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May 4, 1981



Mr. James G. Keppler, Director
Directorate of Inspection and
Enforcement - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Subject: LaSalle County Station Units 1 and 2
Suppression Pool Water Volume
Final Report - 10 CFR 50.55(e)
NRC Docket Nos. 50-373/374

Reference (1): L. O. DelGeorge letter to J. G. Keppler
dated December 24, 1980.

Dear Mr. Keppler:

The purpose of this letter is to supplement the information reported in Reference (1) relative to the LaSalle County Station suppression pool water volume. Subsequent to the submittal of the preliminary report made in Reference (1), the Final Safety Analysis Report was amended (Amendment 54) to correct the documented pool volume to reflect the as-built situation. Reanalysis to verify the adequacy of the volume reduction (i.e., from 142,000 cubic feet originally reported to 128,000 cubic feet as-built) was not necessary inasmuch as the original analysis done to demonstrate adequate water volume to provide the required pressure suppression response was done assuming 117,000 cubic feet of water.

However, the initial analysis of long term pool cooling capability assumed 142,000 cubic feet of water. Therefore, a reanalysis was initiated at General Electric (GE) and a confirmatory analysis was begun by the architect engineer - Sargent & Lundy (S&L). These two analyses, which are now complete, were performed using different computer codes; GE used the code Super HEX and S&L used their own proprietary code SUPTRAN. The solutions agree within 4 percent and identified identical maximum pool temperatures for the limiting case.

The results indicate that for the Mark II Owner's White Paper Case 3, which is a scram with immediate isolation followed by initiation of the RHR Heat Exchanger at the 10 minute point, the maximum temperature is 187°F. Other cases give results from 170 to 177°F using the as-built pool water volume. It has been shown that the 10 percent change in suppression pool water volume did not cause a marked difference in pool temperature response because the

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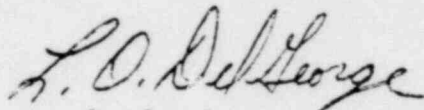
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actual pool thermal capacity is sufficient to absorb the heat inputs prior to the time when the massive heat removal capacity of the RHR heat exchanger comes into effect. This confirmation of thermal adequacy of the as-built LSCS suppression pool water volume is sufficient justification to close this 10 CFR 50.55(e) report.

Formal documentation of the updated pool temperature transient analysis will be made in May, 1981 as part of Chapter 6 of the LaSalle County Station Design Assessment Report. However, as was previously indicated, it is judged that this summary report of the analysis report for the most severe case to be analyzed is sufficient to close this issue. No further reporting under 10 CFR 50.55(e) on this issue will be made.

In the event you have any questions in this regard, please direct them to this office.

Very truly yours,



L. O. DeiGeorge
Nuclear Licensing Administrator

cc: Director of Inspection
and Enforcement
Washington, DC 20555 ✓

NRC Resident Inspector - LaSalle