

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE C B (11)		CAUSE CODE E (12)		CAUSE SUBCODE D (13)		COMPONENT CODE H T E X C H (14)		COMP. SUBCODE F (15)		VALVE SUBCODE Z (16)	
LER/RO REPORT NUMBER 8 1 (17)		EVENT YEAR 8 1 (21) 22		SEQUENTIAL REPORT NO. 0 0 8 (24) 26		OCCURRENCE CODE 0 1 (28) 29		REPORT TYPE T (30)		REVISION NO. 0 (32)	
ACTION TAKEN B (18)		FUTURE ACTION Z (19)		EFFECT ON PLANT Z (20)		SHUTDOWN METHOD Z (21)		HOURS 0 0 0 0 (22) 24		ATTACHMENT SUBMITTED Y (23)	
NPRD-4 FORM SUB. Y (24)		PRIME COMP. SUPPLIER N (25)		COMPONENT MANUFACTURER W 1 2 0 (26) 28							

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

Three of the indications were within the crevice and are considered to be IGA caused by caustic. The fourth indication is 1/2" above the tubesheet and is believed to be a remnant of phosphate wastage. The four tubes with defects have been plugged. A crevice flush will be conducted before startup and operation of the unit at reduced temperature will continue.

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PUBLICITY				PDR ADOCK 05000266										NRC USE ONLY
ISSUED				S										
DESCRIPTION				PDR										
2	0	N	(44)	N/A										

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ATTACHMENT TO LICENSEE EVENT REPORT NO. 81-008/01T-0

Wiscorsin Electric Power Company
Point Beach Nuclear Plant, Unit 1
Docket No. 50-266

On July 10, 1981, verification of all initial steam generator eddy current data for tubes with indications exceeding the plugging limit was completed. Two tubes in the "A" steam generator and two tubes in the "B" steam generator were verified to have degradation greater than 40%, which is the plugging limit of Technical Specification 15.6.2.A.5.

On July 4, 1981, Unit 1 was shut down for steam generator eddy current inspection. The 2000 psid primary-to-secondary hydrostatic test condition was established while the unit was being cooled down. An 800 psig secondary-to-primary leak check was performed on July 6 and 7. Detailed inspections of the tubesheets with remote video equipment showed a total of nine explosive plugs which were either wet, boric acid coated, or dripping at a slow rate (less than one drop every one and one-half minutes). Of the nine plugs, five had similar observations noted in previous outages. Based upon the low leak rate before shut-down (four gallons per day) and potential steam generator demonstration sleeving during the fall 1981 refueling outage, the plugs were not repaired during this outage. The fact that weld repair of an explosive plug involves a relatively high personnel radiation exposure was also a major factor in deciding not to repair the explosive plugs at this time. The specific conditions noted during the leak check are noted below.

"A" Steam Generator

R03C09	explosive plug, wet, less than 1 drip/2 minutes
R19C33	explosive plug, wet, less than 1 drip/2 minutes
R24C37	explosive plug, wet, 1 drip/1.5 minutes
R33C50	explosive plug, boric acid coated
R21C49	explosive plug, wet end, no drips
R04C59	mechanical plug, boric acid ring, appears wet inside
R08C50	mechanical plug, boric acid ring, appears wet inside

"B" Steam Generator

R23C53	explosive plug, wet end, no drips
R29C35	explosive plug, wet, 1 drip/2 minutes
R04C46	mechanical plug, boric acid coated, appears wet inside
R29C41	weld repair, boric acid coated
R28C39	explosive plug, boric acid coated
R29C37	explosive plug, boric acid coated

Of the wet end and dripping plugs, the following have been noted in previous inspections:

R19C33	"A" steam generator
R24C37	"A" steam generator
R21C49	"A" steam generator
R23C53	"B" steam generator
R29C35	"B" steam generator

The eddy current inspection consisted of remote inspection of all readily accessible tubes to the first tube support plate. Of the 2,853 open tubes in the "A" steam generator, 2,814 were inspected and 2,816 of the 2,861 open tubes in the "B" steam generator were inspected. The tubes that were not inspected are tubes under the eddy current fixture foot (19 in "A" and 17 in "B"), the tubes that contain template plugs (17 in both "A" and "B", randomly located), and tubes with restricted tube ends which prevented the insertion of a .720 probe (3 in "A" and 11 in "B"). These tubes were not inspected because of the exposure associated with moving template plugs, hand probing, and opening tube ends. The non-inspected tubes constitute less than 2% of the unplugged tubes, most are not located in the zone where large numbers of defects have occurred and the overall eddy current results did not indicate that it would be necessary to inspect the tubes. The overall eddy current results and comparison with the December 1980 tapes are listed below:

"A" Steam Generator

<u>Tube</u>	<u>Indication</u>	<u>December 1980 Reported</u>	<u>Comparison With December 1980 Tapes</u>
R15C29	43%, 21" ATE	NDD	31%, small change
R23C36	49%, 8" ATE	NDD	small change
R20C20	<20%, 5" ATE	NDD	NDD, small change
R05C07	UDI, 15" ATE	NDD	no change
R18C37	UDI, 8" ATE	NDD	small change
R11C43	UDI, 18-20" ATE	UDI	no change
R23C43	UDI, 5-7" ATE	NDD	small change
R25C44	UDI, 13-15" ATE	UDI	no change
R25C47	UDI, 13-18" ATE	UDI	no change
R08C55	UDI, 17" ATE	UDI	no change
R15C60	UDI, 14-18" ATE	UDI	no change
R18C68	UDI, 15-17" ATE	NDD	small change
R10C54	32%, TTS	<20%	34% in July 1980
F 3C54	<20%, 1/2" ATS	22%	34% in July 1980

ATE - Above Tube End
 NDD - No Defect Detected
 UDI - Undefinable Indication
 ATS - Above Tubesheet

"B" Steam Generator

<u>Tube</u>	<u>Indication</u>	<u>December 1980 Reported</u>	<u>Comparison With December 1980 Tapes</u>
R03C25	80%, 17" ATE	NDD	small change
R06C43	33%, 21" ATE	NDD	no change
R16C47	52%, 1/2" ATS	NDD	no change
R27C30	29%, 1/2" ATS	29%	

ATE - Above Tube End
 NDD - No Defect Detected
 UDI - Undefinable Indication
 ATS - Above Tubesheet

As in the past, all indications were small volume.

Besides the above listed indications, a number of restrictions were encountered with the .720 eddy current probe. In the "A" steam generator three tube ends and 23 first tube support plate restrictions were encountered. In the "B" steam generator 11 tube ends and 40 first tube support plate restrictions were encountered. Most of the same restrictions were noted in the July and December 1980 inspections. During the December 1980 inspection all but seven of the restrictions passed a .700 eddy current probe. Based on this experience, the restricted tubes were not gauged. Gauging of restricted tubes may be done during the refueling outage inspection.

The four tubes with indications greater than 40% were plugged on July 11, 1981. In addition, tube R06C43 in the "B" steam generator was also plugged. Correct plugging was visually verified the same day.

A crevice flush will be performed before the unit is returned to service in an attempt to remove impurities from the tubesheet crevice. Operation of the unit at a reduced temperature in order to reduce the corrosion rate will continue. However, the unit will be returned to service at a slightly higher temperature than that at which it has been operated since December 1979, assuming the results of the crevice flushing are satisfactory.

The NRC Resident Inspector was notified of these findings. This event is reportable in accordance with Technical Specification 15.6.9.A.3 and is similar to others.

Unit 1 return to power is scheduled for about July 20, 1981.