

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE June 15, 1981
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: May 1981
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.	3623.	59136.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	52.7
20. Unit Availability Factor	0.0	0.0	52.7
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	51.3
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	49.2
23. Unit Forced Outage Rate	100.0	100.0	39.8
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

8106290067

(0/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289

UNIT TMI-I

DATE June 15, 1981

COMPLETED BY D. G. Mitchell

TELEPHONE (717) 948-8553

MONTH May 1981

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH May 1981DOCKET NO. 50-289UNIT NAME TMI-1DATE June 15, 1981COMPLETED BY D. G. MitchellTELEPHONE (717) 948-8553

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	5/1/81	F	744	D	1				Regulatory Restriction Order

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵
Exhibit I - Same Source

POOR ORIGINAL

OPERATIONS SUMMARY

The Unit was shutdown the entire month of May by order of the NRC. Core cooling was provided by the Decay Heat Removal System.

MAJOR SAFETY RELATED MAINTENANCE

During the month, while the Unit remained in the cold shutdown condition with restart activities continuing, the following major maintenance work was performed.

- 1) Decay Heat River Water Pump 1A was overhauled to eliminate high vibrations recently encountered on the pump shaft. The overhaul included removal, overhaul, and balancing of the motor, removal and inspection of the pump shafts and housing, and installation of new pump impeller bearings and line shafts. The impeller was balanced and reinstalled and the bowl housing was inspected and reused.

Reinstallation of the pump will continue into June.
- 2) Condensate Storage Tanks 1A and 1B were modified to remove the bladders which had been installed to minimize oxygen concentration in the water. The work included internal cleaning and inspection of the tank and installation of a new suction line screen. Work was completed and both tanks are now refilled.
- 3) Once Through Steam Generator "B" primary upper and lower manway and upper and lower handhole covers - work included removal of one (1) nut at a time, removal of stud lubricante, relube, re-installation of nut, and retorque. This work was performed satisfactorily.
- 4) Inspection of the bearings on Reactor Building purge valve "D" (AH-V-1D) was performed. Work included:

Removal of pneumatic operator, removal/inspection of bearings, and installation of new bearings. After completion of the above work the rubber seats of both AH-V-1C/1D were replaced. Replacement work included removal of duct section, removal of segment bolts and rubber seats, installation of an improved rubber seat material, and torquing of segment bolts.

Both of these jobs (seat replacement and bearing inspection) will be completed during the month of June.

- 5) Repairs to WDG-V-4 were performed with acceptable results. The valve was taken apart, a new disc was installed after a blue check verified the seating surface was good, the valve was re-assembled. Local leak rate testing of the valve was performed with acceptable results.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).