



Wisconsin Electric POWER COMPANY
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July 6, 1981

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. NUCLEAR REGULATORY COMMISSION
Washington, D. C. 20555

Attention: Mr. R. A. Clark, Chief
Operating Reactors Branch 3

Gentlemen:



DOCKET NOS. 50-266 AND 50-301
NUREG-0737 SCHEDULE UPDATE
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

This letter provides additional information concerning the schedule of commitments related to the requirements of NUREG-0737, "Clarification of TMI Action Plan Requirements", for the Point Beach Nuclear Plant, Units 1 and 2.

As discussed with Messrs. Clark and Colburn on July 1, 1981, Item II.F.1, "Additional Accident-Monitoring Instrumentation", needs further clarification. Wisconsin Electric will provide the design details for the sampling and analysis system for Point Beach by September 1, 1981. This design will provide a system which meets the requirements for an isokinetic and representative sample within the limits of equipment capability and location with respect to survivability and personnel access.

Additionally, Wisconsin Electric is in the process of examining the remaining NUREG-0737 items with regard to the present status of our implementation schedule. By September 1, 1981, we will include an update of the incomplete items listed in the Schedule Table and Notes attached to our March 31, 1981 submittal. This September 1, 1981 submittal will include the design information requested in NUREG-0737 regarding the Reactor Coolant System Vents (II.B.1) and Instrumentation for Detection of Inadequate Core Cooling (II.F.2). This information is not presently available from the system vendor or designer and, thus, cannot be provided at this time.

It should be recognized that the requirements imposed by IE Bulletin 79-01B have impacted a number of instrumentation systems in the Plant, including additional qualification requirements

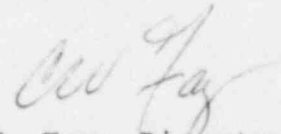
Mr. Harold R. Denton

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for new equipment ordered for NUREG-0737 systems. This Bulletin 79-01B impact has been adverse with respect to equipment delivery schedules and supplier willingness to provide qualification data. Since the systems affected by Bulletin 79-01B are either included as part of the NUREG-0737 items or are being implemented in parallel with NUREG-0737 additions, a delay in equipment deliveries may be incurred. The addition of Regulatory Guide 1.97 requirements into the NRC-required changes will also delay implementation schedules since items must be sequenced to replace or upgrade operating systems in an existing plant. The specific details of these schedular impacts will be provided in our September 1, 1981 submittal.

Very truly yours,



C. W. Fay, Director
Nuclear Power Department

Copy to: NRC Resident Inspector