OPERATING DATA REPORT

DOCKET NO. 50-289
DATE July 15, 1981
COMPLETED BY D. G. Mitchell
TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: June, 1981
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.	4343.	59856.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	52.1
20. Unit Availability Factor	0.0	0.0	52.1
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	50.7
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	48.6
23. Unit Forced Outage Rate	100.0	100.0	40.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-I
DATE July 15, 1981
COMPLETED BY D. G. Mitchell
TELEPHONE (717) 948-8553

MONTH June, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289
UNIT NAME TMI-1
DATE July 15, 1981
COMPLETED BY D. G. Mitchell
TELEPHONE (717) 948-8553

REPORT MONTH June

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	6/1/81	F	720	D	1				Regulatory Restraint Order

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

OPERATIONS SUMMARY

The plant remained in cold shutdown the entire month by order of the NRC. The Decay Heat Removal System provided core cooling.

MAJOR SAFETY RELATED MAINTENENCE

During the month of June the following major maintenance items were performed.

1) Local Leak Rate Test (LLRT)

This test was completed during this month with satisfactory results. The total as left leakage was within the acceptance criteria required for preparation of the Integrated Leak Rate Test of the Reactor Building.

2) Integrated Leak Rate Test (ILRT)

Preparation for the test began with the following items being performed.

a) Instrument calibrations.
b) Reactor Building inspected.
c) Equipment protected in Reactor Building.
d) Gas bottles, flamables, oil, etc. removed.
e) Manometers installed.
f) Test panel area setup (tables, lights, etc.).

Building pressurization began during July and was successfully completed. Details will be reported in the July Report.

3) Purge Valve AH-V-1A

Inspection of the bearings for purge valve AH-V-1A was performed and the following work was performed.

a) Removed pneumatic operator.
b) Removed bearing/inspect bearings.
c) Regreased bearings.
d) Installed bearings.
e) Reinstalled operator.

After completion of the bearing inspection work, the rubber seat in the valve was replaced. Replacement work included opening the direct access port, removal of old rubber material, clean up of seat area, reinstallaton of new rubber seats, torquing of segment bolts, etc. A preliminary leak check was performed to see if any leaks were present. The access port was closed and a satisfactory, final leak check was performed.

4) Decay Heat River Water Pump (DR-P-1A)

Pump work progressed with the pump being reassembled, motor balanced and coupled to pump. On line balancing of the pump and motor was unsuccessfully attempted. The motor and pump were removed and inspections are being performed to determine the cause of high vibration. This work continued into July and will be discussed in the July Report.

REFUELING INFORMATION REQUEST

1. Name of Facility:

   Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

   Unknown

3. Scheduled date for restart following refueling:

   Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   If answer is yes, in general, what will these be?

   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   If no such review has taken place, when is it scheduled?

   Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

   N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

   (a) 177

   (b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).