

**LICENSEE EVENT REPORT**

CONTROL BLOCK: 

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	G	A	E	I	H	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4			5
7	8	14						15	25										26	30					57	CAT	58		
		LICENSEE CODE							LICENSE NUMBER											LICENSE TYPE									

CON'T

0 1 7 8  
REPORT SOURCE L 6 0 5 0 0 0 3 2 1 7 0 7 0 1 8 1 8 0 7 1 7 8 1 9  
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While the plant was in steady state operation at 92% power and performing

0 3 Reactor Water Level (ADS) Functional Test and Calibration, both Reactor

0 4 Recirculation Pumps tripped when returning 1B21-N042B, Reactor Water

0 5 Level (ADS) Yarway, to service. Tech Specs 3.6.J.1 requires forced

0 6 circulation above 1% power. All ECCS systems were operable. A temporary

0 7 forced power reduction resulted. The health and safety of the public was

0 8 not affected. This is a non-repetitive event.

7 8 9 80

SYSTEM CODE 0 9		CAUSE CODE I B		CAUSE SUBCODE A		COMPONENT CODE C		COMPONENT SUBCODE I N S T R U		COMPONENT SUBCODE S		VALVE SUBCODE Z	
7 8		9 10		11 12		13 14		15 16		17 18		19 20	
17 LER/RO REPORT NUMBER		EVENT YEAR 8 1		SEQUENTIAL REPORT NO. —		OCCURRENCE CODE 0 6 6		REPORT TYPE /		REVISION NO. 0 3		—	
21 22		23 24		25 26		27 28		29 30		31 32		33 34	
ACTION TAKEN H		FUTURE ACTION Z		EFFECT ON PLANT B		SHUTDOWN METHOD Z		HOURS 0 0 1 0		ATTACHMENT SUBMITTED Y		NPRD-4 FORM SUB. N	
35 36		37 38		39 40		41 42		43 44		45 46		47 48	
18 19		20 21		22 23		24 25		26 27		28 29		30 31	
PRIME COMP. SUPPLIER N		COMPONENT MANUFACTURER B 0 8 0											

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of this event was personnel error. The technicians failed to

1 1 valve in instrument 1B21-N042B slowly enough. The resulting perturbation

1 2 in the common sensing line caused Reactor High Pressure Recirc Pump Trip

1 3 switches to actuate. Personnel were counseled on proper instrument

1 4 valving.

7 8 9  
FACILITY STATUS (1) 5 (2) E (28) % POWER (10) 0 (11) 9 (12) 2 (29) OTHER STATUS (30) NA METHOD OF DISCOVERY (31) B (32) Surveillance Testing  
3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

ACTIVITY CONTENT  
RELEASED OF RELEASE

1 6 Z 33 Z 34 NA

3 4 5 6 7 8 9 10 11 12 13 14

AMOUNT OF ACTIVITY (35)

LOCATION OF RELEASE (36)

NA

15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL EXPOSURES

NUMBER		TYPE	DESCRIPTION
1	7	000 (37) Z (38)	NA

PERSONNEL INJURIES					
NUMBER			DESCRIPTION		
1	8		0	0	0
			(40)		NA

1 9 Z (42) LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43) NA 8107280235 810717

PUBLICITY  
 ISSUED DESCRIPTION (45) PDR ADACK 05000321  
 S PDR  
 NRC USE ONLY  
 2 0 N (44) NA

NAME OF PREPARER R. T. Nix, Supt. of Maint.

PHONE: 912-367-7781

LER No.: 50-321/1981-066  
Licensee: Georgia Power Company  
Facility: Plant E. I. Hatch  
Docket No.: 50-321

Narrative Report  
for LER 50-321/1981-066

On July 1, 1981, while the plant was in steady state operation at 92% power and performing Reactor Water Level (ADS) Functional Test and Calibration, both Reactor Recirculation Pumps tripped when returning 1B21-N042B, Reactor Water Level (ADS) Yarway, to service. Tech Specs 3.6.J.1 requires forced circulation above 1% power. All ECCS systems were operable. This event resulted in a temporary forced power reduction. The health and safety of the public was not affected.

The cause of this event was personnel error. The technician failed to valve in instrument 1B21-N042B slowly enough. The resulting perturbation in the common sensing line caused Reactor High Pressure Recirc Pump Trip switches to actuate. Personnel were counseled on proper instrument valving.

A generic review revealed no inherent problems.