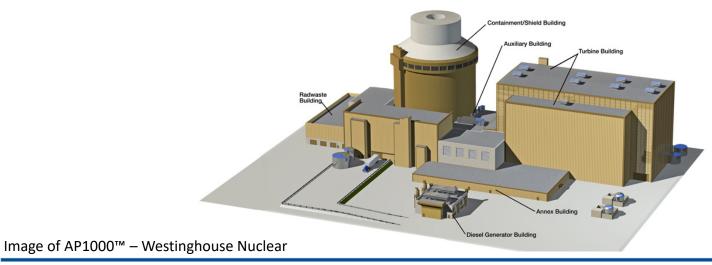
## Reactor Oversight Process (ROP) for Inspecting Vogtle Site with AP1000 Design

January 14 and 22, 2020 Public Meetings

## The AP1000 passive design provides overall safety enhancements.

- Passive versus Active Components
  - AC is not required for safe shutdown
  - Elimination of safety-related motor operated valves
  - Elimination of AC powered safety-related pumps
- Simpler Design no need for human interaction for the first 72 hours
- Uses natural forces such as gravity, convection and condensation cooling

#### The Westinghouse AP1000 Plant



The above illustration is an artist rendering and may not depict actual design and layout



# Simpler design results in the need for an inspection sample range reassessment.

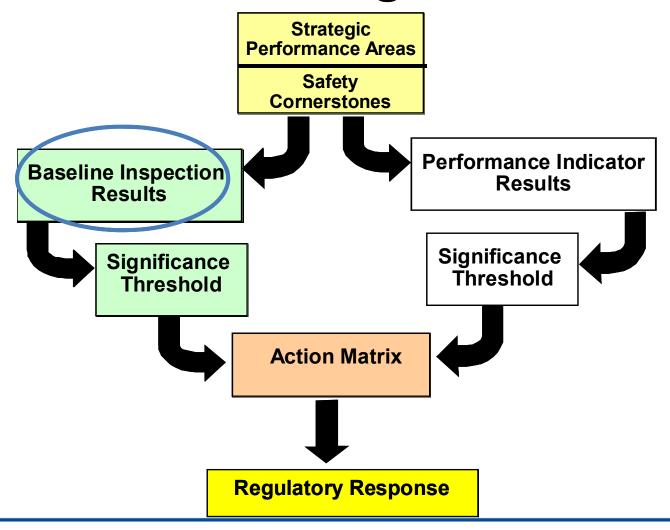
- 50% fewer safety-related valves
- 35% fewer pumps
- 80% less safety related piping
- 85% less control cable
- 45% less seismic building volume

### NRC Performance Goals

 Safety: Ensure adequate protection of public health and safety and the environment.

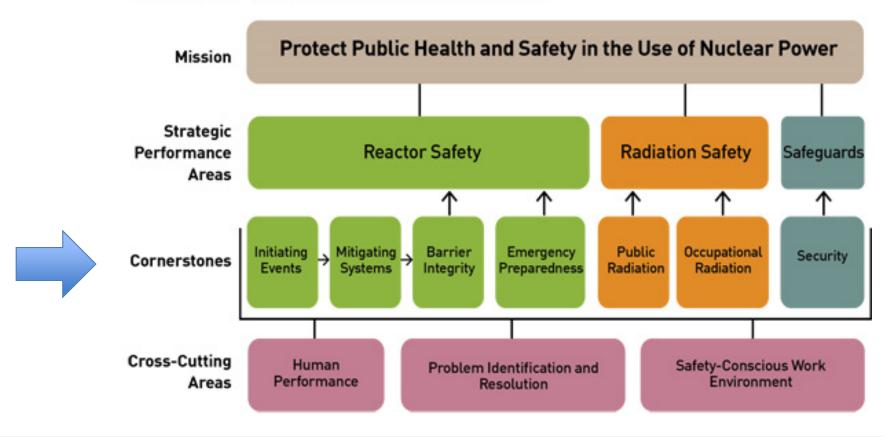
 Security: Ensure adequate protection in the secure use and management of radioactive materials.

### Reactor Oversight Process



# The Cornerstones have supporting inspections.

#### Reactor Oversight Framework



## Inspection oversight for Units 3 and 4 considers current inspection at Units 1 and 2

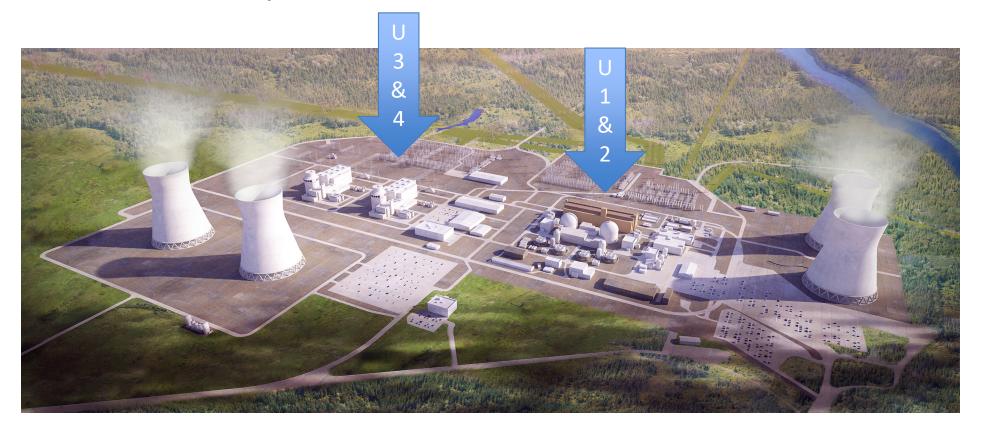


Image of Vogtle 1-4, Waynesboro Georgia – Georgia Power, A Southern Company

# NRC used a three step process to determine proposed sample sizes.

 Began with the current Minimum, Nominal, and Maximum sample sizes and existing guidance for sample size adjustment for conventional plants in all procedures.

**Current Sample** 

#### **New Technology**

- Assess unique aspects of the passive technology
- Overall component numbers and accessibility.

 Assess for common programs across the site due to colocation with conventional plant.

Evaluate Common Programs

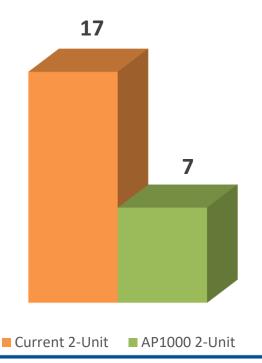
## Inspection procedures are either individual or site wide, and samples were addressed accordingly.

Procedure Category	Sample Size Analysis
Individual Reactor Safety Baseline Inspection Procedures (BIPs)	Staff used technology specific component analysis and risk information inherent from the design of the two AP1000 units to develop sample size ranges in accordance with Inspection Manual Chapter 2515
Common Site-Wide Program BIPs	Staff evaluated and adjusted, if necessary, the maximum sample size to provide an adequate number of samples across the two different designs

### Example of changes to BIPs which are conducted on 2-Unit PWR and 2-Unit AP1000 independently.

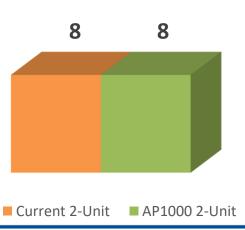
IP 71111.22 Surveillance Testing – Design Specific

**BIP Nominal Samples** 



### IP 71111.11 Licensed Operator Requalification and Performance – Design Independent

**BIP Nominal Samples** 



## Example of BIPs which are conducted as 4 unit site: Vogtle Site Common Engineering Inspections.

### 71111.21N.02 Design Basis Capability of Power Operated Valves

- Current BIP sample range is 8-12 for existing units
- Staff assessment of AP1000 identified BIP sample range of 3-5
- Increase BIP sample
   maximum to 16 samples for
   Vogtle site with samples
   from all 4 units.

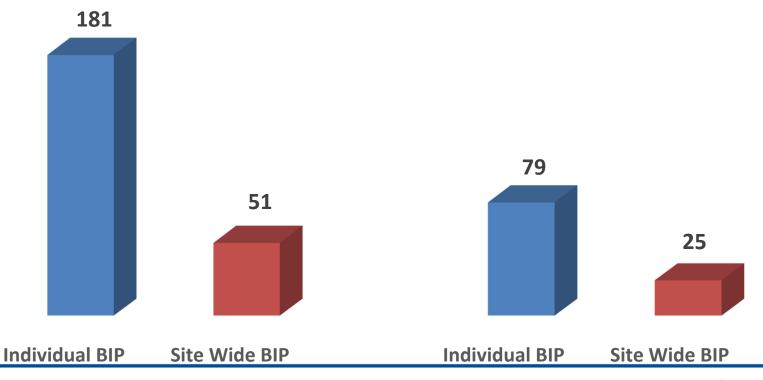
#### 71111.21M Design Basis Assurance

- Will require separate samples from Units 1&2 and Units 3&4 but existing maximum sample size is sufficient to account for all units.
- Vogtle inspection at maximum sample size.
- No change to baseline inspection.

The staff has identified the number of samples for U3&4 based on the number of AP1000 components, risk characteristics, and co-location on an existing site.

Nominal Inspection Samples for Units 1&2

Additional Nominal Inspection Samples for Units 3&4



Subject to additional NRC evaluation of Vogtle Site organizational structure for the applicable programs.



The NRC review identified appropriate modifications to the baseline inspection program for the Vogtle site.

- The three step process considered current baseline, passive design, and co-location with an operating reactor.
- Appropriate sample sizes were identified for each inspection procedure to establish reasonable assurance of adequate protection.
- NRC identified inspection procedures for individual and site wide implementation.

### **Questions?**

### Reference Sources

- Reactor Oversight Process
  - https://www.nrc.gov/reactors/operating/oversight.html
- Public Electronic Reading Room
  - http://www.nrc.gov/reading-rm.html
- Public Document Room
  - 1-800-397-4209 (Toll Free)