

Washington Public Power Supply System

P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000
Docket No. 50-397 June 29, 1981
G02-81-162



Mr. R. H. Engelken
U. S. Nuclear Regulatory Commission
Region V
Suite 202, Walnut Creek Plaza
1990 North California Boulevard
Walnut Creek, California 94596

Dear Mr. Engelken:

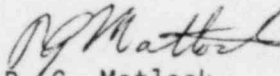
Subject: SUPPLY SYSTEM NUCLEAR PROJECT NO. 2
REPORTABLE DEFICIENCY - 10CFR50.55(e) #156
POTENTIAL REPORTABLE CONDITION ON RIGID
SWAY STRUT PADDLES

Your office was informed by phone on May 28, 1981 of a potentially reportable deficiency, regarding the failure in testing below the required proof load of the rigid sway strut paddles.

We have included an interim report on this matter and will forward the final report by September 18, 1981.

If you have any questions, please contact R. E. Spence, (509) 377-2501, extension 2721.

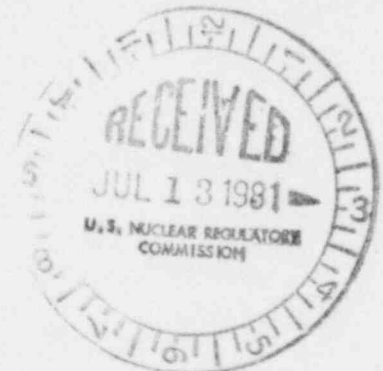
Very truly yours,


R. G. Matlock
Program Director, WNP-2

RGM/RES/kd

Attachment: Interim Report

cc: W.S. Chin, BPA
N.D. Lewis, NRC
T.A. Mangelsdorf, Bechtel
R.C. Root, B/R Site
RE Snaith, B/R NY
V. Stello, NRC
AD Toth, NRC Resident Inspector, WNP-2
E. Wood, NRC
WNP-2 Files



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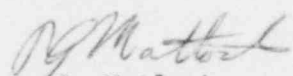
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WASHINGTON PUBLIC POWER SUPPLY SYSTEM

DOCKET NO. 50-397

LICENSE NO. CPPR-93

POTENTIALLY REPORTABLE DEFICIENCY #156
ON RIGID SWAY STRUT PADDLES

SUPPLY SYSTEM NUCLEAR PROJECT NO. 2

INTERIM REPORT

1. POTENTIAL PROBLEM:

Results of pull tests have indicated that a certain type of rigid sway strut paddles is not capable of achieving the proof load required.

Failure of these rigid sway strut paddles would render the sway brace ineffective and could result in piping system failure. This could adversely affect the safe operation of the plant.

2. APPROACH TO RESOLUTION OF THE PROBLEM:

Site Engineering has performed a preliminary review of plant sway strut installations.

3. STATUS OF PROPOSED RESOLUTION:

Three options have been presented by site Engineering as possible solutions:

1. Repair or replacement - a replacement paddle that was not drilled for a setscrew would be acceptable. Repair of the existing paddles with welding could be performed pending the manufacturer's approval. Staking of the spherical bearing would be substituted for the setscrew.
2. Acceptance by load derating - a testing program would be performed to establish a derated allowable.
3. Acceptance by reduction of safety factor - submittal for ASME code committee review could result in acceptance through a code variance.

4. REASON FOR DELAY OF FINAL REPORT:

Review of procurement records indicate that most of the suspect items were procured from Nuclear Power Service Industries (NPSI). NPSI has

been appraised of the situation and stated they would review their design documentation and make a response.

5. PROJECTED COMPLETION OF CORRECTIVE ACTION AND SUBMITTAL OF FINAL REPORT.

When the response has been received from NPSI, site Engineering will develop a corrective action program. A final report detailing our corrective action program will be forwarded to your offices by September 18, 1991.