

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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0 1  
7 8

REPORT SOURCE L 6 0 5 0 - 0 3 4 6 7 0 6 0 1 8 1 8 0 7 0 2 8 1 9  
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

7 8 9

0 9 7 8

SYSTEM CODE: C C (11) 9 10

CAUSE CODE: X (12) 11

CAUSE SUBCODE: Z (13) 12

COMPONENT CODE: V A L V E X (14) 13 18

COMP. SUBCODE: P (15) 19

VALVE SUBCODE: B (16) 20

EVENT YEAR: 8 1 (17) 21 22

SEQUENTIAL REPORT NO.: 0 3 4 (18) 23 26

OCCURRENCE CODE: 0 3 (19) 28 29

REPORT TYPE: L (20) 30

REVISION NO.: 0 (21) 32

ACTION TAKEN: X (22) 33

FUTURE ACTION: X (23) 34

EFFECT ON PLANT: Z (24) 35

SHUTDOWN METHOD: Z (25) 36

HOURS: 0 0 0 0 (26) 37 40

ATTACHMENT SUBMITTED: Y (27) 41

NPRO-4 FORM SUB.: N (28) 42

PRIME COMP. SUPPLIER: N (29) 43

COMPONENT MANUFACTURER: D 2 4 3 (30) 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (31) 48

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 4 80

7 8 9 10  
PUBLICITY  
ISSUED DESCRIPTION (45)  
2 0 N (44) NA  
8107140729 810702  
PDR ADOCK 05000346  
S PDR  
NRC USE ONLY  
68 69  
200-250-5000 Ext. 22

B107140729 B10702  
PDR ADCK 05000346  
S PDR

NRC USE ONLY

PHONE (419) 259-5000, Ext. 225

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TOLEDO EDISON COMPANY  
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-33-81-39

DATE OF EVENT: June 1, 1981

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: One code safety valve on Main Steam Line 1 declared inoperable

Conditions Prior to Occurrence: The unit was in Mode 1 with Power (MWT) = 2018 and Load (Gross MWE) = 672

Description of Occurrence: On May 31, 1981 at 0300 hours while performing Turbine Generator Control Valve Test, PT 5193.01, the station experienced a power swing down to 550 MWe. During the swing, one of the main steam safety valves lifted on the No. 1 side. After reviewing all of the available data from the swing, it was determined that one of the safety valves lifted early. On June 1, 1981 at 1555 hours, one safety valve on Main Steam Line 1 was declared inoperable. The station entered the action statement of Technical Specification 3.7.1.1 which allows power operation to proceed provided that within four hours the high flux trip setpoint be reduced to  $\leq 92.91\%$  full power. As the station was already in a three reactor coolant pump operating condition, the high flux trip setpoint had previously been reduced to 80% full power. The conditions of the action statement were being met.

Designation of Apparent Cause of Occurrence: The cause for the apparent early lifting is unknown at this time. Testing and examination of some main steam line safety valves will be done during the current Reactor Coolant Pump seal repair outage.

Analysis of Occurrence: There was no danger to the health and safety of the public or to station personnel. The power swing was not severe and was controlled without difficulty after suspending turbine control valve testing. The safety valve apparently lifted only 4% lower than its setpoint. The valve would still have served its intended function of relieving pressure had a trip occurred.

Corrective Action: The setpoints for some safety valves will be checked during the next outage. Any problems found at that time will be evaluated and corrective action taken.

Failure Data: Previous safety setpoint problems were reported in Licensee Event Reports NP-33-79-51 (79-049), NP-33-79-34 (79-032), NP-33-79-25 (79-020), NP-33-79-23 (79-018) and NP-33-78-145 (78-124).