OPERATING DATA REPORT

DOCKET NO. 50-346
DATE July 8, 1981
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 259-5000
Ext. 251

OPERATING STATUS

1. Unit Name: Davis-Besse Unit #1
2. Reporting Period: June, 1981
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720	4,343	33,652
12. Number Of Hours Reactor Was Critical	565.4	3,025.6	17,409.8
13. Reactor Reserve Shutdown Hours	0	34.8	2,916.9
14. Hours Generator On-Line	565.4	2,879.0	15,926.8
15. Unit Reserve Shutdown Hours	0	0	1,731.4
16. Gross Thermal Energy Generated (MWH)	1,146,943	6,392,283	33,297,089
17. Gross Electrical Energy Generated (MWH)	383,055	2,138,383	11,113,717
18. Net Electrical Energy Generated (MWH)	362,837	2,002,585	10,267,086
19. Unit Service Factor	78.5	66.3	48.1
20. Unit Availability Factor	78.5	66.3	53.5
21. Unit Capacity Factor (Using MDC Net)	56.2	51.8	36.1
22. Unit Capacity Factor (Using DER Net)	55.2	50.9	35.5
23. Unit Forced Outage Rate	21.5	25.9	24.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: July 21, 1981
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
UNIT Davis-Besse, Unit 1
DATE July 8, 1981
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 259-5000 Ext. 251

MONTH June, 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	645	17	651
2	642	18	649
3	637	19	651
4	638	20	650
5	638	21	649
6	639	22	651
7	638	23	650
8	637	24	360
9	638	25	0
10	644	26	0
11	645	27	0
12	640	28	0
13	634	29	0
14	630	30	0
15	626	31	
16	635		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346
UNIT NAME Davis-Besse Unit #1
DATE July 8, 1981
COMPLETED BY Bilal Sarsour
TELEPHONE (419) 259-5000, Ext. 251

REPORT MONTH June, 1981

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	81 06 24	F	154.6	H	3	NP-33-81-44	EB	CTBRK	The reactor was tripped due to a loss of E2 power during control rod drive (CRD) breaker logic surveillance testing. Loss of E2 was due to mechanical shock from construction in the area.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4. Continuation
5. Reduction
6. Other

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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OPERATIONAL SUMMARY

JUNE, 1981

6/1/81 - 6/24/81

The reactor power was maintained at between 72 and 74 percent power with three reactor coolant pumps operating (the 2-1 RCP was shutdown on May 12, 1981 due to high seal return temperature) until 1324 hours on June 24, 1981 when the reactor tripped. The incident was initiated by an actuation of a ground fault relay on a non-essential breaker (due to mechanical shock from construction in the area) resulting in a loss of the control rod drive (CRD) inductrol power supply. This was concurrent with a control rod drive breaker logic surveillance test de-energization of the opposite control rod drive supply breaker. This resulted in a loss of power to the control rods, and the rods dropped into the core.

6/25/81 - 6/30/81

A planned shutdown was initiated to repair the 2-1 reactor coolant pump seals.

REFUELING INFORMATION

DATE: June, 1981

1. Name of facility: Davis-Besse Nuclear Power Station Unit 1

2. Scheduled date for next refueling shutdown: March, 1982

3. Scheduled date for restart following refueling: May, 1982

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Reload analysis is scheduled for completion as of December, 1981. No technical specification changes or other license amendments identified to date.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. January, 1982

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None identified to date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177 (b) 44 - Spent Fuel Assemblies
8 - New Fuel Assemblies

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present 735 Increase size by 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date 1988 (assuming ability to unload the entire core into the spent fuel pool is maintained)