

80-053-032 ✓
ATTACHMENT 1

SURKY POWER STATION, UNIT 1

DOCKET NO: 50-280

REPORT NO: 80-067/OIX-1

EVENT DATE: 10-28-80

Principal
OFFICIAL COPY



MAIN STEAM PIPING WELD DESCREPARNCIES

1. DESCRIPTION OF EVENT

With the Unit defueled, radiography examination of the Main Steam elbow at the Steam Generator outlet nozzle revealed indications in the weld area. The indications were found on all three Steam Generators. This examination was being preformed as required by ETA 30006 in preparation for Steam Generator replacement. The indications are judged to be artifacts of the welding process and not service induced. This event is reportable per Technical Specification 6.6.2.a.(9).

2. PROBABLE CONSEQUENCES

The integrity of the Main Steam Piping may have been degraded. Since the Unit is in a defueled condition, the health and safety of the public was not affected.

3. CAUSE

The indications were the results of original construction of the main steam system.

4. IMMEDIATE CORRECTIVE ACTIONS

A review of the original construction records was initiated to determine the origin of these indications. The results of the review revealed that these indications did exist, and were originally judged to be acceptable.

5. SUBSEQUENT CORRECTIVE ACTIONS

The subject weld areas will be completely machined out and replaced during the Steam Generator Replacement.

6. ACTION TAKEN TO PREVENT RECURRENCE

None

7. GENERIC IMPLICATIONS

An evaluation of the original radiographs of the main steam piping inside the containment is in progress. Approximately 40% of the original radiographs have been evaluated and no rejectable indications have been identified. In addition, RT examination of some of the welds will be conducted to confirm the results of the above evaluation. The evaluation and RT examination will be completed prior to hydrostatic testing of the Steam Generators. A similar review of original construction radiographs for Unit No. 2 is in progress. It should be noted that the nozzle to elbow welds for unit No. 2 were replaced during the Steam Generator age.

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ATTACHMENT 2

Surry Power Station Unit 1

Docket No: 50-280

Report No: 80-067/01X-1

Event Date: 10-28-80

UPDATED REPORT

1.) DESCRIPTION

While conducting RT examination as described in the "Generic Implications" to LER-80-067/01T-0, linear indication was discovered in the "A" Loop main steam piping.

2.) PROBABLE CONSEQUENCES

Since the unit was in a defueled condition, the health and safety of the public were not affected.

3.) CAUSE

The cause of the indication is under investigation.

LICENSEE EVENT REPORT

CONTROL BLOCK: _____

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 V A S F S 1 2 0 0 - 0 0 0 0 0 - 0 3 4 1 1 1 1 4 9
8 9 14 15 24 25 30 31 CAT 38
 LICENSE CODE LICENSE NUMBER LICENSE TYPE CAT 38

01 REPORT SOURCE L 6 0 5 0 0 0 2 8 0 7 0 2 2 5 8 1 8 0 2 2 5 8 1 9
60 61 66 68 74 75 80
 DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 RT examination of loops B & C Main Steam Piping, weld nos. 36 & 22 respectively,
 03 revealed linear indications. The RT examination was being conducted as a follow-up
 04 to the findings reported in LER 80-067/01T-1. Since the unit is in a defueled
 05 condition, the health and safety of the public were not affected.

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09 SYSTEM CODE 11 CAUSE CODE 12 CAUSE SUBCODE 13 COMPONENT CODE 14 COMP. SUBCODE 15 VALVE SUBCODE 16
9 10 11 12 13 14 15 16
 17 LER/NO REPORT NUMBER 18 EVENT YEAR 19 EFFECT ON PLANT 20 SHUTDOWN METHOD 21 HOURS 22 ATTACHMENT SUBMITTED 23 NRC-4 FORM 348 24 PRIME COMP. SUPPLIER 25 COMPONENT MANUFACTURER
21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 The cause of the indications (defects) is under investigation. The defects will be
 11 repaired.

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15 FACILITY STATUS 28 S. POWER 29 OTHER STATUS 30 METHOD OF DISCOVERY 31 DISCOVERY DESCRIPTION 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

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