

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE February 12, 1990
COMPLETED BY D. L. Stice
TELEPHONE (402)636-2474

MONTH January 1990

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	466	17	467
2	466	18	467
3	466	19	467
4	467	20	467
5	458	21	467
6	443	22	467
7	443	23	466
8	451	24	465
9	466	25	466
10	467	26	466
11	467	27	466
12	467	28	467
13	467	29	467
14	467	30	467
15	467	31	467
16	467		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT Fort Calhoun Station
DATE February 12, 1990
COMPLETED BY D. L. Stice
TELEPHONE (402)636-2474

OPERATING STATUS

- | | Notes |
|------------------------------------------------------------------------------------------------------------|-------|
| 1. Unit Name: Fort Calhoun Station | |
| 2. Reporting Period: January 1990 | |
| 3. Licensed Thermal Power (MWt): 1500 | |
| 4. Nameplate Rating (Gross MWe): 502 | |
| 5. Design Electrical Rating (Net MWe): 478 | |
| 6. Maximum Dependable Capacity (Gross MWe): 502 | |
| 7. Maximum Dependable Capacity (Net MWe): 478 | |
| 8. If changes occur in Capacity Ratings (Item Numbers 3 through 7) Since Last Report, Give Reasons:
N/A | |
| 9. Power Level to Which Restricted, If Any (Net MWe): N/A | |
| 10. Reasons for Restrictions, If Any: N/A | |

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	744.0	744.0	143,354.0
12. Number of Hours Reactor was Critical	744.0	744.0	111,910.3
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	744.0	744.0	110,749.4
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,107,379.1	1,107,379.1	144,722,490.3
17. Gross Electrical Energy Generated (MWH)	362,196.0	362,196.0	47,572,304.2
18. Net Electrical Energy Generated (MWH)	345,492.6	345,492.6	45,413,045.4
19. Unit Service Factor	100.0	100.0	77.3
20. Unit Availability Factor	100.0	100.0	77.3
21. Unit Capacity Factor (Using MDC Net)	97.1	97.1	68.8
22. Unit Capacity Factor (Using DER Net)	97.1	97.1	67.2
23. Unit Forced Outage Rate	0.0	0.0	2.9
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Refueling Outage estimated to begin February 17, 1990, with a planned duration of 86 days.			
25. If Shut Down at End of Report Period, Estimated Date of Startup: N/A			
26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved	

~ INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending January 1990

1. Scheduled date for next refueling shutdown. February 17, 1990
2. Scheduled date for restart following refueling. May 13, 1990
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?
 - 1) Decrease the total unrodded integrated radial peaking factor from 1.80 to 1.70.
 - 2) Decrease the total unrodded planar radial peaking factor from 1.80 to 1.75.
 - 3) Decrease the PLHGR from 15.22 to 14.4 Kw/ft
 - 4) Change the TM/LP P_{var} equation to reflect a change in the gamma term from -11350 to -11240.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. N/A
 - c. If no such review has taken place, when is it scheduled? N/A
4. Scheduled date(s) for submitting proposed licensing action and support information. Submitted Jan. 1990
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. None
6. The number of fuel assemblies:
 - a) in the core 133 assemblies
 - b) in the spent fuel pool 437 "
 - c) spent fuel pool storage capacity 729 "
 - d) planned spent fuel pool storage capacity May be increased via fuel pin consolidation or dry cask storage
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1994*

*Capability of full core offload of 133 assemblies lost.

Prepared by Ken Hatt Date February 9, 1990

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1990DOCKET NO. 50-285UNIT NAME Fort Calhoun StationDATE February 12, 1990COMPLETED BY D. L. SticeTELEPHONE (402) 636-2474

No.	Date	Type (1)	Duration (Hours)	Reason (2)	Method of Shutting Down Reactor (3)	Licensee Event Report #	System Code (4)	Component Code (5)	Cause & Corrective Action to Prevent Recurrence
									There were no unit shutdowns or significant reductions in power during the month of January, 1990.

1
F-Forced
S-Scheduled2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other (Explain)3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)5
Exhibit 1 - Same Source

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

January 1990
Monthly Operating Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power throughout the month of January, 1990, except for a period from January 5 through January 8. During this period power was reduced to 95% to perform required moderator temperature coefficient surveillance testing.

The diesel fire pump discharge piping replacement was completed and the fire pump was returned to service. All four raw water pump discharge check valves were replaced.

A design problem was identified where the containment spray pump suction piping is rated for a lower pressure than the shutdown cooling system. Previous analysis for a worst case fire took credit for a containment spray pump used in an alternate shutdown cooling alignment to achieve and maintain cold shutdown. The design problem invalidates this analysis. Additional compensatory measures have been implemented until procedures and repair parts are in place to restore a low pressure safety injection pump to operable status following a worst case fire.

The control room toxic gas monitors have been inoperable due to modification work on the ventilation system. As a result the control room has been in the recirculation mode for an extended time period. Limited sampling capability by the toxic gas monitors was restored, and, with additional administrative controls in place, intermittent fresh air make up to the control room is now permitted.

Modification work continues on the instrument air dryer and security system. A modification was started to install air dryers on the emergency diesel generator starting air system. Work continues for the third Auxiliary Feedwater Pump modification.

Construction continues on the Chemistry and Radiation Protection and Rad Waste Buildings.

The following NRC inspections took place in January, 1990:

IR 89-50	Resident Inspectors' Monthly Inspection (continued thru January 15, 1990)
IR 90-01	Regulatory Guide 1.97 Inspection
IR 90-02	Resident Inspectors' Monthly Inspection (from January 16, 1990)
IR 90-03	Surveillance Program Inspection
IR 90-05	Inservice Testing Program

The following LER's were submitted:

Date Submitted

89-023	Failure to Comply with High Radiation Area Access Requirements	January 10, 1990
89-S10	Discovery of Shotgun Shell in Protected Area	January 03, 1990

A. SAFETY VALVES OR PORV CHALLENGE: & FAILURES WHICH OCCURRED

None

B. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

<u>Procedure</u>	<u>Description</u>
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None

System Acceptance Committee Packages for January 1990:

<u>Package</u>	<u>Description/Analysis</u>
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MR-FC-86-59	American Technology Corporation (ATCOR) System Removal
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Description:

This modification provided for removal of the ATCOR waste processing system which was nonfunctional.

Safety Analysis:

The removal of the unused ATCOR waste processing system did not increase the probability or consequences of an accident, nor did it reduce the margin of safety as defined in the basis for any Technical Specification. The removal of the ATCOR system did not damage any Critical Quality Element (CQE). Therefore, this modification does not have an adverse effect on the safety analysis.

MR-FC-87-018	Relocation of Underground Fire Loop
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Description:

This modification provided for the relocation and expansion of the underground fire loop for building expansion at Fort Calhoun Station.

Safety Analysis:

Modification of the Underground Fire Loop was carried out beyond the confines of the main plant and there's no safety related equipment located in or directly adjacent to the construction area. Modification of this Fire Protection System maintained compliance with the National Fire Protection Association (NFPA) codes. No change or impact on the existing power plant systems were afforded by this modification. The relocation of the Underground Fire Loop did not increase the probability or consequences of an accident, and the margin of safety was maintained since no changes to the existing power plant systems were performed. This modification, therefore, does not have an adverse effect on the safety analysis.

MR-FC-87-026 Replacement of the P250 Computer

Description:

This modification provided for the transfer of computer points from the P250 computer to the Emergency Response Facility (ERF) computer.

Safety Analysis:

Transfer of computer points from the P250 computer to the ERF computer doesn't increase the consequences of an accident. This modification doesn't reduce the margin of safety as defined in the basis for any Technical Specification since the CQE portion of the modification is a functionally equivalent equipment replacement with no design basis change. No increase in the probability of occurrence of an accident exists. This modification, therefore, does not have an adverse effect on the safety analysis.

MR-FC-88-063 Control Room Heating, Ventilating, and Air Conditioning (HVAC) Prep Work

Description:

This modification provided for the beginning of pre-outage work on the control room HVAC modification MR-FC-81-51. This removed non-seismic, non-boundary walls, rest room, kitchenette, the ceiling of the control room and removed VA-171 air conditioning to the computer room.

Safety Analysis:

This modification consisted of architectural changes being made to the Control Room and removal of auxiliary steam piping. The safety function of any CQE equipment was not impacted. Control room habitability was not compromised and no other new accident possibilities were created. Control room envelope integrity wasn't breached so habitability will be maintained through radiological or toxic gas release. Toxic Gas Monitor and Control Room Ventilation operability Technical Specification requirements were observed. Installation didn't impact the operability of either so the margin of safety as defined in the basis for any Technical Specification was not reduced. This modification did not increase the probability of occurrence or consequences of an accident. This modification, therefore, does not have an adverse effect on the safety analysis.

MR-FC-89-047 Electrical Service for Station Engineering Trailers

Description:

This modification provided for installing a new power panel MPP-037 and utilized existing power panel MPP-039 to provide electrical power service to the station engineering trailers. This modification also provided gal-tronics and communications to the station engineers' trailers.

Safety Analysis:

The installation of this modification had no affect on plant equipment and did not create the possibility of degrading CQE equipment. Nuclear safety was not affected since no CQE components were located within the construction area. The provision of electrical power, gal-tronics and communications to the station engineers trailers did not increase the probability or consequences of an accident. This modification, therefore, does not have an adverse effect on the safety analysis.

C. RESULTS OF LEAK RATE TESTS

The reactor coolant system leak rate test, ST-RLT-3 F.1, indicates for the month of January a continuation of the elevated leak rates seen throughout Cycle 12. The actual leak rate tests gave varying results due to changes in the condition of the operating and standby charging pumps.

The maximum leak rate for the month was recorded on January 27, 1990. This test registered a total leak rate of 0.691 gpm and an unknown leak rate of 0.229 gpm. The minimum leak rate for the month was recorded on January 9, 1990. At this time, the total leak rate was 0.473 gpm and the unknown leak rate was -0.002 gpm. There was no evidence of primary to secondary leaks into the steam generators.

Plans have been formulated by the System Engineering Department to attempt to correct the leaks from the Reactor Coolant System.

D. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.

Description

Amendment No. 123

This amendment modifies Fort Calhoun Technical Specification Section 3.1, Table 3-3 in order to correct an administrative error that was made when Amendment No. 32 to the Technical Specifications was issued. The changes requested pertain to surveillance testing of the Interlocks-Isolation Valves on Shutdown Cooling Line.

<u>Amendment No.</u>	<u>Description</u>
Amendment No. 124	This amendment adds an operating mode number for each operating mode in Table 3-4 of the Technical Specifications. In addition, in the Refueling Shutdown mode, when fuel is in the core, a water sampling and analysis is required. However, when the total core has been off-loaded, this amendment incorporates that this water testing and analysis is not required since there is no fuel in the core.

II. MAINTENANCE (Significant Safety Related)

See Attachment

Gary R. Peterson
Manager-Fort Calhoun Station

CLOSED CQE MWO'S COMPLETED DURING JANUARY 1990

MWO #	SYSTEM	EQUIP ID	CQE	CLASS	COMPLETE	WORK DESCRIPTION/PERFORMED
893912	DG	DG-1	Y	N	01/16/90	INSULATE DIESEL GEN. EXHAUST LINE WITH CAL SIL INSULATION. INSULATED AND METAL LAGGED DG-1 EXHAUST AND MUFFLER.
894467	SA	SA-193	Y	3	01/19/90	REPAIR AIR START REGULATOR ON D-2 WHICH IS LEAKING AIR. ISOLATED REGULATOR AND REMOVED PROCESS GAUGE. INSTALLED TEST GAUGE, REPRESSURIZED REGULATOR AND ADJUSTED REGULATOR TO 180 PSI. REINSTALLED PROCESS GAUGE. PRESSURIZED SYSTEM AND SCOOPED FOR LEAKS. (FOUND REGULATOR SET AT APPROXIMATELY EQUAL TO 295 PSI)
894503	TM	YIT-6288A	Y	S	01/12/90	MONITOR IS TEARING TAPE, TROUBLE SHOOT AND REPAIR AS REQUIRED ADJUSTED HEAD TENSION SPRING TO ATTAIN PROPER VACUUM AND NO TEAR TAPES.
894641	TM	TM	Y	S	01/12/90	REMOVE FROM SERVICE/REINSTALL FOUR (4) TOXIC GAS MONITORS TO BE RETURNED TO MDA SCIENTIFIC INC. FOR THE SCHEDULED EIGHTEEN MONTH CALIBRATION TESTING. MONITORS ARE: 2-EA, CL2, YIT-6288A/B AND HYDROGEN FLUORIDE YIT-6286A/B. MWO USED TO COVER WORK/SHIPPING/INSTALLATION OF MONITORS. PULLED FUSES FOR YIT-6286A/B AND YIT-6288A/B. REMOVED MONITORS FROM HOFFMAN BOXES. PLACED IN SHIPPING CRATES AND DELIVERED TO WAREHOUSE (DKC 9/26/89). PER DETAILED WORK INSTRUCTIONS. BENCH CHECKED YIT-6288A, YIT-6286A AND B AND INSTALLED PER ATTACHED DETAILED WORK INSTRUCTIONS. VERIFIED ALARM ON BENCH TO WORK. YIT-6288A MWO 895691 INITIATED TO REPAIR. (12/07/89 PD) MWO 895691 COMPLETED. UNIT INSTALLED AS PER DETAILED WORK INSTRUCTIONS (YIT-6288A) PD.
894803	EE	EE	Y	N	01/05/90	INSPECT ALL CABLE TRAYS IN CORRIDOR #4 IN THE AUX BUILDING TO IDENTIFY ANY DAMAGE, LOOSE BOLTS OR DIVIDERS, TRASH ETC. AND DOCUMENT INSPECTION FINDINGS. INSPECTED CABLE TRAYS IN CORRIDOR 4 IN THE AUX BUILDING. IDENTIFIED ANY DAMAGE. LOOSE BOLTS OR DIVIDERS, TRASH ETC. AND DOCUMENTED INSPECTION FINDINGS.
894824	AC-RW	AC-10A	Y	3	01/22/90	RE-ADJUST IMPELLER LIFT. RECORD AS FOUND AND AS LEFT READINGS. ADJUSTED LIFT AS PER MM-RR-RW-0001 (MP-AC-10) AS FOUND .079" LIFT TOTAL LIFT .199" AS LEFT .015 LIFT.
895293	AC-RW	HCV-2883B	Y	N	01/12/90	REGULATOR DRIPWELL DRAIN VALVE IS PLUGGED, REMOVE REGULATOR, CLEAN AND REFURBISH AIR REGULATOR. READJUST AIR PRESSURE TO PROPER SETTING AFTER CLEANING. FOUND INTERNALS OF REGULATOR CORRODED, REPLACED REGULATOR AND GAUGE, SET TO 30 PSI, SUPPLY TO OPERATOR. NEW DRAIN VALVE WAS INCLUDED WITH NEW REGULATOR.
895691	TM	YIT-6288A	Y	S	01/12/90	REPAIR TOXIC GAS MONITOR-CHLORINE GAS, TEST SWITCH ENGAGED GIVES ERRATIC METER READINGS AND THE OPTIC LIGHT FLICKERS. BENCH CHECKED YIT-6288A AND WITH AN OSCILLOSCOPE, OBSERVED EXCESSIVE RINGING AT U1. CHANGED OUT ELECTRONICS MODULE AND OPTICS LAMPS. ADJUSTED UNIT FOR 4-20MA OUTPUT. RINGING WAS GREATLY REDUCED.
897598	A1-ESF	43-3/1AD1	Y	N	01/12/90	WHEN BREAKER PROTECTION MODE SWITCH IS TAKEN TO OFF LOWER-LEFT BULB IN DIESEL AUTO STANDBY WINDOW FLASHES AND AUDIBLE ALARM WOULD NOT ACKNOWLEDGED. SWITCH HANDLE IS LOOSE. REPLACED ANNUNCIATION CARD 70-ARLN IN ANN PANEL A-30-C1. THE BRASS KEY HANDLE IS LOOSE ON SWITCH SHAFT. THIS DOES NOT EFFECT THE OPERATION OF THE SWITCH. THIS WILL BE CORRECTED DURING THE 90 OUTAGE.

OPPD FORT CALHOUN PLANT
CHAMPS REPORT M07

COMPLETED CQE MWO'S - JANUARY (WFFRMMH1)

DATE: 02/07/90 PAGE: 1
TIME: 16:55

MWO # SYSTEM
EQUIP ID

LEAD PRINT.
DISC STATUS

883739 AC-SFP VALVE HAS A PACKING LEAK WHEN SHUT.

PE C

AC-186

CQE: V CLASS: 3

WORK PERFORMED
REPACK VALVE AC-186 AND REPLACED THE O-RING AS PER PE-RR-VX-04285 TORQUE
TO PE-RR-VX-04285
REPLACE COVER AS QC CAGE TESTING AREA KEG 1-25-90

COMPLETE: 01/25/90

888081 RC
RC-3C
WORK DESCRIPTION
PERFORM PRESSURE BREAKDOWN TEST ON THE SEAL REMOVED FROM RC-3C PER
MP-RC-3-3-C.
1990 OUTAGE PREREQUISITE

MM C

CQE: V CLASS: 1

WORK PERFORMED
PERFORMED BREAKDOWN TEST PER ATTACHED PROCEDURE MM-RR-RC-0007. THIS
PROCEDURE SUPERCEDES MP-RC-3-3-C. WORK WAS PERFORMED WITH NO PROBLEMS
ALL BREAKDOWN'S FLOWS WERE ACCEPTABLE.

COMPLETE: 01/19/90

888092 RC
RC-3D
WORK DESCRIPTION
PERFORM PRESSURE BREAKDOWN TEST ON THE SEAL REMOVED FROM RC-3D PER
MP-RC-3-3-C.
1990 OUTAGE PREREQUISITE

MM C

CQE: V CLASS: 1

WORK PERFORMED
PERFORMED BREAKDOWN TEST PER ATTACHED PROCEDURE MM-RR-RC-0007. THIS
PROCEDURE SUPERCEDES MP-RC-3-3-C. THE FLOW ON THE MIDDLE PRESSURE
BREAKDOWN RC-25088 DID NOT MEET THE ACCEPTABLE CRITERIA ON TABLE 1.
THE UPPER BREAKDOWN RC-24734 AND THE LOWER BREAKDOWN RC-24724 WERE
ACCEPTABLE PER TABLE 1.

COMPLETE: 01/19/90

889180 SI-CS
SI-3A
WORK DESCRIPTION
INLET AND OUTLET FLANGES HAVE EIGHT NUTS WHICH DO NOT MEET MINIMUM THREE
D ENGAGEMENT. TIGHTEN BOLTS TO ENSURE ALL NUTS MEET MINIMUM THREAD ENGAG
EMENT.

PE C

CQE: V CLASS: 2

WORK PERFORMED
PERFORM AS REQUIRED BY WORK INSTRUCTIONS. ON DISCHARGE FLANGE 10 OF TE 1
2 STUDS WERE CHANGED. TWO STUDS HAD ON BOTH END 1 / 2 THREAD ENGAGEMENT

COMPLETE: 01/15/90

OPPD FORT CALHOUN PLANT
CHAMPS REPORT M07

COMPLETED CQE MWO'S - JANUARY (WFFRMMH1)

DATE: 02/07/90 PAGE: 1
TIME: 16:55

MWO # SYSTEM
EQUIP ID

LEAD PRINT
DISC STATUS

894055 AC-SFP
AC-191
WORK DESCRIPTION
ISOLATION VALVE NEEDS REBUILT, IT LEAKS BY THE SEAT.

PE C

CQE: Y CLASS: 3
COMPLETE: 01/24/90
WORK PERFORMED
ISOLATED AND DRAINED SYSTEM. REMOVED VALVE FROM SYSTEM AND INSTALLED NEW SEAL RING, NEW O-RING AND NEW PACKING. PACKING FOLLOWER WAS CRACKED DURING REPACKING WHEN THE FOLLOWER WAS TORQUED. FABRICATED AND INSTALLED A 1/4" PLATE BETWEEN THE PACKING FOLLOWER AND THE VALVE BODY SEE ATTACHED TEMPORARY MODIFICATION NUMBER 90-M-02. DRILLED NEW HOLE INTO THE VALVE STEM, TO ADAPT THE VALVE HANDLE TO THE 1/4" SPACER PLATE. INSTALLED VALVE

894056 AC-SFP
AC-192
WORK DESCRIPTION
ISOLATION VALVE NEEDS REBUILT, IT LEAKS BY THE SEAT.

PE C

CQE: Y CLASS: 3
COMPLETE: 01/24/90
WORK PERFORMED
ISOLATED AND DRAINED SYSTEM. REMOVED VALVE FROM SYSTEM AND INSTALLED NEW SEAL RING, NEW O-RING AND NEW PACKING. PACKING FOLLOWER WAS CRACKED DURING REPACKING WHEN THE FOLLOWER WAS TORQUED. FABRICATED AND INSTALLED A 1/4" SPACER PLATE BETWEEN THE PACKING FOLLOWER AND THE VALVE BODY TEMPORARY MODIFICATION # 90-M-02. DRILLED NEW HOLE INTO THE VALVE STEM, TO ADAPT THE VALVE HANDLE TO THE 1/4" SPACER PLATE. INSTALLED VALVE BACK IN SYSTEM

894057 AC-SFP
AC-199
WORK DESCRIPTION
DISCHARGE ISOLATION VALVE NEEDS REBUILT, IT LEAKS BY THE SEAT.

PE C

CQE: Y CLASS: 3
COMPLETE: 01/24/90
WORK PERFORMED
REMOVED SOME ROUND BAR FROM THE VERY HIGH ROD CAGE TO ENABLE FOR THE REMOVAL OF THE VALVE. ISOLATED SYSTEM AND REMOVED VALVE FOR REPAIRS. INSTALLED NEW SEAL RING, NEW O-RING AND NEW PACKING. INSTALLED VALVE BACK IN SYSTEM USING NEW 8" FLEX GASKET AND NEW STUDS AND NUTS. TORQUED PER PROCEDURE. ALL WORK WAS PERFORMED USING ATTACHED PE-RR-VX-04285 PROCEDURE. RETURNED SYSTEM BACK TO NORMAL. *PER TONY CHRISTENSEN (RADIOLOGY OPERATOR)

MWO # SYSTEM
EQUIP ID

LEAD PRINT
DISC STATUS

894059 AC-SFP AC-5B
WORK DESCRIPTION
GAUGE LINE OFF OF AC-5B DISCHARGE FLANGE NEEDS REMOVED, CLEANED AND RESEALED.
PE C

CQE: Y CLASS: 3
COMPLETE: 01/24/90
WORK PERFORMED
REMOVED GAUGE LINE CLEANED THREADS AND REDOPED WITH CQE LOCTITE P.S.T. RETURNED FITTINGS TO ORIGINAL CONFIGURATION AND RETURNED TO SERVICE.

894060 AC-SFP AC-193
WORK DESCRIPTION
ISOLATION VALVE NEEDS REBUILT, IT LEAKS BY THE SEAT.
PE C

CQE: Y CLASS: 3
COMPLETE: 01/22/90
WORK PERFORMED
ISOLATED AND DRAINED DOWN SYSTEM. REMOVED VALVE FROM SYSTEM AND INSTALLED NEW SEAL RING, NEW O-RING, NEW PACKING. INSTALLED VALVE BACK IN SYSTEM USING NEW 8" FLEX GASKET AND TORQUED PER PRC PROCEDURE. ALL WORK WAS PERFORMED USING ATTACHED PE-RR-VX-04285 MAINTENANCE PROCEDURE. RETURNED SYSTEM BACK TO NORMAL.

894061 AC-SFP AC-188
WORK DESCRIPTION
ISOLATION VALVE NEEDS REBUILT, IT LEAKS BY THE SEAT.
PE C

CQE: Y CLASS: 3
COMPLETE: 01/22/90
WORK PERFORMED
REBUILT AC-188 AS PER PE-RR-VX-04285 MAINTENANCE PROCEDURE. ISOLATED AND DRAINED DOWN SYSTEM. REMOVED VALVE FROM SYSTEM AND INSTALLED NEW SEAL RING, NEW O-RING, NEW PACKING. INSTALLED VALVE BACK IN SYSTEM USING NEW 8" FLEX GASKET AND TORQUED PER PRC PROCEDURE. ALL WORK WAS PERFORMED USING ATTACHED PE-RR-VX-04285 PROCEDURE. CG 1-22-90 RETURNED SYSTEM BACK TO NORMAL.

894453 AC-RW RW-125
WORK DESCRIPTION
THE INTERNALS OF RW-125 ARE BADLY ERODED. REPLACE RW-125 WITH A NEW VALVE.
PE 3

CQE: Y CLASS: 3
COMPLETE: 01/25/90
WORK PERFORMED
ACQUIRED NECESSARY TOOLS & RIGGING. REMOVED BOLTING & EXISTING VALVE BODY CLEANED & INSPECTED ALL GASKET SURFACES. MADE 2 NEW GASKETS. CLEANED STUDS & NUTS AND HAD Q.C. INSPECT THEM AS PER THE PROCEDURE. LUBRICATED BOLTING WITH APPROVED LUBRICANT. INSTALLED NEW VALVE AND GRAFOIL GASKETS. INSTALLED BOLTING & TORQUED AS PER THE PROCEDURE. PERFORMED VISUAL LEAK CHECK AND FOUND NO LEAKAGE. CLEANED UP WORK AREA AND RETURNED TOO

MWO # SYSTEM
EQUIP ID

LEAD PRINT
DISC STATUS

MWO #	SYSTEM	WORK DESCRIPTION	LEAD	PRINT
EQUIP ID			DISC	STATUS
894523	AC-RW	WORK DESCRIPTION RW-115 HAS BEEN REPORTED LEAKING BY. REPLACE RW-115 WITH A NEW VALVE.	PE	C
RW-115				
CQE: Y	CLASS: 3	WORK PERFORMED LOAD TEST PERFORMED ON 1-23-90 D GASKILL QC# 90-390. REMOVED CHECK VALVE FROM SYSTEM AND FOUND ONE FLAPPER ON CHECK VALVE MISSING AND PULLED RO W WATER PUMP (ON A DIFFERENT MWO) FOUND MISSING FLAPPER INSIDE PUMP. REPLACED OLD CHECK VALVE WITH NEW ONE INSTALL A WITH NEW GASKETS.		
COMPLETE: 01/25/90				
894524	AC-RW	WORK DESCRIPTION RW-117 HAS BEEN REPORTED LEAKING BY. REPLACE RW-117 WITH A NEW VALVE.	PE	3
RW-117				
CQE: Y	CLASS: 3	WORK PERFORMED TAGGED AND REMOVED CHECK VALVE. CLEANED FLANGE SURFACE. INSTALLED PER MWO SIGN OJ TORQUED AND CHECKED FOR LEAKS. TEMPORARY REINSULATED RW-117-WILL REINSULATED WHEN NEW MWO IS ISSUED LATER. T.S 01-26-90 T-1 220 TO 450 LBS. 11-9-90 CALIDATE 1-25-90 T-3 730 LBS. 2-6-90 CALIDATE		
COMPLETE: 01/29/90				
894525	AC-RW	WORK DESCRIPTION RW-121 HAS BEEN REPORTED LEAKING BY. REPLACE RW-121 WITH A NEW VALVE.	PE	3
RW-121				
CQE: Y	CLASS: 3	WORK PERFORMED RW 121 REMOVED CLEANED AND INSTALLED NEW CHECK VALVE CLEANED TAGS INSPECTED FOR LEAKS. TEMPORARY INSULATED RW-121-WILL REINSULATE WHEN NEW MWO IS ISSUED LATER. T.S. 1-26-90 1-25-90 T1 220 TO 450LBS 11-9-90 CAL DATE 7 TORQUE 1-25-90 T-3 730LB 2-6-90 CAL DATE 5 WRENCH'S		
COMPLETE: 01/29/90				

MWO # SYSTEM
EQUIP ID

LEAD PRINT
DISC STATUS

895146	AC-SFP	WORK DESCRIPTION		
		AC-319 APPEARS TO BE PLUGGED.NEED TO UNPLUG,POSSIBLY REPLACE DIAPHRAM.	PE	C
	AC-319	THIS IS A 2 INCH DIAPHRAM VALVE.WE MIGHT BE ABLE TO FREE BLOCKAGE BY		
		PRESSURIZING LINE WITH AIR,CONNECTING AT DW-207.THIS IS A 3/4 INCH VALVE		
		SEE JIM ALLEN.		
		WORK PERFORMED		
	CQE: Y CLASS: 3	TEMP AIR AND WATER LINES ASSEMBLED AND INSTALLED DOWN STREAM OF DW 204 A		
		S PER DRAWING 11-10-89 RC OPS TO PE RFORM FLUSHING OF LINE PER PROCEDURE		
		INSTALLED RELIEF VALVE SET TO 150# AS PER DRAWING. 12-8-89 RC REMOVE AIR		
	COMPLETE: 01/12/90	PRESSURE REGULATOR. INSTALLED SWAGLOCK FILLING UP STREAM OF BALL VALVE O		
		N TEMPOARY SLUICE PIPING (AIR SIDE) FOR ATTACHMENT TO NITROGEN BOTTLE RC		
		1-2-90 SLUICE COMPLETED BY OPERATIO NS. TEMPOARY SLUICE PIPING *PROCEDU		
		WORK DESCRIPTION		
	895851 EE-5	THE RED INDICATING LIGHT FOR FW-34B ON UNIT B01 DOES NOT WORK.NEED TO TR	EM	C
		DOUBLESHOOT TO DETERMINE PROBLEM,CHECK THE BULB AND SOCKET,MAKE REPAIRS A		
	MCC-4A2	S REQUIRED. J.BRAUN/JEV		
		WORK PERFORMED		
	CQE: Y CLASS: N	FOUND RED INDICATION LIGHT BULB BURNT OUT. REPLACED BULB, VERIFIED PROPE		
		R OPERATION.		
	COMPLETE: 01/18/90			

MWO: # SYSTEM
EQUIP ID

LEAD PRINT
DISC STATUS

MWO: #	SYSTEM	WORK DESCRIPTION	LEAD	PRINT
EQUIP ID			DISC	STATUS
900033	AC-SFP	WORK DESCRIPTION VENDOR TO CREATE FREEZE SEAL ON 8" S.S. PIPE SUCTION LINE TO AC-5A&5B THIS WILL ALLOW REBUILDING OF ISOLATION VALVES.	PE	C
AC-SFP				
CQE: Y	CLASS: 3	WORK PERFORMED HAD OPERATIONS CLOSE FLOOD DOORS TO THE SI ROOMS. HAD G.M INSTALL FLOOD BARRIER FOR ROOM 5. HAD VENDOR INSTALL FREEZE SEAL EQUIPMENT AND ESTABLISH A FREEZE SEAL ON THE 8" STAINLESS STEEL PIPE TO ENABLE IN THE REMOVAL OF THE ISOLATION VALVES TO AC-5A AC-5B. ALL WORK WAS PERFORMED AS PER A ATTACHED MD-RR-MX-1002 PROCEDURE. REMOVED FLOOD BARRIER AND SCAFFOLD OUT OF ROOM 1-25-90 KAG.		
COMPLETE:	01/25/90			
900092	AI-RPS	WORK DESCRIPTION REPAIR TRIP INDICATING LIGHT ON RPS CHANNEL A. TRIP UNIT NO. 8. PRESS- URIZER PRESSURE CHANNEL.	IC	3
AI-RPS				
CQE: Y	CLASS: S	REF. SCHMITD WORK PERFORMED REPAIRED SPRING MECHANISM FOR RESET BY RE-SEATING SPRING IN ITS RETAINER . SPRING HAD SLIPPED OFF. PERFORMED CP-RPS-2 FOR THIS CHANNEL.		
COMPLETE:	01/11/90			
900235	DG	WORK DESCRIPTION PERFORM TROUBLESHOOTING ON REGULATOR TO FURTHER INVESTIGATE THE OVERVOLT AGE PROBLEM. SYSTEM ENGINEER TO PROVIDE DWI. LPH (6681) FOR R. RONNING(6 887) @ 1527 01/16/90	EM	C
AI-133B-28				
CQE: Y	CLASS: N	WORK PERFORMED SEE D.W.I.		
COMPLETE:	01/18/90			
900334	AC-RW	WORK DESCRIPTION NEED TO PULL AC-10D FROM PIT TO LOCATE FLAPPER MISSING FROM RW-115 DISCHARGE CHECK VALVE.	MM	C
AC-10D				
CQE: Y	CLASS: 3	WORK PERFORMED PUMP WAS REMOVED FROM PUMP WELL TO LOCATE CHECK VALVE FLAPPER. FLAPPER WAS FOUND ON TOP OF SECOND STAGE. REMOVED FLAPPER. PUMP WAS VISUALLY IN SPECTED NO DAMAGE WAS FOUND. PUMP SET BACK IN PIT, LEFT LIFT AT .017". FG BUCK 1-25-90 AS PER MM-RR-RW-0001		
COMPLETE:	01/26/90			

MWO # SYSTEM
EQUIP ID

LEAD
DISC PRINT
STATUS

900357	AC-SFP	AC-7	WORK DESCRIPTION THE LATERIALS IN AC-7 HAVE BECOME PLUGGED.WE NEED TO CONNECT N2 BOTTLE TO THE SYSTEM AND FLUFF THE RESIN BED.INSTALL N2 BOTTLE UPSTREAM OF NG-233.THIS VALVE IS LOCATED IN RM. 5. MANWAY REMOVAL MAY BE NEEDED.	PE	C
CQE: Y	CLASS: 3	COMPLETE: 01/29/90	WORK PERFORMED HOOK UP LINE TO NG.233 TO BOTTLE N.G. AC 01-26-90 DISCONNECTED LINE FROM NG 233 TO BOTTLE REPLACED CAP ON NG 233 CLOSED DOOR TO ROOM 5. AC 01-26-90 CONNECTED NG LINE FROM NG BOTTLE TO NG233 TO SPARG LINE. WHEN SPARGI NG WAS COMPLETE DISCONNECTED LINE FROM BOTTLE TO NG233. FOLLOWED DETAILED WORK INSTRUCTIONS NO PROCEDURE WAS USED JC 1-27-90		
907506	DG	DG-2	WORK DESCRIPTION SA-193 DIAPHRAM APPEARS TO BE RUPTURED.	IC	C
CQE: Y	CLASS: N	COMPLETE: 01/19/90	WORK PERFORMED REMOVED PIPING AND REGULATOR FOR I&C REINSTALLED SAME PIPING AND NEW REGULATOR. LEAK CHECKED ALL FITTING CONNECTIONS OK. TJS 1-19-90 INSTALLE D CALIBRATED TEST GAUGE #00117 AND SPT AIR REGULATOR PRESSURE AT 180 PSI G. REMOVED TEST GAUGE AND INSTALLED PROCESS GAUGE. VERIFIED SYSTEM TO BE LEAK FREE USING SNOOP.		
907507	RC	FI-102-2	WORK DESCRIPTION NEED LUG REPLACED ON WIRE FI-102-2-2 TO FI-102-2 (CB-1,2,3, IB9-2 LUG BROKEN	EM	C
CQE: Y	CLASS: S	COMPLETE: 01/15/90	WORK PERFORMED REPLACED LUG-PERFORMED ST-ACCOUSTIC-1 F.1- SPECTRUM ANALYZER SECTIONS. PS 1-19-90 (MONTHLY ST) NOTE: LUG WAS BROKEN DURING THRESHOLD SECTIONS O F ST-ACCOUSTIC-1 WHERE LUG HAD NO EFFECT ON T. COMPLETETION OF SPECTRUM ANALYSIS SECTIONS COMPLETED UPON REPAIR OF LUG.		
907511	AC-RW	AC-10B	WORK DESCRIPTION AC-10B FAILED FLOW REQUIREMENTS OF ST-RW-2-NEED TO ADJUST PUMP LIFT.	MM	C
CQE: Y	CLASS: 3	COMPLETE: 01/26/90	WORK PERFORMED LIFT AS FOUND .138" TOATL LIFT .240" AS LEFT .018" ADJUSTED LIFT AS PER PROCEDURE MM RR RW 0001		