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December 20, 1989
JPN-89-084

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

SUBJECT: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
**Proposed Change to the Technical Specifications Regarding
Updated SRV Performance Requirements and Miscellaneous
Changes (JPTS-89-017)**

Dear Sir:

This application for an amendment to the James A. FitzPatrick Technical Specifications proposes new Safety/Relief Valve (SRV) performance limits to take credit for the currently installed SRV capacity. Other changes, unassociated with SRV performance, clarify selected portions of the Technical Specifications and correct minor typographical and editorial errors.

New SRV Performance Requirements

Three changes to the existing SRV performance limits are proposed. The first permits continued plant operation with two SRVs out of service. (Currently, only one SRV out of service is permissible for thirty days.) These changes also reduce the number of automatic depressurization system (ADS) valves required to be operable to five.

Secondly, the setpoints for all eleven SRVs are changed to a single nominal setpoint.

The third change increases the maximum permissible setpoint tolerance from one percent to three percent.

A detailed analysis of these changes has been performed for the Authority by the General Electric Company. The results of these analyses are summarized in a report entitled "Updated SRV Performance Requirements for the James A. FitzPatrick Nuclear Power Plant" (NEDC-31697P). Since this report contains proprietary information, copies will be transmitted under a separate cover.

The Authority plans to implement the revised SRV performance requirements during the 1990 refueling outage currently scheduled to start in March. Since recalibrating, testing and installing the SRVs requires several weeks, approval by February 15, 1990 is requested to support this planned outage.

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Five Specifications related to SRV performance are revised in this proposed amendment: Limiting Safety System Setting 2.2.1.B, "Reactor Coolant System" and associated Bases, (pages 27 and 29); Specifications 3.5.D and 4.5.D, "Automatic Depressurization System" and associated Bases, (pages 119, 120, and 128); and Specification 3.6.E and 4.6.E, "Safety Relief Valves" and associated Bases, (pages 142a, 143, and 152).

Miscellaneous Changes

Miscellaneous changes to seven Specifications clarify terminology, correct typographical errors, remove a surveillance requirement which should have been deleted as part of Amendment 130, and delete a duplicate specification.

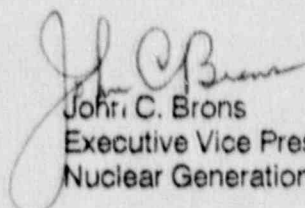
Specifications 1.2.1 ("Reactor Coolant System" on page 27), Table 4.2-2 ("Minimum Test and Calibration Frequency for Core and Containment Cooling Systems" on page 80), Specification 3.5.D ("Automatic Depressurization System" on page 120), Specification 4.6.E ("Safety Relief Valves" on page 143), and Bases Sections 1.2 (on page 29) 3.6.E (on page 152) are updated by this proposed amendment.

Enclosed for filing is the signed original of a document entitled "Application for Amendment to Operating License," with Attachments I and II thereto, comprising a statement of the proposed changes to the Technical Specifications and the associated Safety Evaluation.

In accordance with 10 CFR 50.91, a copy of this application and the associated attachments are being provided to the designated New York State official.

Should you or your staff have any questions regarding the proposed changes, please contact S. M. Toth of my staff.

Very truly yours,


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