



LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION

P.O. BOX 618, NORTH COUNTRY ROAD • WADING RIVER, N.Y. 11792

July 15, 1981

SNRC-594

Mr. Harold R. Denton
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



SHOREHAM NUCLEAR POWER STATION - Unit 1
Docket No. 50-322

Dear Mr. Denton:

Enclosed herewith are sixty (60) copies of LILCO responses to specific NRC concerns which were previously identified as requiring additional information to complete NRC review. Attachment A provides a list of the specific responses included.

If you require additional information or clarification, please do not hesitate to contact this office.

Very truly yours,

B. R. McCaffrey

B. R. McCaffrey
Manager, Project Engineering
Shoreham Nuclear Power Station

Enclosures

cc: J. Higgins

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Attachment A

Additional information is provided for the following items:

1. SER Open Item No. 4 - Safety Related Snubbers which will be listed in the Tables of STS 3/4.7.9.
2. SER Open Item No. 48 - High Energy Line Breaks
3. SER Open Item No. 55 - Q-List
4. NUREG-0737 Item I.C.7 - NSSS Vendor Review of Procedures
5. NUREG-0737 Item II.K.3.22 - Automatic Switchover of Reactor Core Isolation Cooling System Suction.

Item #4 - Safety Related Snubbers which will be listed in the Tables of STS 3/4.7.9.

Prior to installation, all safety related snubbers are operationally stroked to determine that they are not frozen, seized or jammed. Upon completion of installation, all snubbers are inspected to assure that there are no visual signs of damage, impaired operability and that snubber location, orientation, position setting and configuration are according to design drawings and specification. Each snubber is checked to assure that adequate swing clearance is provided and that structural connections are installed correctly. These inspections are performed in accordance with Construction Procedure CIP 6.2 and Quality Control Procedure Q.C. 11.4. After inspection, each snubber is wrapped and protected from damage in accordance with site instructions. No more than 6 months prior to system preoperational testing, snubbers are unwrapped and visually inspected for signs of damage, impaired operability and adequate swing clearance in accordance with Startup Procedure CG 000.36. Fluid level and leakage are not applicable to Shoreham as hydraulic snubbers are not employed in any safety related systems.

The snubber thermal movements for those systems that have operating temperatures in excess of 250°F will be verified during the Startup Test phase under STP-811--"System Thermal Expansion Test Procedure". This includes verification of the expected snubber thermal movements and swing clearances. Discrepancies or inconsistencies will be dispositioned prior to proceeding to the next test plateau. A preliminary list of the snubbers is attached. The final list will be included in the station technical specification.

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#48 HIGH ENERGY LINE BREAKS

The applicant will supplement the high energy line break analysis (Appendix 3C) to specifically include the consequential effects as related to non-safety grade control systems. The present analysis does not take credit for non-safety grade control systems to demonstrate reactor shutdown capability. The purpose of this analysis is to determine if the pipe break event affects any non-safety grade control system in such a manner as to worsen the event and, if so, to confirm shutdown capability and that the event is bounded by Chapter 15 analyses. The pending supplemental analysis will be provided by September 30, 1981.

Open Item 55 - Q List

Item A. Conduit and Cable Trays and their supports in installations containing Class IE cables and other installations whose failure during a seismic event may damage safety-related items.

LILCO Response

The replacement or modification of conduit, cable trays and their supports is controlled by the Station Modification Program or Station Maintenance Work Request Program. Each of these programs is delineated and controlled by approved station procedures. The Operational Quality Assurance (OQA) organization is an integral part of each program from both a review and audit standpoint. Routine inspections are made by Operations field personnel as part of normal plant rounds. Pertinent information is logged by Operations Personnel in accordance with approved procedures. The OQA Organization will apply the pertinent portions of the QA Program by auditing the implementation of the above mentioned procedures.

Item B. Main Steam Isolation Valves Leakage Control System

LILCO Response

Modification or maintenance of the main steam isolation valve leakage control system will be conducted in accordance with the Station Modification or Station Maintenance Work Request Programs. Each of these programs is delineated in and controlled by approved station procedures. Main Steam Isolation Valves Leakage Control System is controlled by the station technical specifications. The OQA Organization is an integral part of each program from both a review and audit standpoint. The OQA Organization will apply the pertinent portions of the QA Program by auditing the implementation of these procedures and station compliance with technical specifications.

Item C. Various Radiation Measuring, Monitoring, and Clean-up Items

LILCO Response

The various radiation measuring, monitoring and clean-up items associated with the emergency plan are controlled and calibrated in accordance with the Station Health Physics Program. This program is delineated in and controlled by approved station procedures. In addition, there are the Emergency Plan, Emergency Plan Implementing Procedures, and required drills. The OQA Organization applies the pertinent portions of the QA Program by auditing the implementation of these procedures, the Plan and drills.

Item D. Site Grading

LILCO Response

This item was responded to in SNRC-577. No further action is required.

Item E. Sentiment Measurements in the Intake Canal

LILCO Response

Intake canal measurements referenced above are controlled by the station technical specifications including acceptance criteria and surveillance requirements. The OQA Organization will apply the pertinent portions of the QA Program by auditing station compliance with technical specifications.

Item F. Accident-Related Meteorological Data Collection Equipment

LILCO Response

The meteorological data collection system will be inspected and calibrated on a periodic basis as part of the radiation monitoring system. The program will be audited by QA.

Item G. Expendable and Consumable Items Necessary for the Functional Performance of Safety-Related Structures, Systems, and Components.

LILCO Response

Procurement of expendable and consumable items is accomplished and controlled by approved station procedures. The OQA Organization is an integral part of the procurement review process. The OQA Organization, during this review process, applies the pertinent portions of the QA Program such as documentation, inspection requirements, etc., consistent with the item(s) being procured. In addition, the OQA Organization audits the implementation of the procurement control procedures.

Item H. C.W. Intake Pipes (Service Water Piping)

LILCO Response

The service water system is already identified as a Q listed system. No further action is required.

Item I. Intake Canal Slopes
Item J. Rock Jetty - Long Island Sound
Item K/L. Sheet Pile Retaining Walls (on the side of Screenwell) and Headwalls.

LILCO Response

Item J is controlled by the station technical specifications including acceptance criteria and surveillance requirements. The OQA Organization will apply the pertinent portions of the QA Program by auditing station compliance with technical specifications.

Items I, K and L are controlled by the Preventative Maintenance Program. This program is delineated and controlled by approved station procedures. The Operational Quality Assurance organization is an integral part of the program from both a review and audit standpoint.

Item M. The following Items from NUREG-0737:

- Plant Safety Parameter Display Console (I.D.2)
- Post Accident Sampling Capability (II.B.3)
- Valve Position Indication (II.D.3)
- Emergency Plans (III.A.1.1/III.A.2)
- Equipment and Other Items Associated with Emergency Support Facilities (III.A.1.2)

LILCO Response

The plant safety parameter display console, post accident sampling equipment and safety relief valve positive position indicators are maintained and calibrated in accordance with the requirements delineated in the Station Maintenance Work Request and Modification Procedures. The OQA Organization applies the pertinent portions of the QA Program by auditing the implementation of these procedures. The emergency plan, including the emergency support facility, is controlled in accordance with the approved Emergency Plan Program and Emergency Plan Implementing Procedures. The OQA Organization will apply the pertinent portions of the QA Program by auditing the conduct of emergency drills and the implementation of the program.

I.C.7 NSSS Vendor Review of Procedures

The following position revises the commitment made in SNRC-563, dated 5/15/81:

The Site Operations Manager of the General Electric Company, the nuclear steam supply system vendor, reviews and approves the procedures for the low power and power ascension test program. His approval is required by the station procedure administering the Startup Testing Program prior to the release for performance of any NSSS startup test procedure.

Emergency procedures are being prepared using the Emergency Procedures Guidelines developed by the Emergency Procedures Subgroup of the BWR Owners Group and the General Electric Company. These Emergency Procedures will be reviewed by the General Electric Company Site Operations Manager prior to Fuel Load.

II.K.3.22 Automatic Switchover of Reactor Core Isolation Cooling
System Suction

The RCIC system will be modified to automatically switch over the suction from the Condensate Storage Tank to the suppression pool upon Condensate Storage Tank low level. Implementation of the modification prior to fuel load is contingent upon timely delivery of Class IE instrumentation.