

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun #1

DATE July 15, 1981

COMPLETED BY R. W. Short

TELEPHONE (402)536-4543

MONTH June, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>188.9</u>
2	<u>189.3</u>
3	<u>189.2</u>
4	<u>188.5</u>
5	<u>186.0</u>
6	<u>185.0</u>
7	<u>185.8</u>
8	<u>225.3</u>
9	<u>293.5</u>
10	<u>305.2</u>
11	<u>305.7</u>
12	<u>305.9</u>
13	<u>305.1</u>
14	<u>303.9</u>
15	<u>306.1</u>
16	<u>250.3</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>210.8</u>
18	<u>305.9</u>
19	<u>363.0</u>
20	<u>368.0</u>
21	<u>368.9</u>
22	<u>376.0</u>
23	<u>422.3</u>
24	<u>435.4</u>
25	<u>434.5</u>
26	<u>439.0</u>
27	<u>441.0</u>
28	<u>441.0</u>
29	<u>440.0</u>
30	<u>439.9</u>
31	<u></u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE July 15, 1981
COMPLETED BY R. W. Short
TELEPHONE (402) 536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: June, 1981
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
N/A

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.0	4,343.0	68,064.0
12. Number Of Hours Reactor Was Critical	720.0	4,210.0	53,988.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	713.4	4,161.4	52,921.3
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	744,754.5	4,096,843.5	63,815,083.8
17. Gross Electrical Energy Generated (MWH)	240,476.0	1,346,474.0	21,134,745.6
18. Net Electrical Energy Generated (MWH)	225,592.8	1,259,040.0	19,957,222.9
19. Unit Service Factor	99.1	95.8	77.8
20. Unit Availability Factor	99.1	95.8	77.8
21. Unit Capacity Factor (Using MDC Net)	65.5	60.2	64.2
22. Unit Capacity Factor (Using DER Net)	65.5	60.2	63.8
23. Unit Forced Outage Rate	0.9	4.2	4.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling, September 15, 1981, 4-7 Weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June, 1981

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun #1
 DATE July 15, 1981
 COMPLETED BY R. W. Short
 TELEPHONE (402) 536-4543

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-05	810616	F	6.5	H	1	N/A	MA	MECFUN	Cause: Defective potentiometer in load limit control. Corrective Action: Replaced potentiometer.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending June 1981.

1. Scheduled date for next refueling shutdown. September 15, 1981
2. Scheduled date for restart following refueling. November 15, 1981
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
No
 - a. If answer is yes, what, in general, will these be?
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.
No
 - c. If no such review has taken place, when is it scheduled?
August 1, 1981
4. Scheduled date(s) for submitting proposed licensing action and support information.
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>197</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
1985

Prepared by J K Hager

Date July 1, 1981

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

June 1981
Monthly Operations Report

I. OPERATIONS SUMMARY

The unit operated at power levels from 66 to 95% reactor power during June. One outage occurred on June 16, due to a failed turbine load limit potentiometer. The unit was returned to service early June 17.

Preparation for the SRO examination for three candidates continued during June.

The Supervisor-Operations attended a one day meeting at the Combustion Engineering Facilities in Windsor, Conn. on Inadequate Core Cooling.

Surveillance tests and operational tests were performed.

No safety valve or PORV challenges occurred on the primary system.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
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NONE

B. CHANGES IN OPERATING METHODS

None

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance tests schedule. The following is a summary of the surveillance tests which results in Operations Incidents and are not reported elsewhere in the report:

<u>Operations Incident</u>	<u>Deficiency</u>
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OI-1325

ST-RM-4

OAE Sampler out of service.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
EEAR/DCR #FC-74A-21	<p>Steam Generator Blowdown Tie Ins./Completed as designed.</p> <p>This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 since it functions to reduce the chance of an uncontrolled radioactive release due to a primary to secondary release.</p>
SP-VA-80	<p>Hydrogen Purge System Test</p> <p>This procedure did not constitute an unreviewed safety question as defined in 10CFR50.59 since it is only a test to demonstrate the operability of the hydrogen purge system.</p>
SP-FAUD-1	<p>Fuel Assembly Uplift Condition Detection the two operable Δp channels shows greater than 99% assurance that the 150 pound criterion is not being violated.</p> <p>An unreviewed safety question as defined in 10CFR50.59 did not exist as this procedure only involves evaluating data from a surveillance test.</p>
EEAR FC-80-45	<p>RCP Cooling Pipe Replacement. Completed as designed.</p> <p>This modification did not constitute an unreviewed safety question as defined in 10CFR50.59 as it only involved replacing some of the reactor coolant pump cooling piping with stainless steel rather than carbon steel piping using appropriate nuclear grade material.</p>
FC-78-141 72-17	<p>Seal Water Mod. for Circ. Water Pumps and Raw Water Pumps.</p> <p>These modifications did not constitute an unreviewed safety question as defined in 10CFR50.59 as they only involved replacing and improving the seal water piping to the circulating water pumps and the raw water pumps.</p>

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL (Continued)

EEAR FC-81-53
SRDCO #81-37

Deletion of Clutch Power to Part Length CEDM's

An unreviewed safety question as defined in 10CFR50.59 did not exist as the operational method of operating rod drives was not changed. The rods in question do not have clutches and could not have been affected in anyway.

E. RESULTS OF LEAK RATE TESTS

NONE

F. CHANGES IN PLANT OPERATING STAFF

Merl Core has been assigned as acting I&C and Electrical Field Supervisor.

Bruce Hickle has been assigned as acting Chemical & Rad. Prot. Supervisor.

Tom Chapman has been assigned as acting Health Physicist.

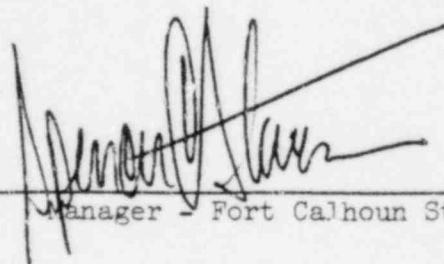
G. TRAINING

Training for June at the Fort Calhoun Station included general employee training for both new and plant personnel, operator training, monitor team training and Emergency Preparedness training for offsite Support Personnel.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

NONE

Approved By



Manager - Fort Calhoun Station

Monthly Operations Report

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II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
10093	5-29-81	RM-080 is spiking into "Alert"	Cleaned switch and contacts observed for 4-days. No other problems.
10400	6-9-81	DG-2 DC pr. supply (normal supply replaced tripped and would not reclose).	Bad Ann. Card A-1 on A-37 ann. and fuse. Installed new power supply. Diesel remained operable.
9876	4-14-81	Part Length Rod #18 drives in when not selected. Also drives in when selected and moving "Rod Block" switch to bypass.	Separated clutch power from cables going to CEDM's. Refer to EEAR FC-81-58 SRDCO #81-37
10243	5-19-81	Air sampler at left station (OAE) has high flow.	After replacing filter & cartridge problem could not be duplicated.
10241	5-19-81	VOPT channel C reset demand push button on CB 4 does not reset setpoints.	Replaced switch preformed ST-RPS-1 F.3.
10019	4-28-81	Repair breaker and cable which feed AI-41B.	Replaced cable & breaker CB-22 checked for grounds none. Returned to normal. See LER 81-03.