vendor						
Material Specification	SA216 Grade WCC					
Heat No.	1750262					
	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.21	1.19	0.43	0.011	0.009	0.043
Grain Size (ASTM No.)	NA					
Heat Treatment	1700°F (10 hrs) normalize					
	+ 1225°F (7.5 hrs) Temp					
	+ 1100°F (6 hr) post weld					
Charpy V - Notch Impact Toughness						
Test Temperature:	+40°F					
Ft-lb.	29.0, 33.0, 35.0					
Mils	25.0, 26.0, 30.0					
% Shear	15, 15, 15					

NA - Not Available

Table 24

Project Laguna Verde 1  
 Valve MSIV  
 Component Bonnet  
 Applicable Code ASME Sect. III, 1971 with Summer 1973 Addenda  
 Valve Vendor Rockwell International  
 Material Vendor Cann & Saul Steel Co.

Material Specification SA105 Grade NUC

Heat No. 211971

	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.27	1.03	0.22	0.010	0.014	NA

Grain Size (ASTM No.) NA

Heat Treatment 1550°F (10 hr) Quench in water  
+ 1175°F (10 hr) Furnace cool

Charpy V - Notch Impact Toughness

Test Temperature: +40°F

	Ft-lb.	35,45,34	31,34,35	45,38,48
	Mils	NA	NA	64,57,65
	% Shear	30,40,30	30,30,30	15,15,15
Ft-lb	55,47,43	62,64,52	58,62,72	39,44,38
Mils	66,64,60	74,72,65	70,68,75	56,60,57
% Shear	20,15,15	20,20,20	20,20,20	15,15,15

NA - Not Available

Table 24

Project	<u>TVA X20</u>					
Valve	<u>MSIV</u>					
Component	<u>Body</u>					
Applicable Code	<u>ASME Sect. III, 1974 with Summer 1975 Addenda</u>					
Valve Vendor	<u>Atwood &amp; Morrill Co.,</u>					
Material Vendor	<u>Quaker Alloy Casting Co.</u>					
Material Specification	ASME SA216 Grade WCB					
Heat No.	F3547					
Chemical Composition (Wt. %)	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
	0.23	0.88	0.38	0.016	0.015	NA
Grain Size (ASTM No.)	NA					
Heat Treatment	1700°/1725°F (6 hr, 20 min) air cool + temper 1345°F (6 hr, 45 min) air cool + post weld 1200°/1225°F (6 hrs, 30 min) air cool					
Charpy V - Notch Impact Toughness						
Test Temperature:	+60°F					
Ft-lb.	66,56,54					
Mils	53,50,53					
% Shear	40,40,40					

NA - Not Available

Table 24

Project	<u>TVA X 20</u>
Valve	<u>MSIV</u>
Component	<u>Cover (Bonnet)</u>
Applicable Code	<u>ASME Sect. III, 1974 with Summer 1975 Addenda</u>
Valve Vendor	<u>Atwood &amp; Morrill Co.,</u>
Material Vendor	<u>Cann &amp; Saul Steel Co.</u>

Material Specification	ASME SA105
------------------------	------------

Heat No.	217630
----------	--------

	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.23	0.92	0.19	0.013	0.013	NA

Grain Size (ASTM No.)	#9
-----------------------	----

Heat Treatment	1650°F (6 hrs) air cool
	+ 1550°F (6 hr, 30 min) water quench
	+ Temper 1200°F (12 hr, 30 min)

Charpy V - Notch Impact Toughness

Test Temperature:	+60°F
Ft-lb.	90,89,77
Mils	71,67,59
% Shear	50,50,40

NA - Not Available

Table 24

Project	<u>CNV</u>
Valve	<u>MSIV</u>
Component	<u>Body</u>
Applicable Code	<u>ASME Sect. III, 1971 with S73 Addenda</u>
Valve Vendor	<u>Rockwell International,</u>
Material Vendor	<u>Rockwell International</u>

Material Specification	SA216 Grade WCC												
Heat No.	3760171												
Chemical Composition (Wt. %)	<table><tr><td><u>C</u></td><td><u>Mn</u></td><td><u>Si</u></td><td><u>P</u></td><td><u>S</u></td><td><u>Al</u></td></tr><tr><td>0.17</td><td>1.09</td><td>0.50</td><td>0.008</td><td>0.011</td><td>0.060</td></tr></table>	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>	0.17	1.09	0.50	0.008	0.011	0.060
<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>								
0.17	1.09	0.50	0.008	0.011	0.060								
Grain Size (ASTM No.)	NA												
Heat Treatment	1700°F (8 hours) Normalize 1275°F (8 hours) Temper 1100°F (6 hours) Post Weld												
Charpy V - Notch Impact Toughness													
Test Temperature:	+40°F												
Ft-lb.	35.0, 38.0, 29.0												
Mils	32.0, 36.0, 29.0												
% Shear	20, 20, 20												

NA - Not Available

Table 24

Project CNV  
 Valve MSIV  
 Component Bonnet  
 Applicable Code ASME Sect. III, 1971 with S73 Addenda  
 Valve Vendor Rockwell International,  
 Material Vendor Cann & Saul Steel Co.

Material Specification SA105  
 Heat No. 214943  

	<u>C</u>	<u>Mn</u>	<u>Si</u>	<u>P</u>	<u>S</u>	<u>Al</u>
Chemical Composition (Wt. %)	0.35	0.78	0.25	0.014	0.023	NA

 Grain Size (ASTM No.) NA  
 Heat Treatment 1550°F (10 hours) water quench  
 1175°F (10 hours) furnace cool  
 1100°F (10 hours) post weld

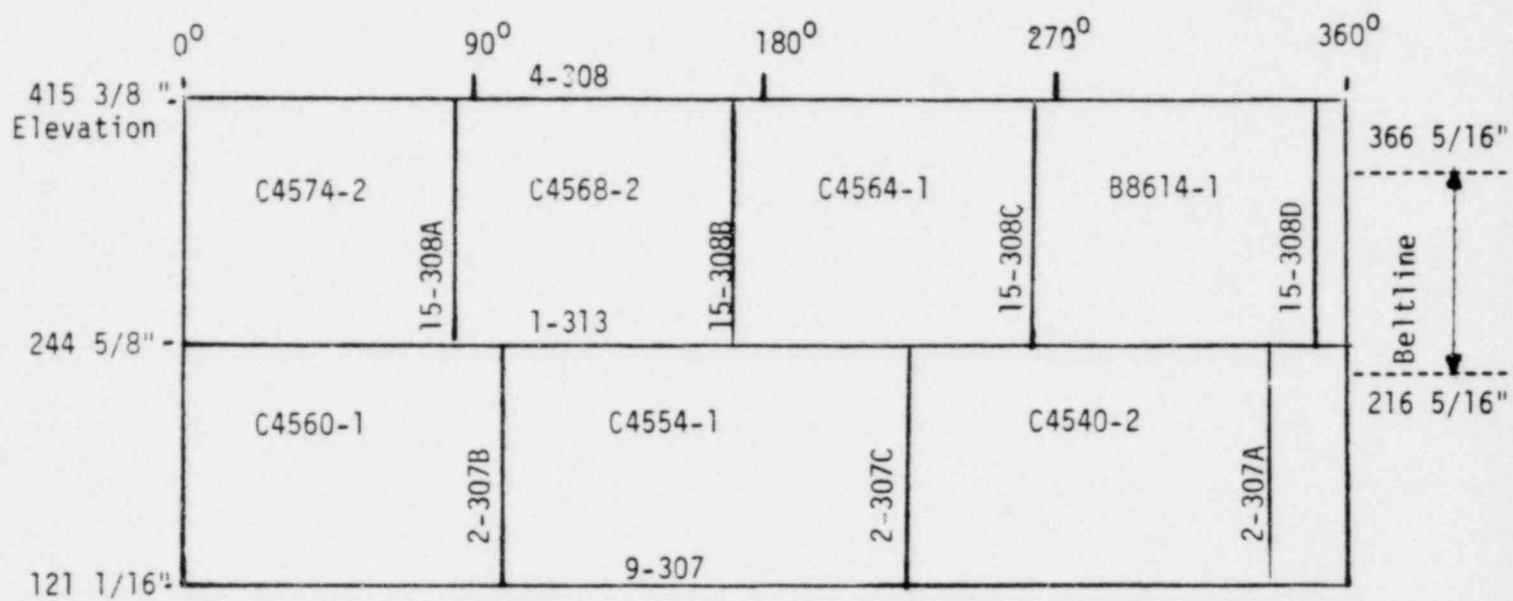
Charpy V - Notch Impact Toughness

Test Temperature:	+40°F	
Ft-lb.	25,25,29	28,30,34
Mils	36,34,34	36,37,35
% Shear	20,20,20	20,20,20

NA - Not Available



Figure 1

FERMI 2 BELTLINE PLATE AND WELD SEAM LOCATIONS

On the basis of the last paragraph on page 19013 of the July 17, 1973 Federal Register, the following subsection discusses what is considered to be an appropriate method of compliance.

#### 5.2.4.2.1 Method of Compliance

The intent of the proposed special method of compliance with Appendix G for this vessel is to provide operating limitations on pressure and temperature based on fracture toughness. These operating limits assure that a margin of safety against a non-ductile failure of this vessel is very nearly the same as that for a vessel built to the Summer 1972 Addenda.

The specific temperature limits for operation when the core is critical are based on a proposed modification to 10 CFR Part 50, Appendix G, Paragraph IV.A.2.c. The proposed modification and the justification for it are given in GE Licensing Topical Report NEDO-21778-A.

#### 5.2.4.2.2 Method of Obtaining Operating Limits Based on Fracture Toughness

Operating limits that define minimum reactor-vessel metal temperatures versus reactor pressure during normal heatup, cool-down, inservice hydrostatic testing, and anticipated operational occurrences were established using the methods of Appendix G of Section III of the ASME Boiler and Pressure Vessel Code, 1971 Edition (Appendix G first appeared in the Summer 1972 Addenda). The results are shown in Figure 5.2-1.

*Estimated RT<sub>NPT</sub> values and temperature limits are given in this section for the limiting locations in the reactor vessel.*  
 All the vessel shell and head areas, remote from discontinuities, and the feedwater nozzles were evaluated, and the operating limit curves are based on the limiting location. The boltup limits for the flange and adjacent shell region are based on a minimum metal temperature of  $RT_{NPT} + 60^{\circ}\text{F}$ . The maximum through-wall temperature gradient from continuous heating or cooling at  $100^{\circ}\text{F}$  per hour was considered. The safety factors applied were as specified in Appendix G of the ASME Code and in GE Licensing Topical Report NEDO-21778-A.

For the purpose of setting these operating limits, the reference temperature,  $RT_{NPT}$ , is determined from the toughness test data taken in accordance with requirements of the ASME Code to which this vessel is designed and manufactured. This toughness test data, CVN and/or drop-weight NDTT, is analyzed to permit compliance with the intent of 10 CFR Part 50, Appendix G. Because not all toughness testing needed for strict compliance with Appendix G was required at the time of vessel procurement, some toughness results are not available. For example, longitudinal CVNs, instead of transverse CVNs, were tested for plate and forging materials. Also, at the time either CVN or NDT testing was permitted; therefore, in many cases for welds, it is expected that both tests were not performed as is currently required. To compensate for this absence of certain data, toughness property

correlations were derived for the vessel materials in order to operate upon the available data to give a conservative estimate of RT<sub>NDT</sub>, in order to comply with the intent of Appendix G criteria.

These toughness correlations vary, depending on the specific material analyzed. They were derived from the results of WRB Bulletin 217, "Properties of Heavy Section Nuclear Reactor Steels," and from toughness data from the Fermi 2 vessel and from other reactors. In the case of vessel plate material (SA-533, Grade B, Class 1), the predicted limiting toughness property is either NDT or transverse (CVN 50-ft-lb temperature minus 60°F). Longitudinal CVN transition curve results and NDT values are available for all Fermi 2 vessel plates. The transverse CVN 50-ft-lb transition temperature is estimated from longitudinal CVN data in the following manner. The lowest longitudinal CVN foot-pound value is adjusted to derive a longitudinal CVN 50-ft-lb transition temperature by adding 2 °F/ft-lb to the test temperature. If the actual data equal or exceed 50 ft-lb, the test temperature is used. If sufficient data are available, as in the case of Fermi 2, the 50-ft-lb temperature is derived by interpolation. Once the longitudinal 50-ft-lb temperature is derived, 30°F is added to account for orientation effects and to estimate the transverse CVN 50-ft-lb temperature minus 60°F, estimated in the preceding manner. Using this general approach, an initial RT<sub>NDT</sub> of -10°F<sub>or lower</sub> was established for plates in the core beltline region of Fermi 2.

23

For forgings (SA-508 Class 2), the predicted limiting property is the same as for the vessel plates. Both CVN and NDT values are available for the vessel flange and closure head flanges for Fermi 2. Only CVN results at +10°F are available for feedwater-nozzle forgings. For the flange forgings, RT<sub>NDT</sub> is estimated in the same way as for vessel plate, and an RT<sub>NDT</sub> value of 10°F was obtained.

For the feedwater-nozzle forgings, a maximum 40°F-NDT value was required by the purchase specification and there were no deviations from this requirement. The CVN results indicate a maximum RT<sub>NDT</sub> of +12°F. Therefore, an RT<sub>NDT</sub> of 40°F was used for the feedwater nozzles. +4°F.

For the vessel weld metal, the predicted limiting property is the CVN 50-ft-lb transition temperature minus 60°F, as the NDT values are -50°F or lower for these materials. This temperature is derived in the same way as for the vessel plate material, except that the 30°F addition for orientation effects is omitted since there is no principal working direction. When NDT values are available, they are also considered and the RT<sub>NDT</sub> is taken as the higher of the NDT or the 50-ft-lb temperature minus 60°F. When the NDT is not available, the RT<sub>NDT</sub> shall not be less than -50°F, because lower values are not supported by the correlation data. The limiting beltline RT<sub>NDT</sub> for Fermi 2, established from CVN beltline weld metal values, was -44°F. No toughness data were

available for nonbeltline welds; however, the purchase specification required an average of 30 ft-lb and a minimum of 25 ft-lb at +10°F. Quality assurance records show no deviations from these requirements, which produce an RT<sub>N</sub>DT value of 0°F for nonbeltline welds.

For vessel weld heat-affected zone (HAZ) material, the RT<sub>N</sub>DT is assumed to be the same as for the base material, as ASME Code weld-procedure, qualification test requirements indicate this assumption is valid.

*a 33 mil*

Toughness test requirements for closure-bolting material in Fermi 2 were for 30 ft-lb at 60°F below the boltup temperature. Current ASME Code requirements are for 45 ft-lb and 25 mils lateral expansion (MLE) at the preload or lowest service temperature. The reactor-vessel closure studs have a minimum CVN impact energy of 50 ft-lb and a 4-mil lateral expansion at 10°F for Fermi 2. Therefore, since CVN values for Fermi 2 studs exceed current requirements at 10°F, the lowest service temperature is +10°F.

The effect of the main closure flange discontinuity was considered by adding 60°F to the RT<sub>N</sub>DT to establish the minimum temperature for boltup and pressurization. The minimum boltup temperature of 71°F for Fermi 2, which is shown on Figure 5.2-1, is based on an initial RT<sub>N</sub>DT of +11°F for the shell plate connected to the closure-flange forging.

23

The effect of the feedwater-nozzle discontinuities was considered by adjusting the results of a BWR/6 reactor discontinuity analysis to the Fermi 2 reactor. The adjustment was made by increasing the minimum temperatures required by the difference between the Fermi 2 and BWR/6 feedwater nozzle forging RT<sub>N</sub>DT's. The feedwater nozzle adjustment was based on an RT<sub>N</sub>DT of 40°F.

#### 5.2.4.2.3 Temperature Limits for Preoperational System Hydrostatic Tests and ISI Hydrostatic or Leak Pressure Tests

Based on 10 CFR Part 50, Appendix G, IV.A.2.d, which allows a reduced safety factor for tests prior to fuel loading, the preoperational system hydrostatic test at 1563 psig may be performed at a minimum temperature of 150°F, which is established by the feedwater nozzle.

The fracture toughness analysis for system pressure tests resulted in the curves labeled A shown in Figure 5.2-1. The curve labeled feedwater nozzle is based on an initial RT<sub>N</sub>DT of 40°F. The beltline weld material is expected to be more limiting at end-of-service fluence levels, and this weld material has an initial RT<sub>N</sub>DT of -44°F.



The predicted shift in the RTNDT from Figure 5.2-2 (based on the neutron fluence at 1/4 of the vessel wall thickness) must be added to the beltline curve to account for the effect of fast neutrons.

#### 5.2.4.2.4 Temperature Limits for Boltup

A minimum temperature of 100°F is required for the closure studs. A sufficient number of studs may be tensioned at 70°F to seal the closure flange O-rings for the purpose of raising reactor water level above the closure flanges in order to assist in warming them. The flanges and adjacent shell are required to be warmed to minimum temperatures of 71°F before they are stressed by the full intended bolt preload. The fully preloaded boltup limits are shown on Figure 5.2-1.

#### 5.2.4.3 Operating Limits During Heatup, Cooldown, and Core Operation

23 The fracture toughness analysis was done for the normal heatup or cooldown rate of 100°F per hour. The temperature gradients and thermal stress effects corresponding to this rate were included. The results of the analyses are a set of operating limits for non-nuclear heatup or cooldown shown as curves labeled B on Figure 5.2-1. Curves labeled C on these figures apply whenever the core is critical. The basis for curves labeled C is described in GE BWR Licensing Topical Report NEDO-21778-A.

#### 5.2.4.4 Surveillance Programs for the Reactor Pressure Vessel

A surveillance program will be carried out to monitor the neutron radiation effects on the RPV base metal, the weld HAZ metal, and the weld metal from a steel joint that simulates a welded joint in the RPV beltline. For the extent of compliance to 10 CFR Part 50, Appendix H, see Table 5.2-10.

##### 5.2.4.4.1 Program Content

The program will consist of three baskets, each containing tensile and CVN specimens hermetically sealed in an inert gas environment in thinwall austenitic stainless steel capsules. The capsules are not buoyant and thus present no handling problems. The three baskets will be placed near core midplane adjacent to the RPV wall where the neutron flux and temperature will simulate that of the RPV wall. The three baskets contain test specimens made from the original RPV beltline material in accordance with the requirements of ASTM E185-66. In total, the program consists of 84 impact and 19 tensile specimens. In addition, there are 75 impact and 15 tensile baseline and spare specimens. The specimens will include the following:

- a. Base metal impact, transverse and longitudinal
- b. Weld metal impact
- c. HAZ impact

ATTACHMENT A

## EF-2-FSAR

- d. Base metal to tile
- e. Weld metal to tile
- f. HAZ tensile

The following general statements apply to these specimens:

- a. Base metal impact and tensile specimens are taken from the 1/4 T planes of the specimen plate.
- b. HAZ impact and tensile specimens are all oriented parallel to the rolling direction.
- c. Weld metal impact specimens are all transverse to the axis of the weld; tensile specimens are parallel. The fracture areas consist of all weld metal.

Details of the manufacture of these specimens are given in Reference 6.

The specimens were taken from two plates trimmed from the lower, intermediate shell section of the reactor vessel. The plate sections for the base material specimens were given a simulated stress relief for 40 hours at 1150°F to ensure that they represent the metallurgical condition of the lower, intermediate shell plates of the reactor vessel after final fabrication.

The plate sections for the weld and HAZ specimens were joined with a continuous central weld identical to the reactor vessel longitudinal weld. The welded plate was then given a simulated stress relief for 40 hours at 1150°F, similar to the base material plate. The weld was X-rayed to ensure quality; no repair to the weld was allowed by the specifications.

#### 5.2.4.4.2 Withdrawal Schedule

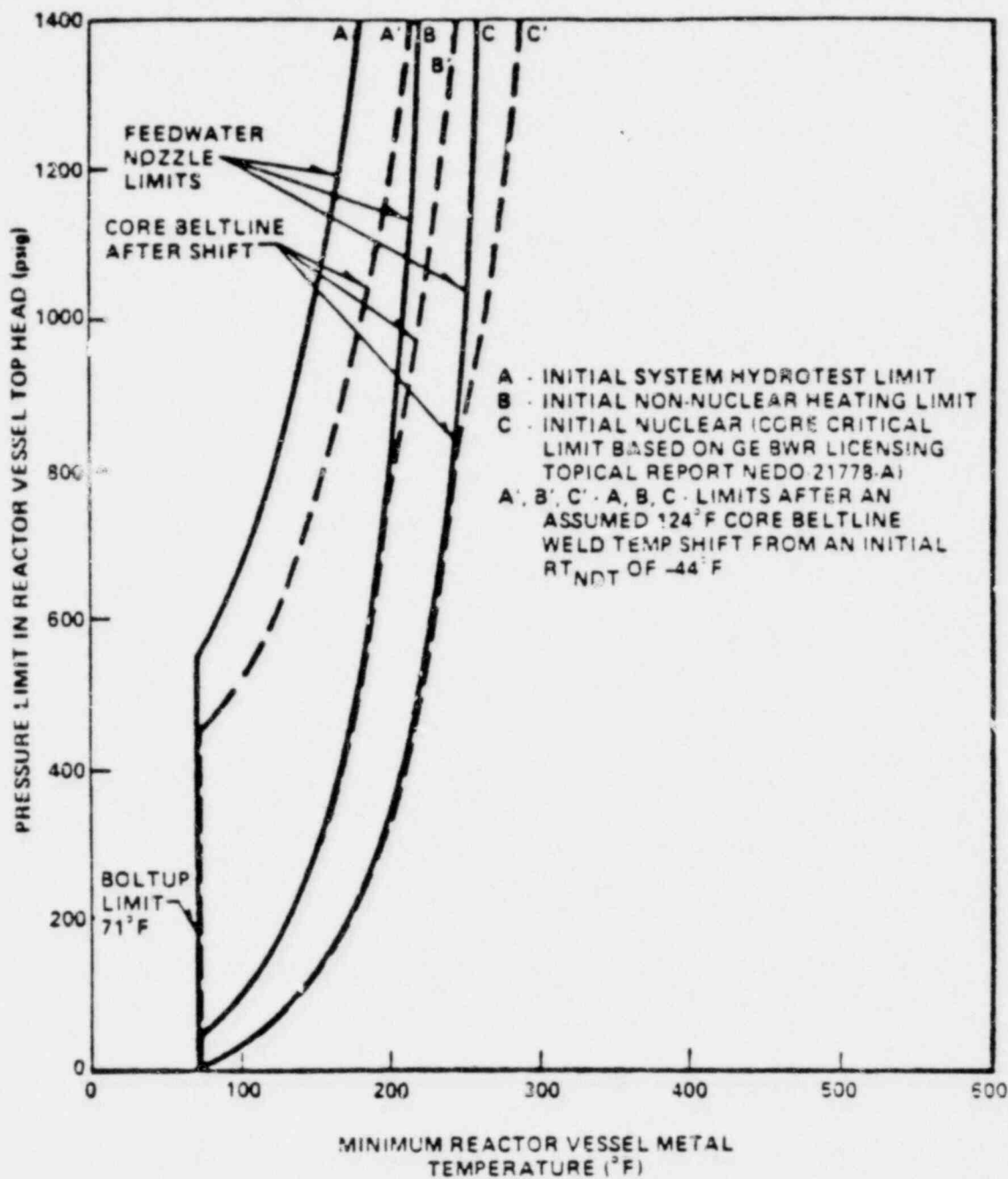
The withdrawal schedule of the three sets of specimens in the reactor is planned as follows:

- a. The first set will be withdrawn at 25 percent of the reactor service life.
- b. The second set will be withdrawn at 75 percent of the reactor service life.
- c. The third set will be a standby.

#### 5.2.4.5 Reactor Vessel Annealing

Inplace annealing of the reactor vessel because of radiation embrittlement is unnecessary because the predicted end-of-life value of adjusted reference temperature will not exceed 200°F (see 10 CFR Part 50, Appendix G, Paragraph IV.C).

# ATTACHMENT A EF-2-FSAR



ENRICO FERMI ATOMIC POWER PLANT  
UNIT 2  
FINAL SAFETY ANALYSIS REPORT

FIGURE 5.2-1  
MINIMUM TEMPERATURE REQUIRED  
VERSUS REACTOR PRESSURE

AMENDMENT 23 - MAY 1979