

CORE FLOW (% RATED)

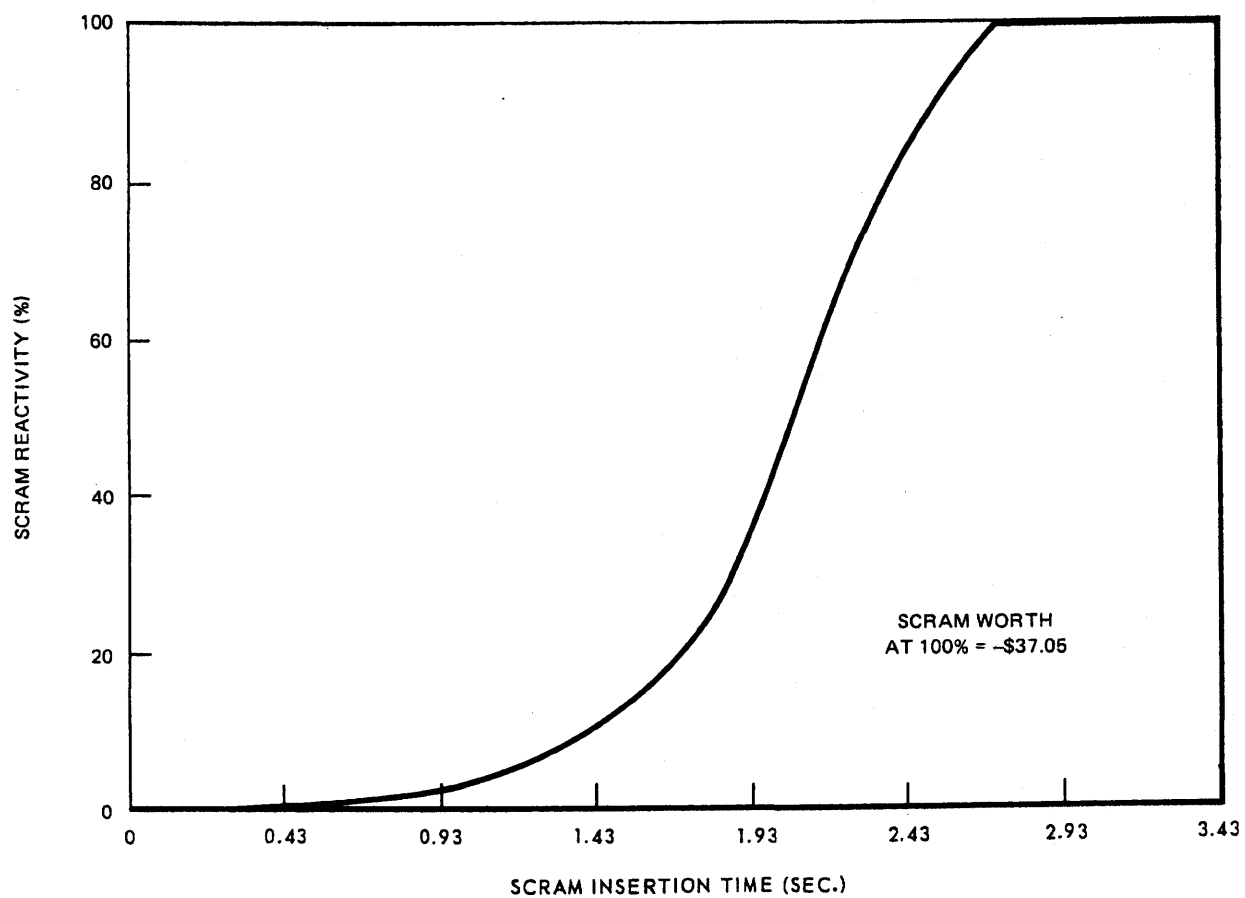
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PERRY NUCLEAR POWER PLANT

Typical Design Power/Flow Map
Standard Power/ Flow Region

Figure 15.0-1



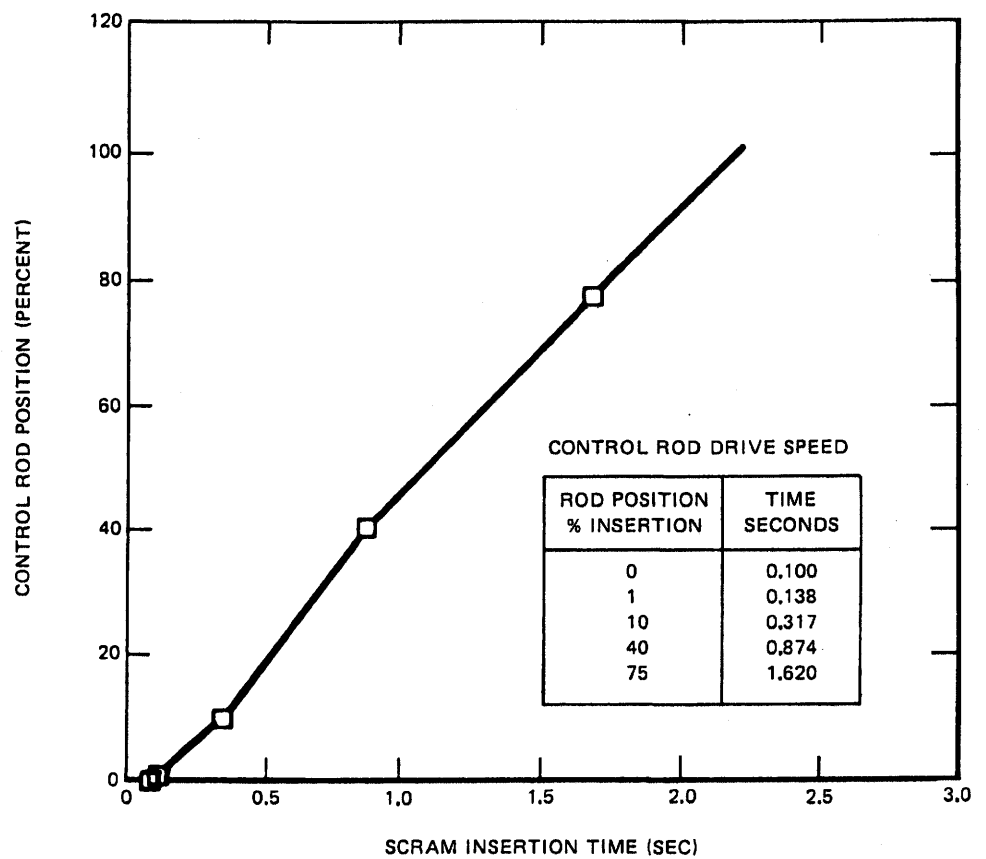
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PERRY NUCLEAR POWER PLANT

Scram Reactivity Characteristics
(REDY)

Figure 15.0-2



85-424-01

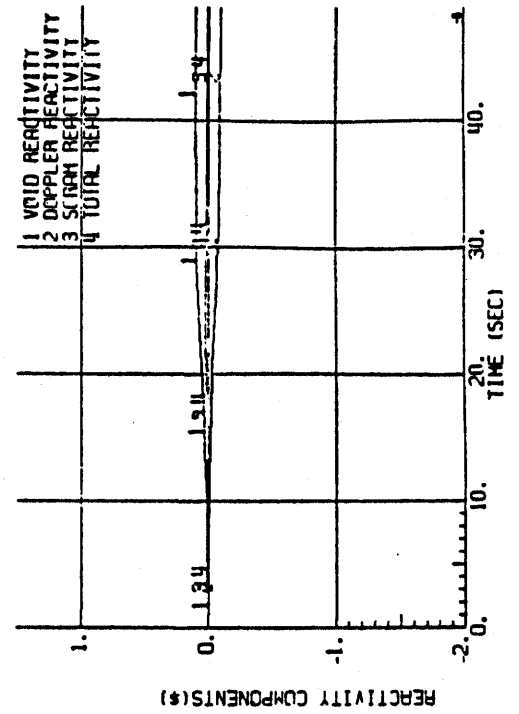
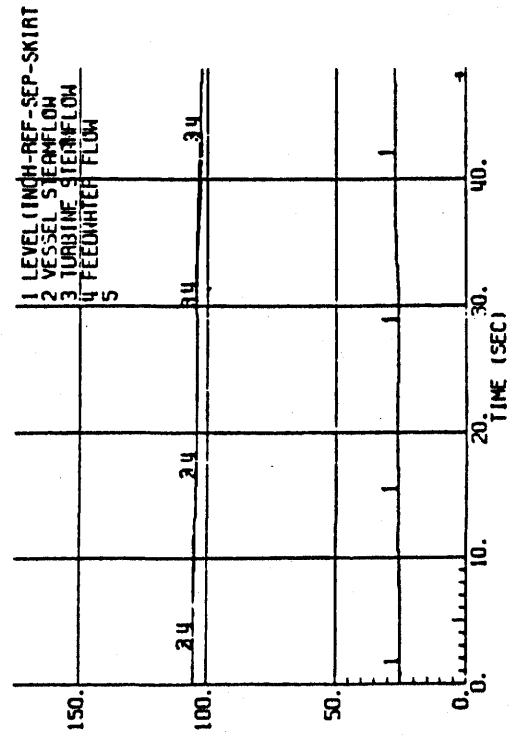
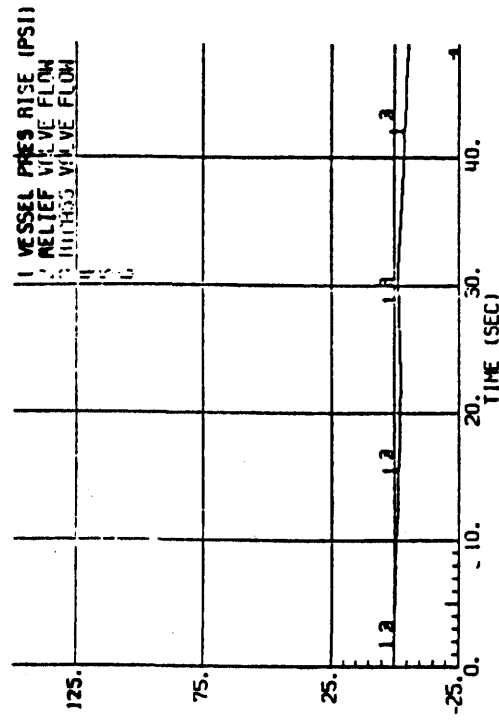
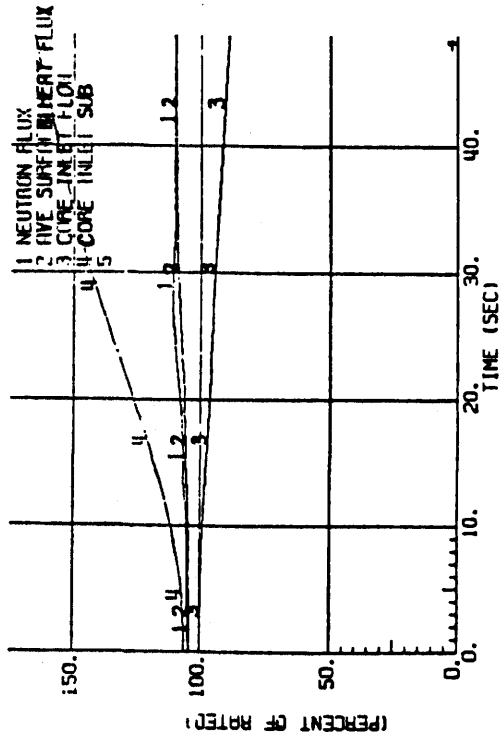
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PERRY NUCLEAR POWER PLANT

Scram Time Characteristics

Figure 15.0-3



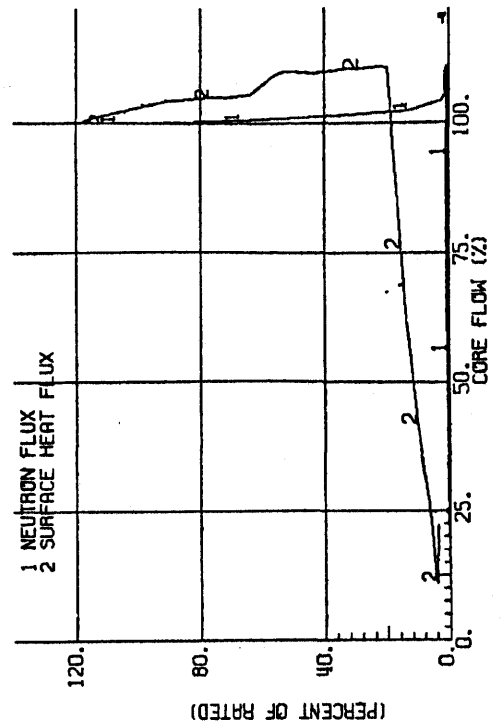
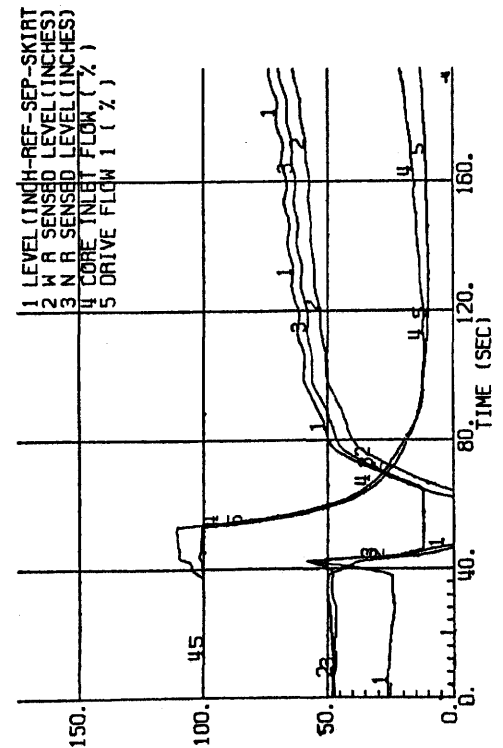
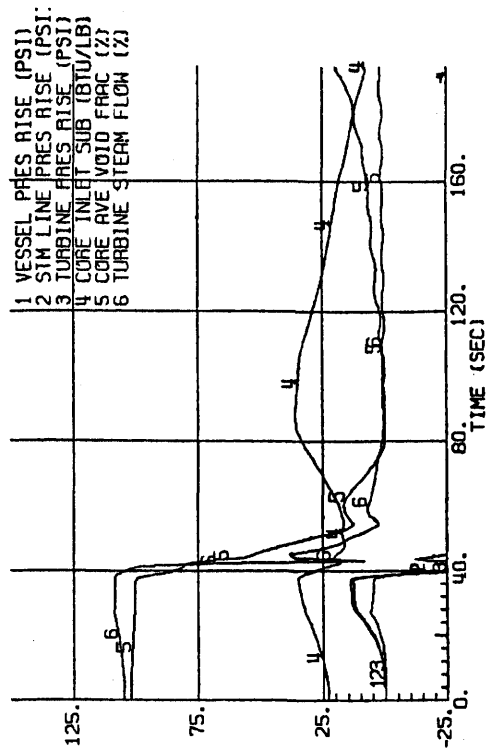
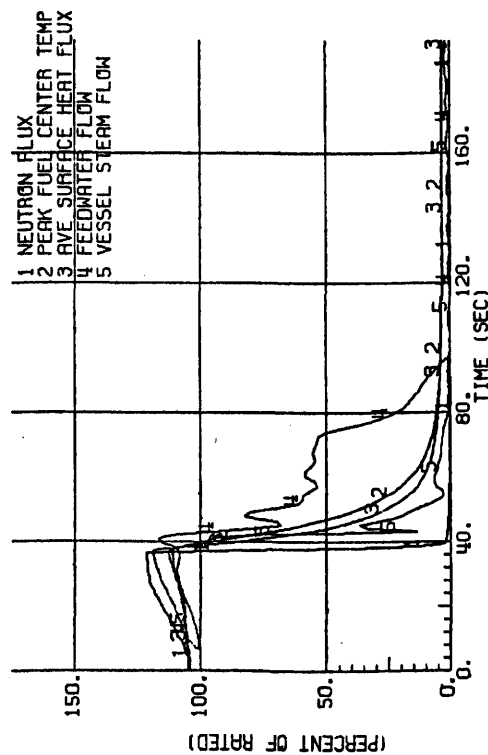
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PERRY NUCLEAR POWER PLANT

Loss of 100°F Feedwater Heating,
Automatic Flow Control

Figure 15.1-1



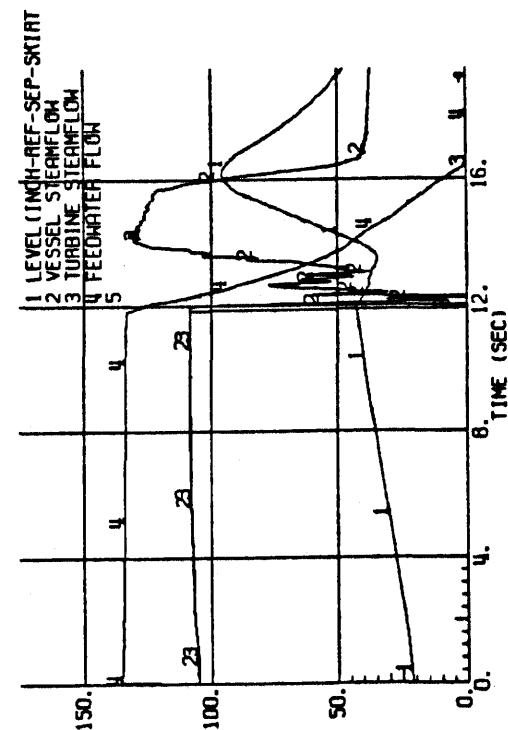
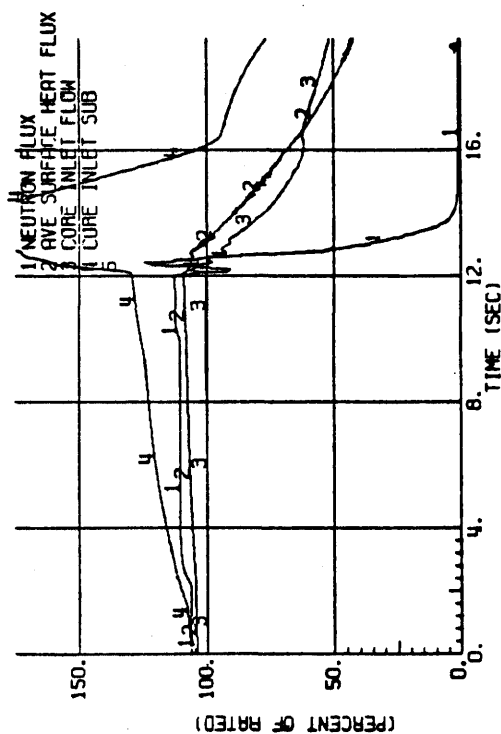
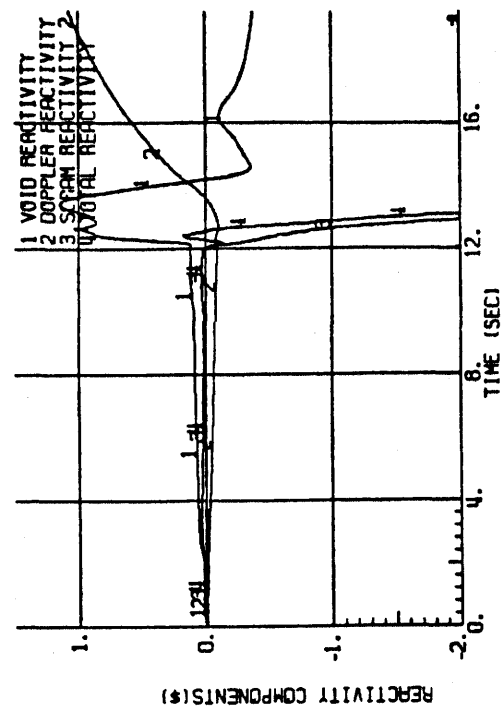
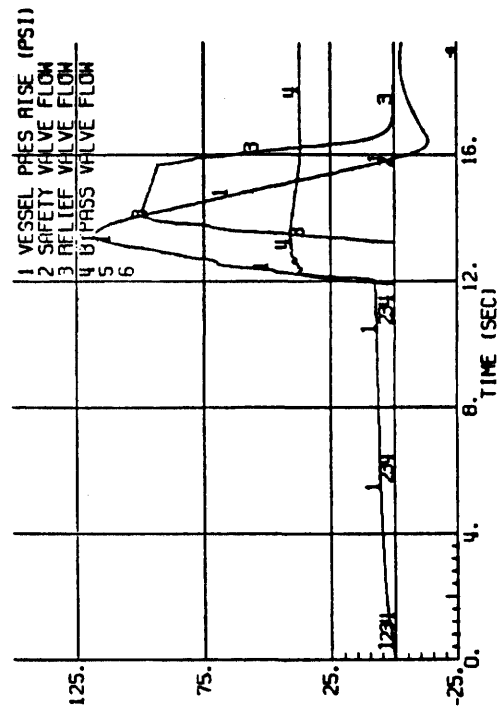
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PERRY NUCLEAR POWER PLANT

Loss of 100°F Feedwater Heating,
Manual Flow Control

Figure 15.1-2



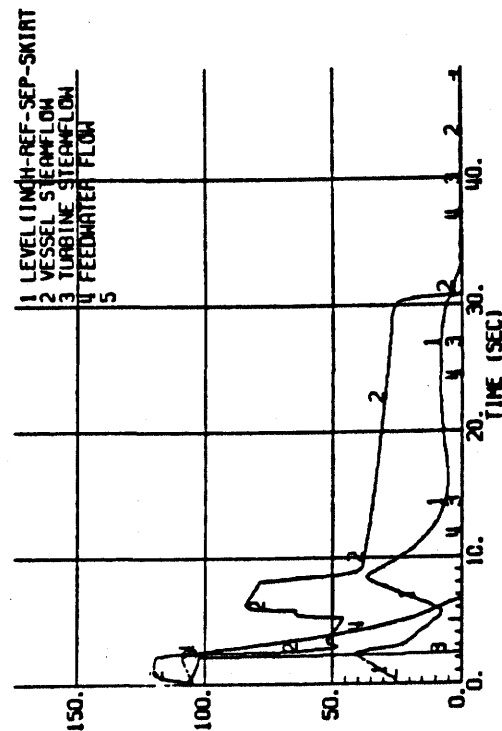
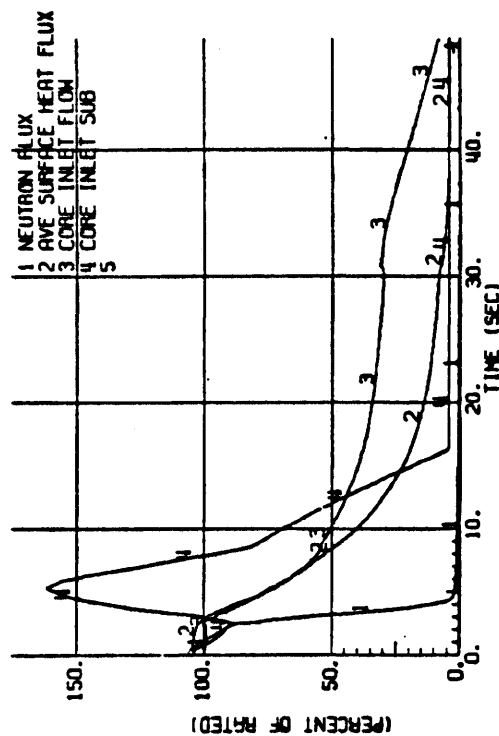
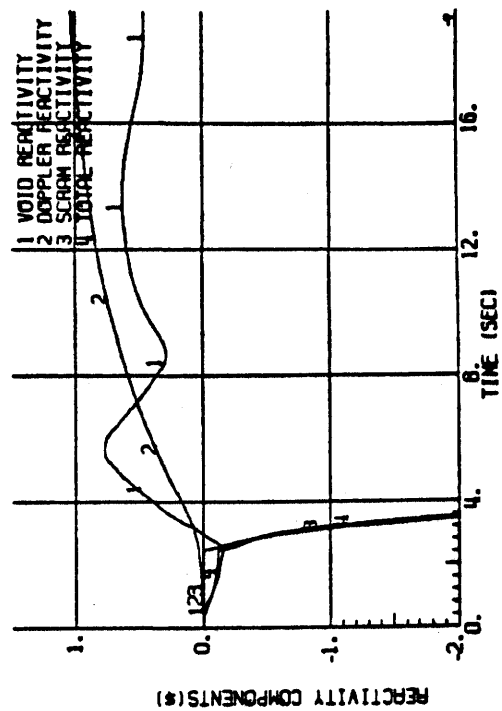
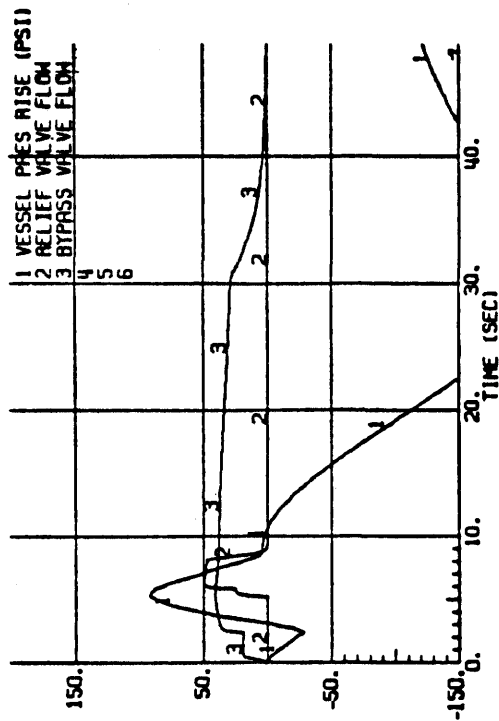
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure,
Maximum Demand, With
HWL Trips

Figure 15.1-3



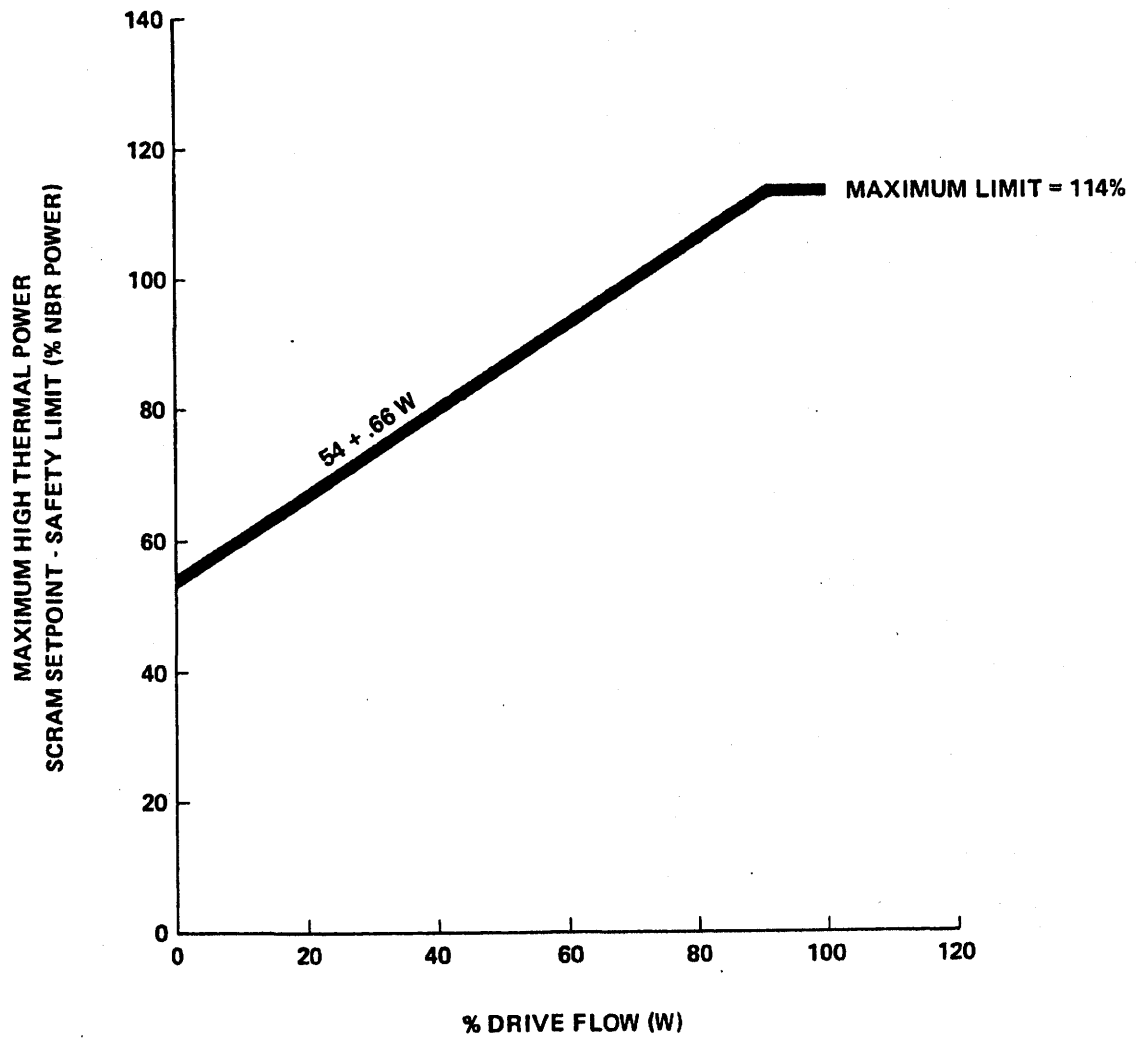
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PERRY NUCLEAR POWER PLANT

Pressure Regulator Failure -
Open to 130%

Figure 15.1-4



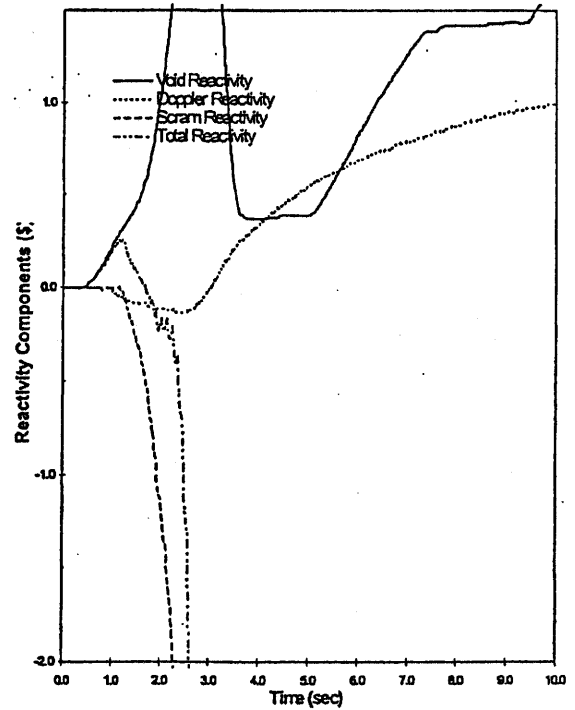
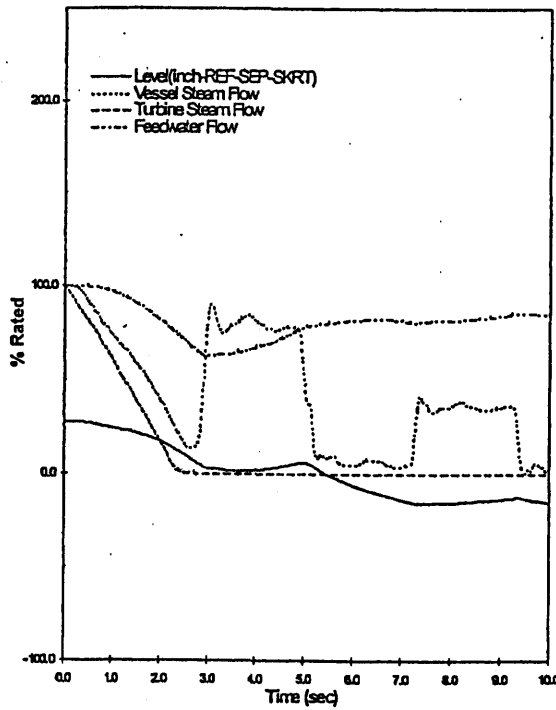
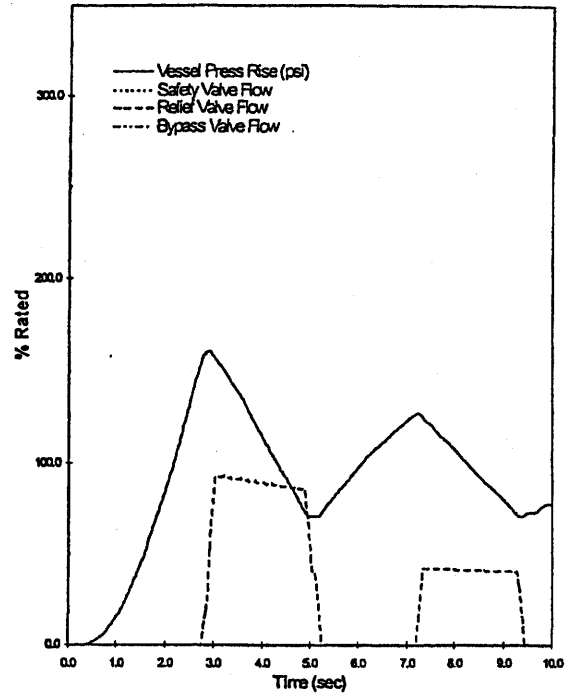
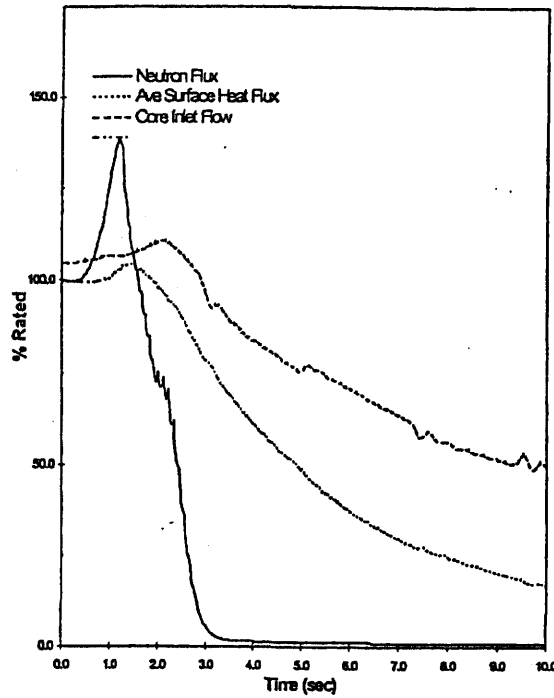
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PERRY NUCLEAR POWER PLANT

High Thermal Power Scram
Setpoint for Plant Operation
up to 100% NBR Power

Figure 15.1-5



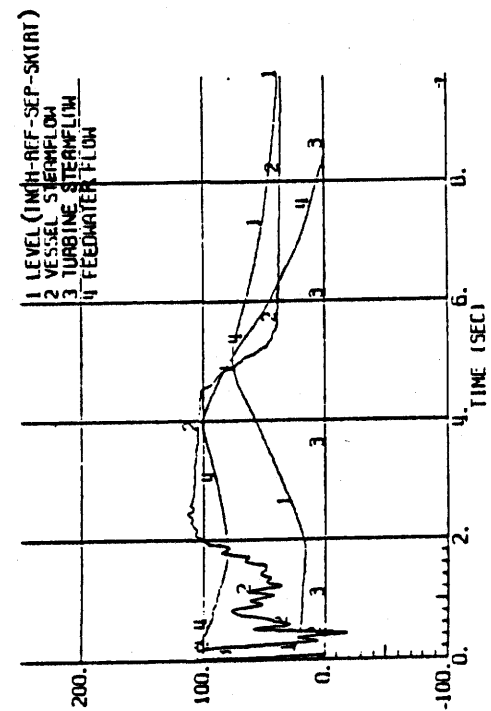
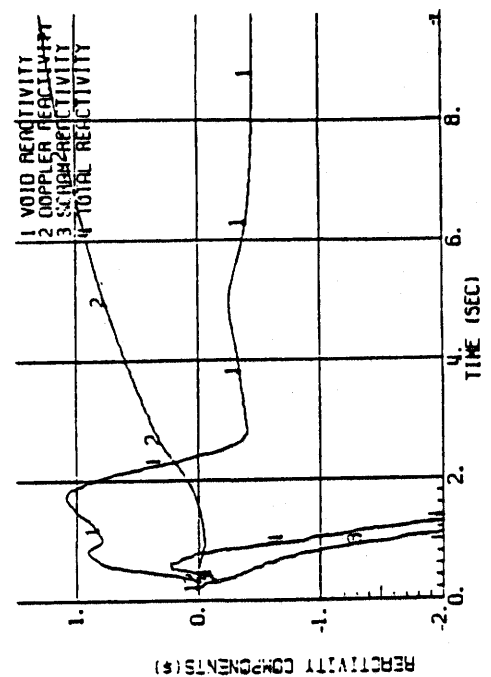
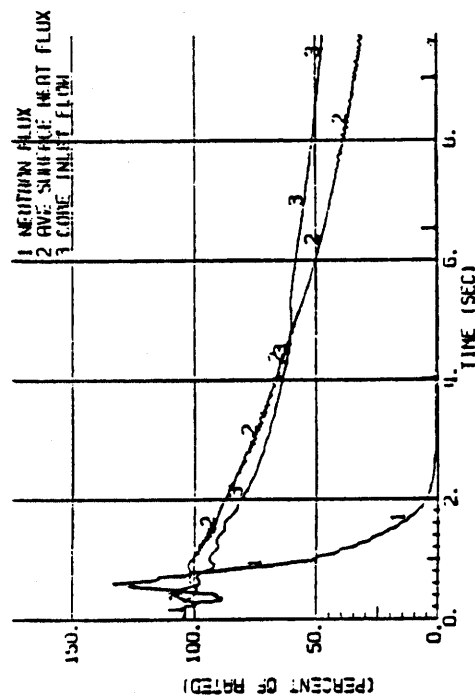
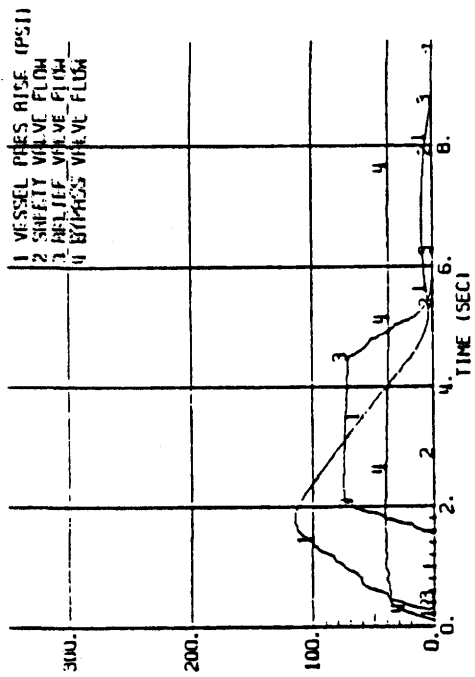
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PERRY NUCLEAR POWER PLANT

Pressure Regulator Downscale
Failure

Figure 15.2-1

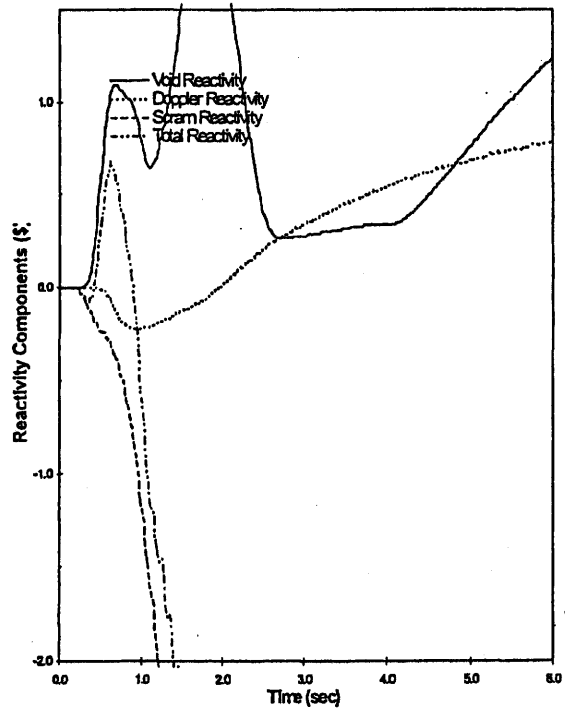
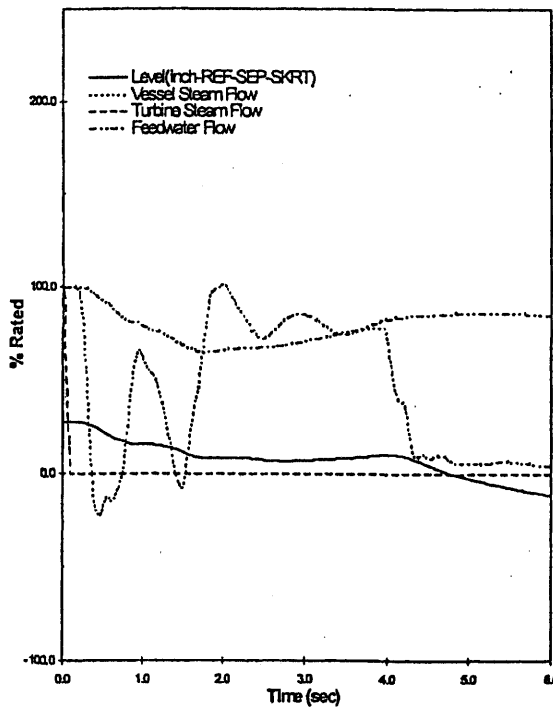
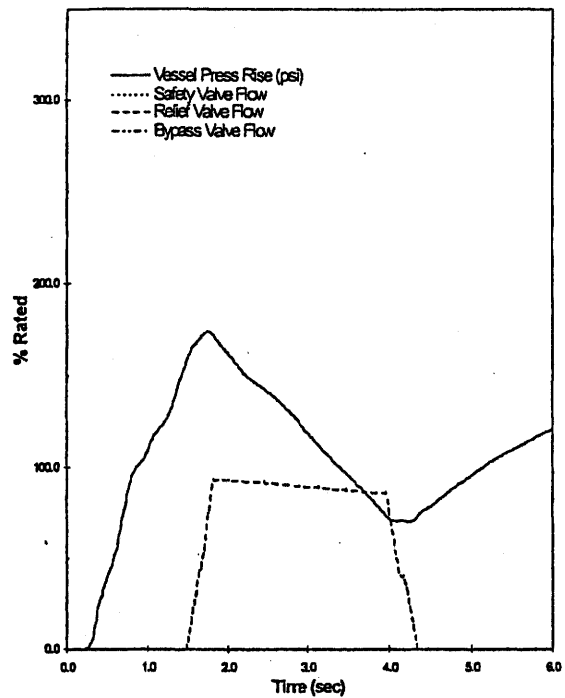
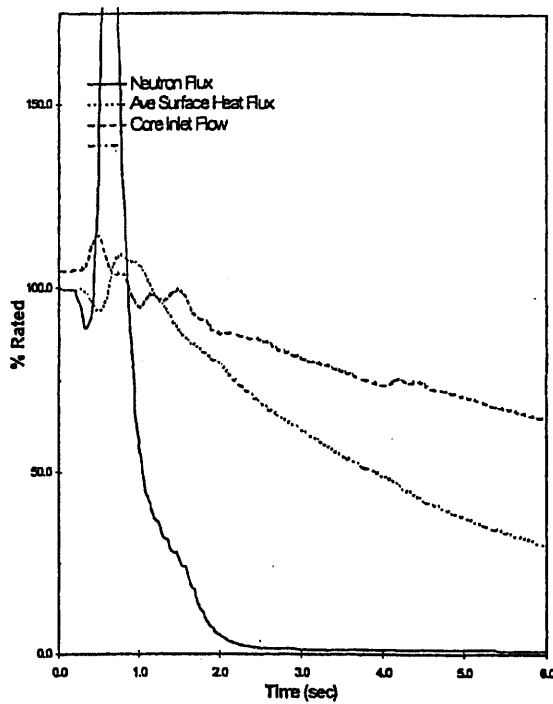


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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
Trip Scram, Bypass - On

Figure 15.2-2



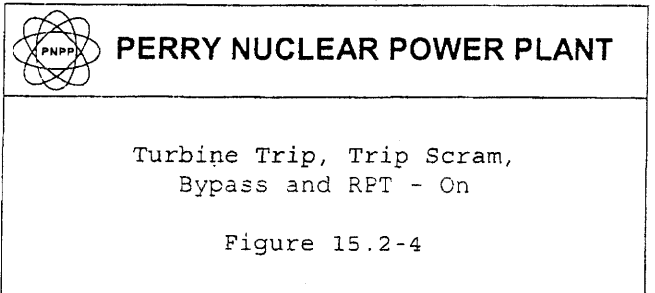
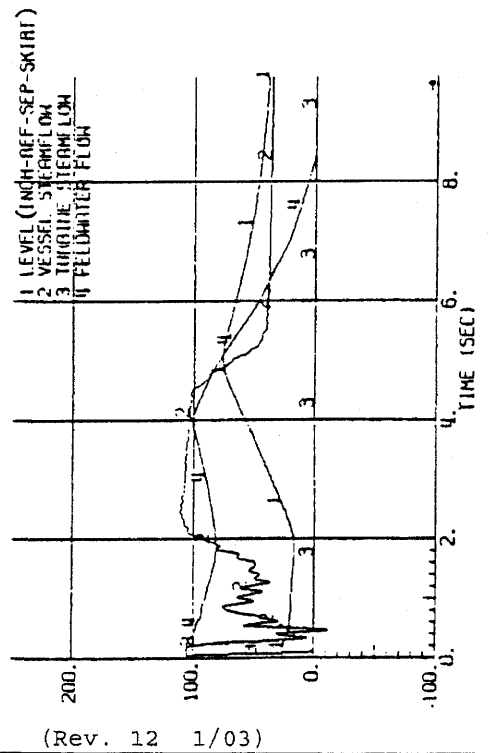
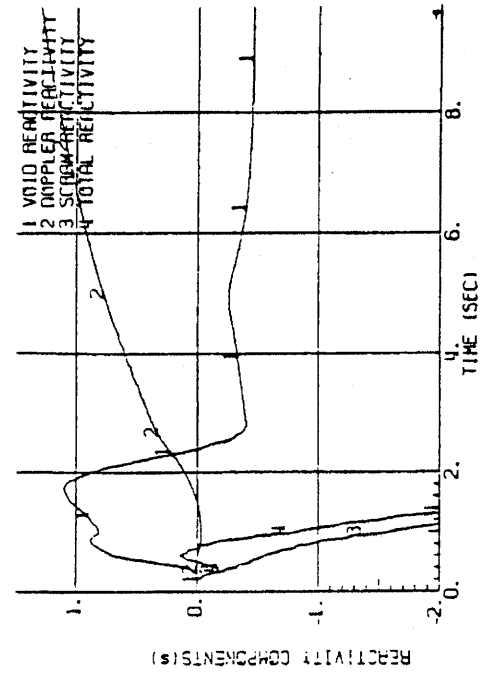
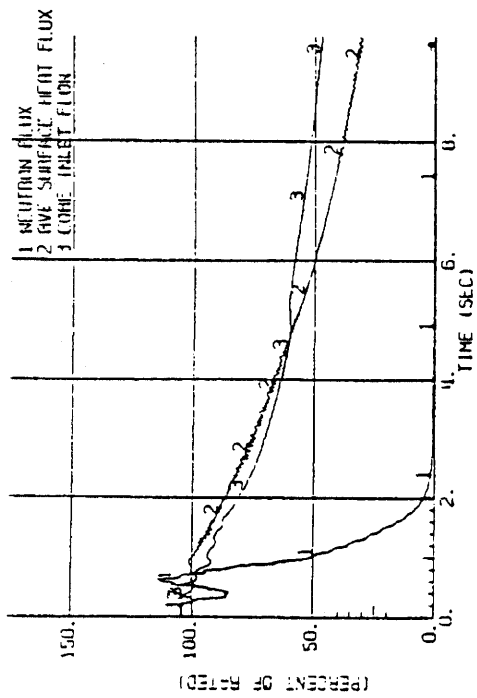
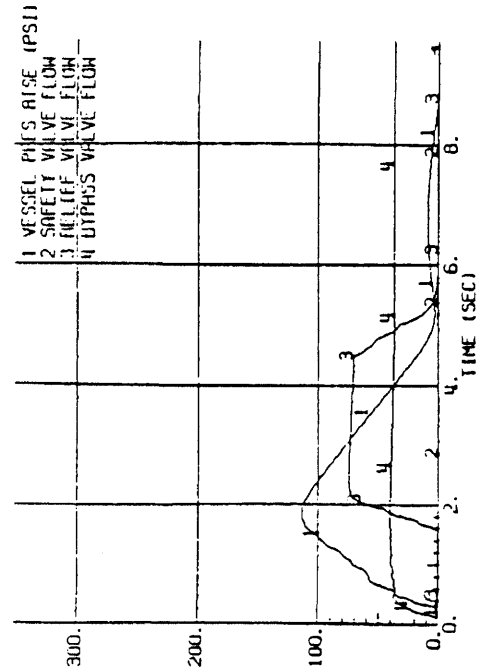
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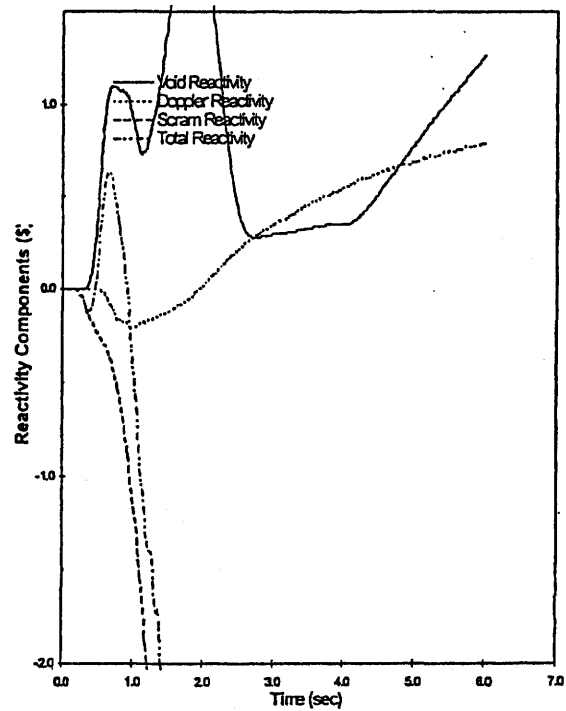
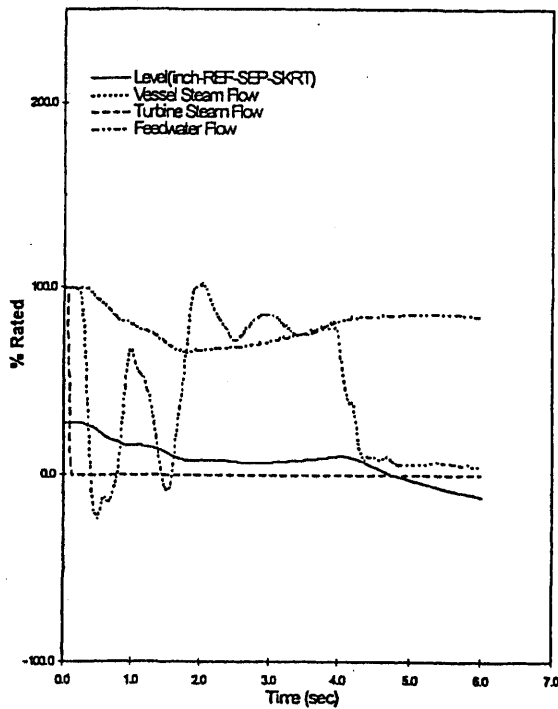
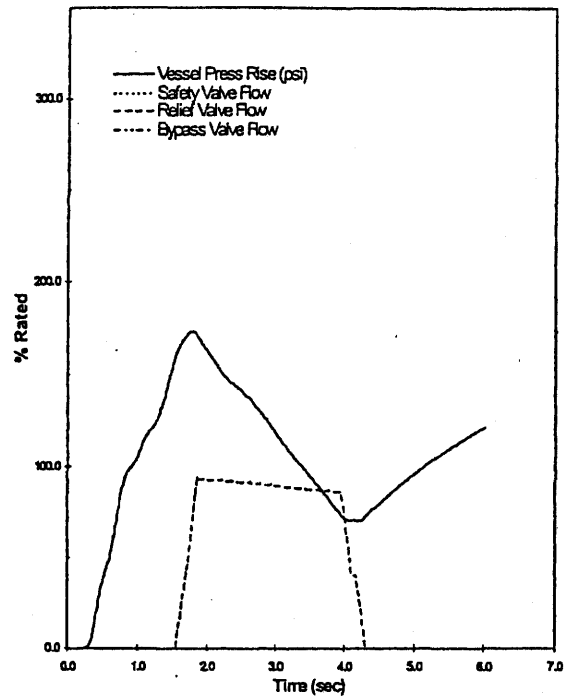
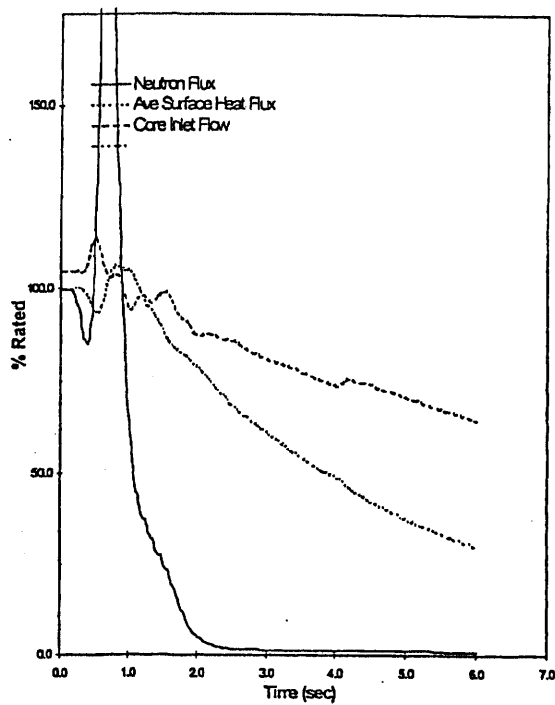


PERRY NUCLEAR POWER PLANT

Generator Load Rejection
Trip Scram, Bypass - Off

Figure 15.2-3





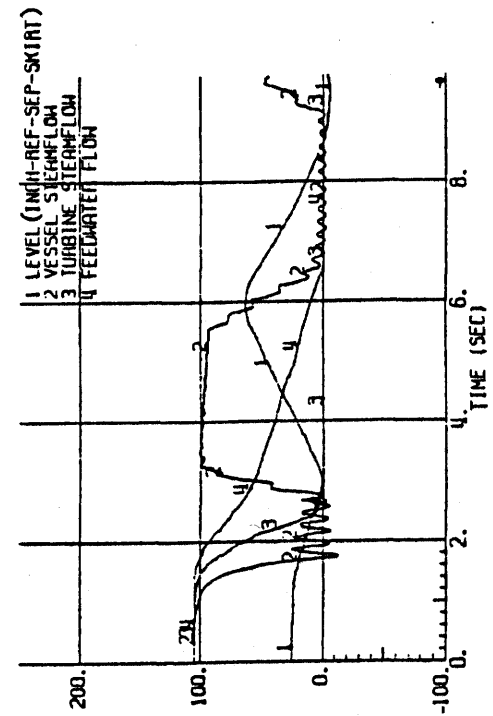
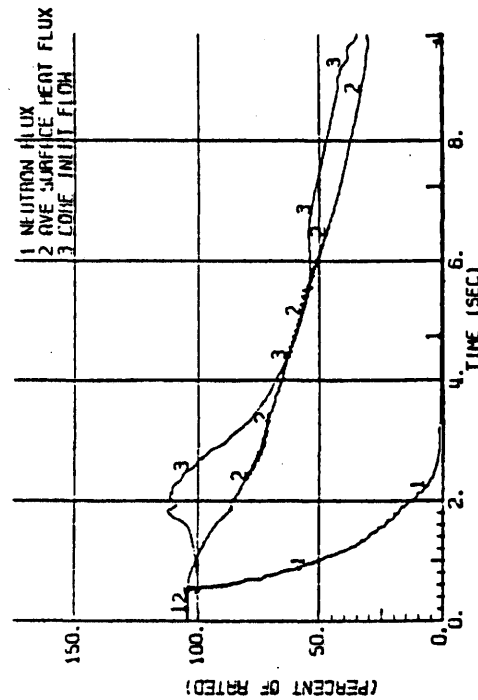
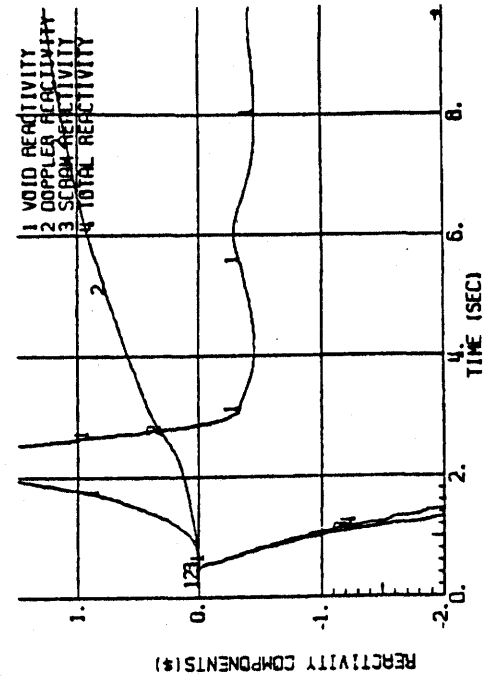
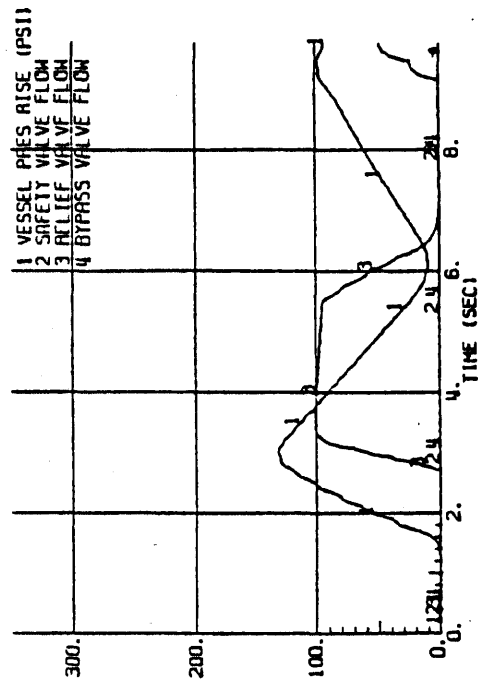
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PERRY NUCLEAR POWER PLANT

Turbine Trip, Trip Scram,
Bypass - Off, RPT - On

Figure 15.2-5



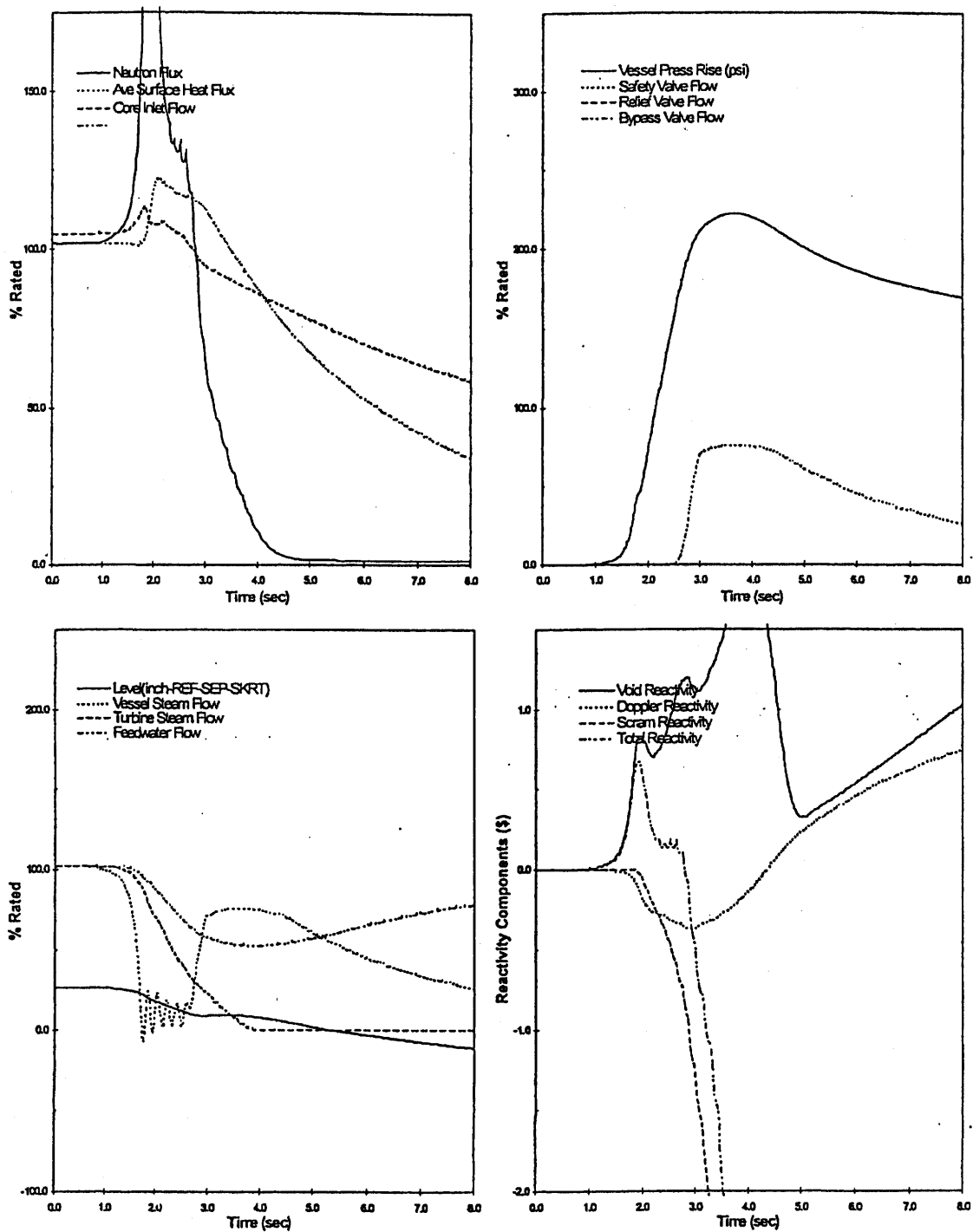
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PERRY NUCLEAR POWER PLANT

Three Second Closure of all Main
Steam Line Isolation Valves
With Position Switch Scram Trip

Figure 15.2-6a



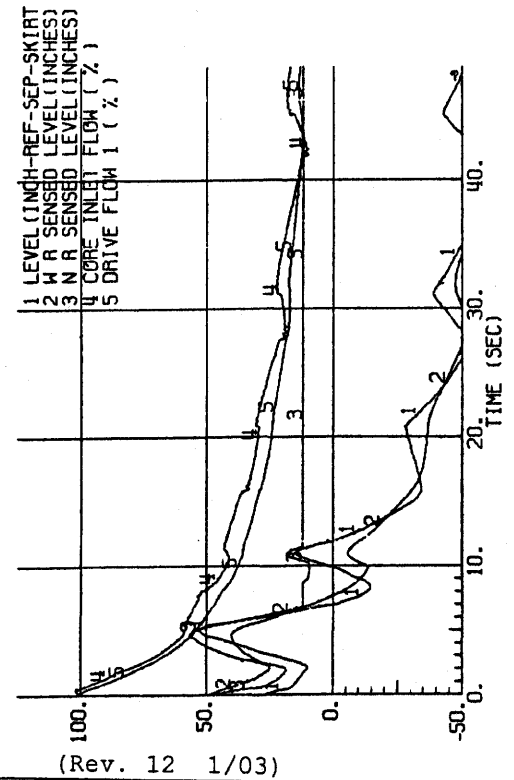
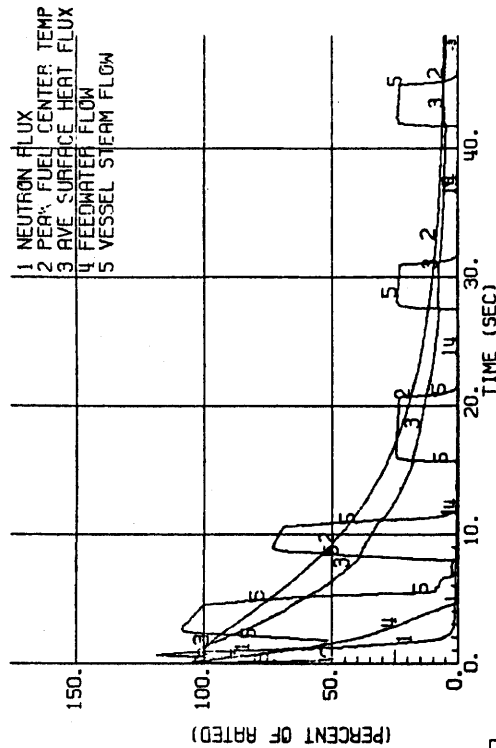
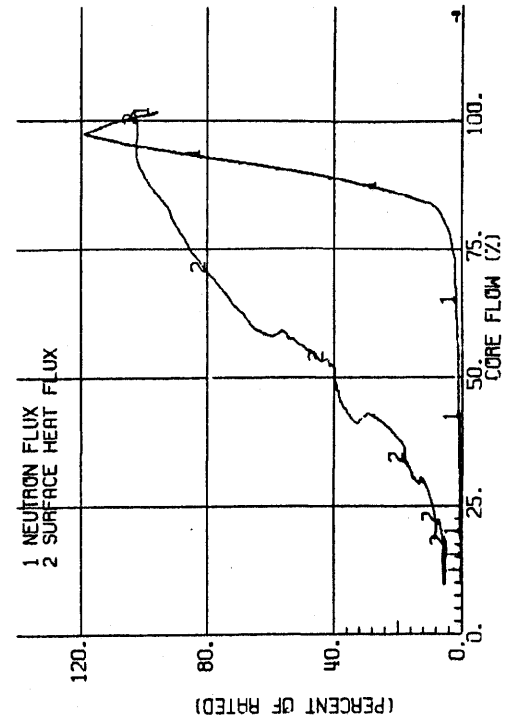
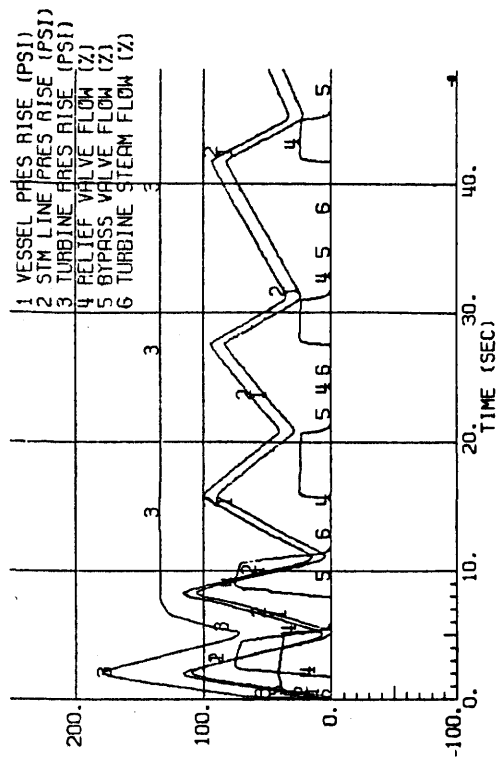
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PERRY NUCLEAR POWER PLANT

Response to MSIV
Closure With Flux Scram

Figure 15.2-6b



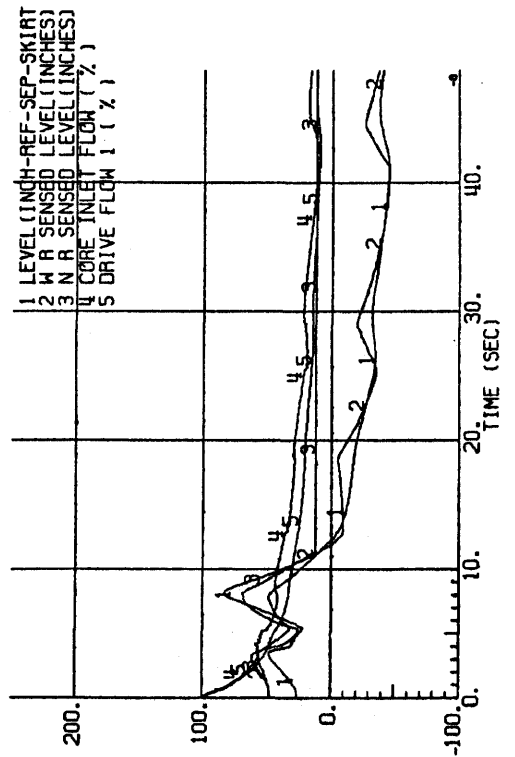
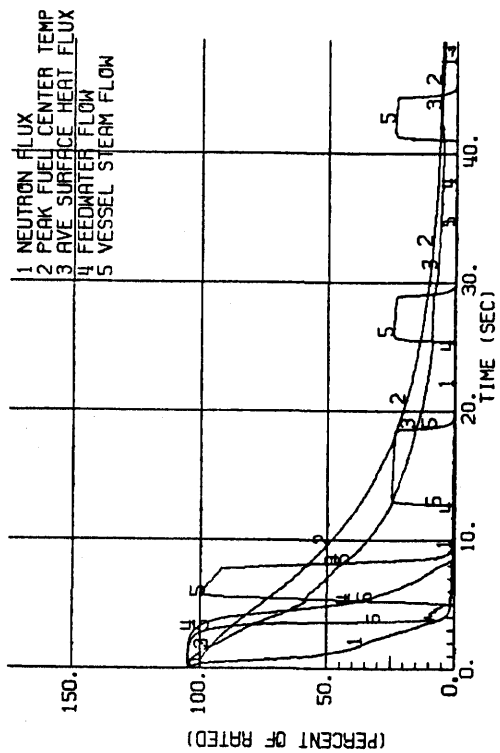
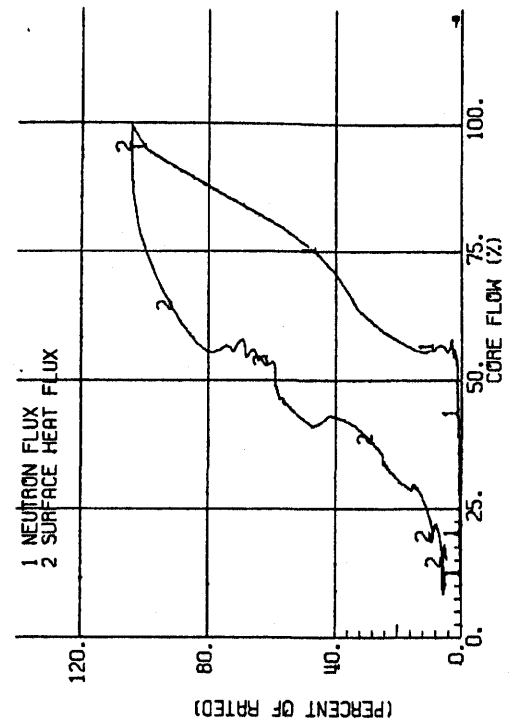
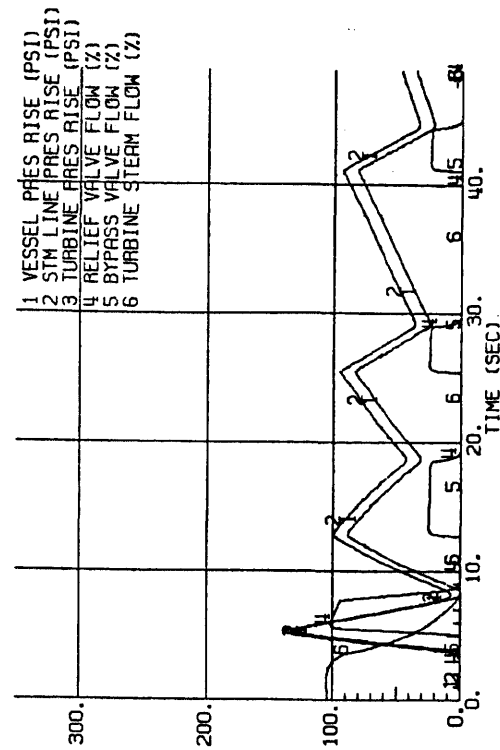
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PERRY NUCLEAR POWER PLANT

Loss of Condenser Vacuum at 2
Inches Per Second

Figure 15.2-7



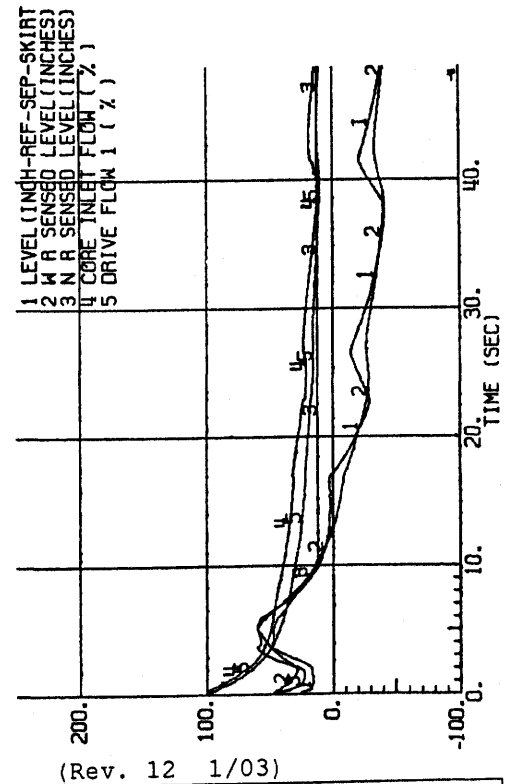
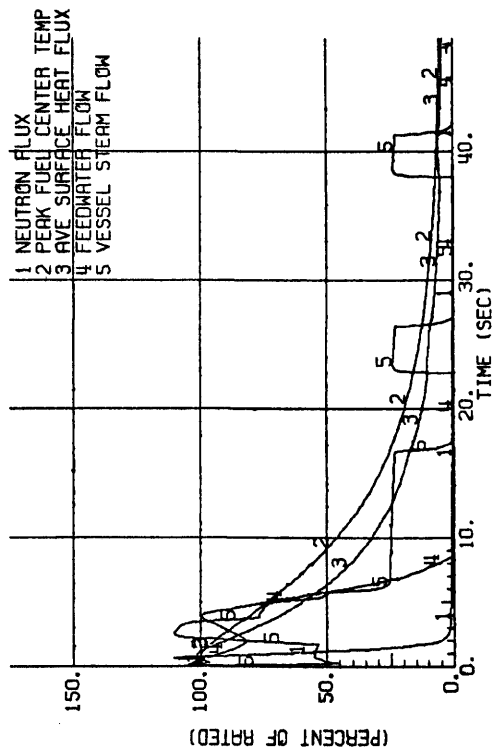
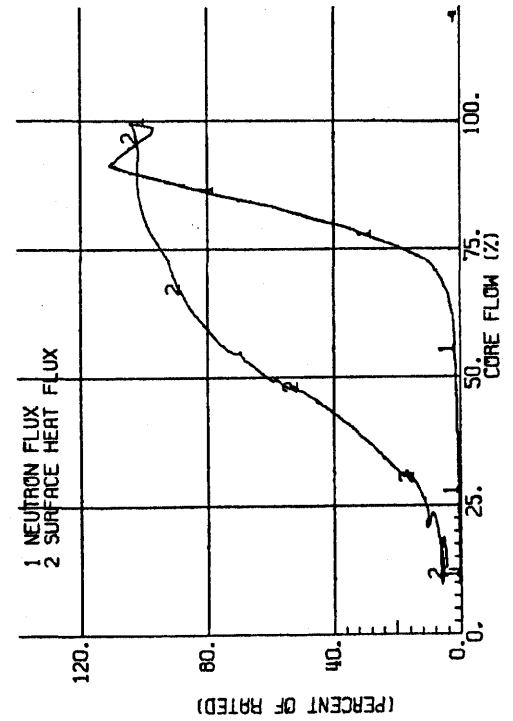
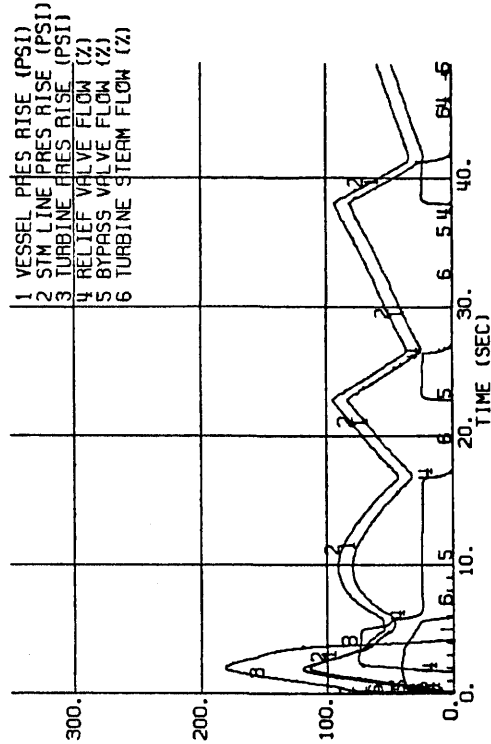
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Loss of Auxiliary Power Transformer

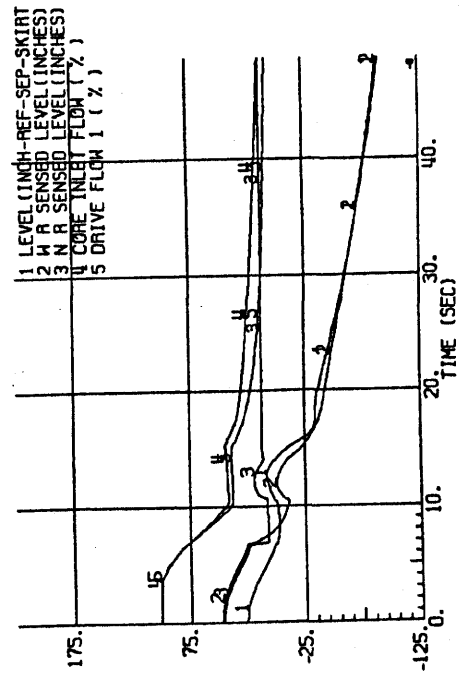
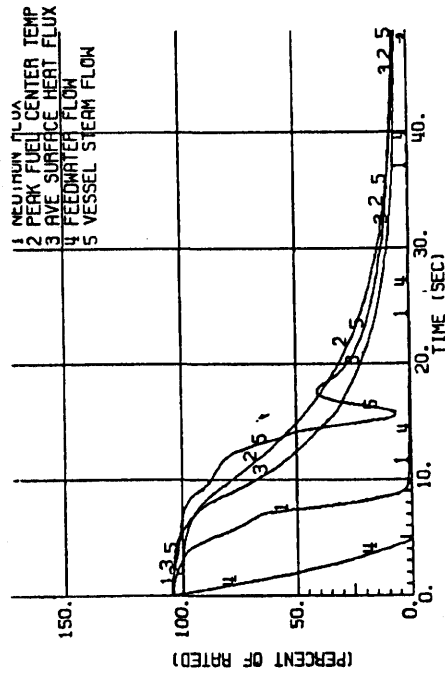
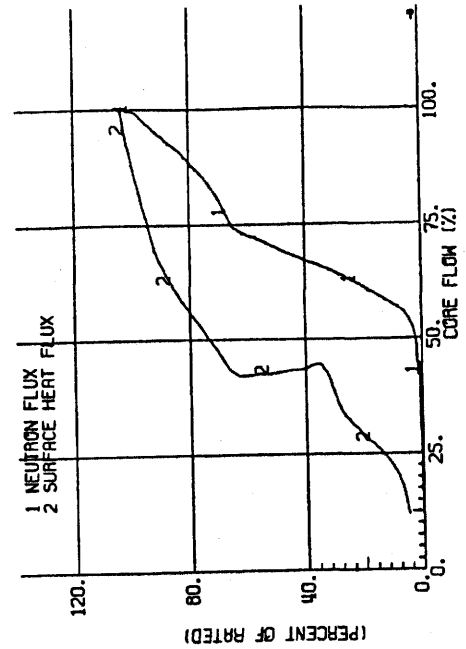
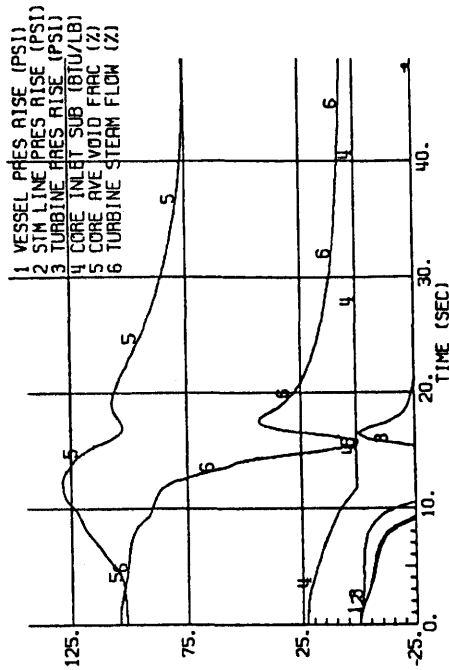
Figure 15.2-8



PERRY NUCLEAR POWER PLANT

Loss of All Grid Connections

Figure 15.2-9



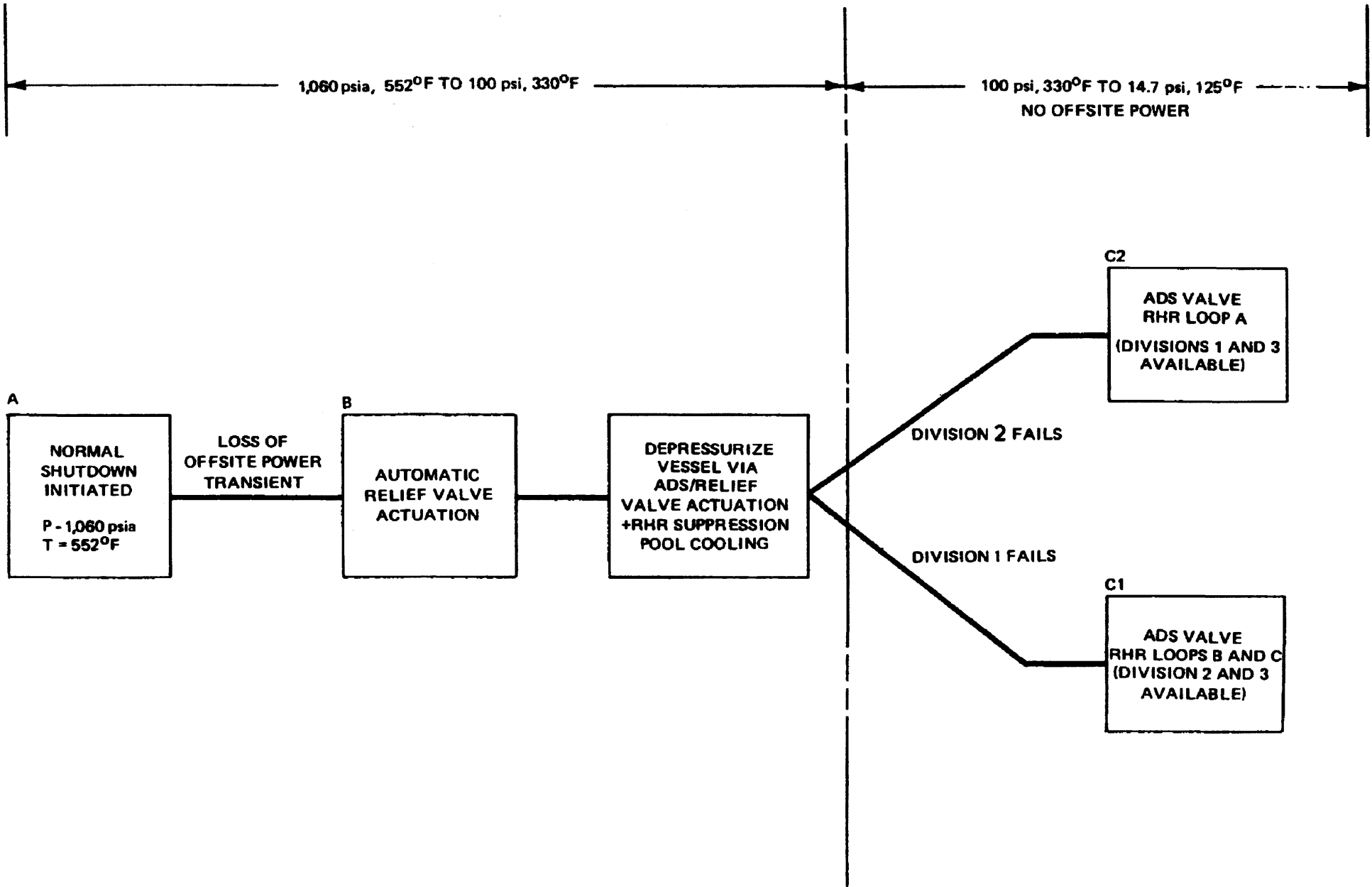
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PERRY NUCLEAR POWER PLANT

Loss of All Feedwater Flow

Figure 15.2-10



ACTIVITY A

Initial pressure 1,060 psia
Initial temperature 552°F

For purposes of this analysis, the following worst-case conditions are assumed to exist:

- The reactor is assumed to be operating at 102X rated power;
- A loss of power transient occurs (see Section 15.2.6);
- A simultaneous loss of onsite power (Division 1 or Division 2), which eventually results in the operator not being able to open one of the RHR shutdown cooling line suction valves.

ACTIVITY B

Initial system pressure 1,060 psia
Initial system temperature 552°F

Operator Actions

During approximately the first 13 minutes, reactor decay heat is passed to the suppression pool by automatic operation of the SRVs. Reactor water level will be returned to normal by HPCS and RCIC system automatic operation.

At approximately 13 minutes into the transient, the operator initiates depressurization of the reactor vessel. Controlled depressurization consists of controlling vessel pressure and water level by using selected safety/relief valves with the RCIC and HPCS systems. After approximately 13 minutes, it is assumed one RHR heat exchanger will be placed in the suppression pool cooling mode to remove decay heat. At this time, the suppression pool will be 107°F.

When the reactor pressure approaches 100 psig, the operator would normally prepare for operation of the RHR system in the shutdown cooling mode. At this time (135 min), the suppression pool temperature will be 146°F.

ACTIVITY C1 (Division 1 fails, Division 2 available)

System pressure approximately 100 psig
System temperature approximately 330°F

Operator Actions

The operator establishes a closed cooling path as follows:

- Utilizing RHR loops B and C together, water is taken from the suppression pool and is pumped directly into the reactor vessel. The water passes through the vessel (picking up decay heat) and out the ADS valves returning to the suppression pool as shown in Figure 15.2-14. Suppression pool water is then cooled by operation of RHR loop B in the cooling mode (see Figure 15.2-15). In this alternate cooling path, RHR loop C is used for injection and RHR loop B for cooling. Cold shutdown is achieved approximately 9.5 hours after transient occurred.

ACTIVITY C2 (Division 2 fails, Division 1 available)

System pressure approximately 100 psig
System temperature approximately 330°F

Operator Actions

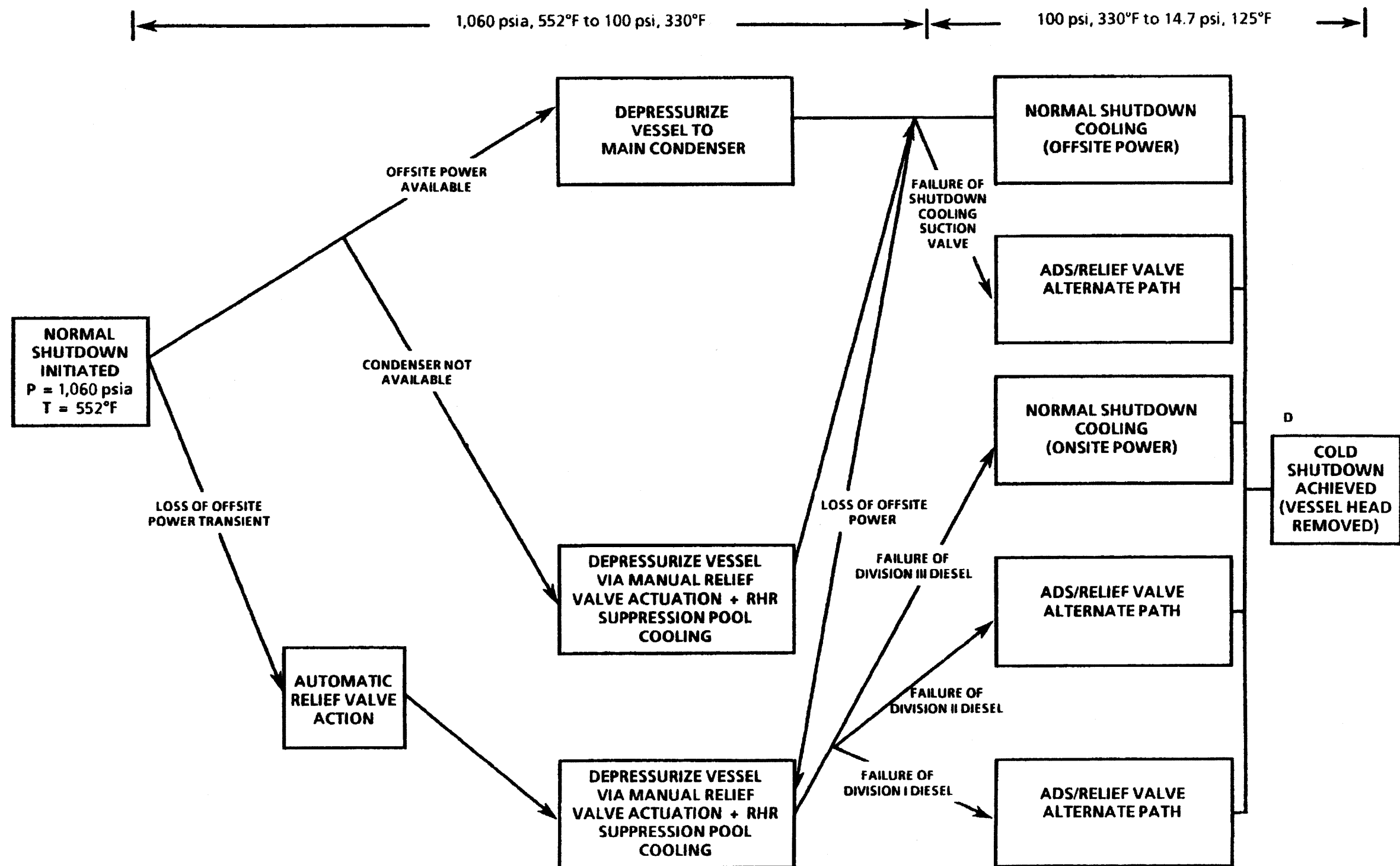
Utilizing RHR loop A (Figure 15.2-15, Sheet 2 of 2) instead of loop B, an alternate cooling path is established as in Activity C1. Item (a) above. LPCS is utilized in lieu of LPCI C. Again, cold shutdown is reached in approximately 9.5 hours.

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PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

ADS/RHR
COOLING LOOPS

FIGURE 15.2-11

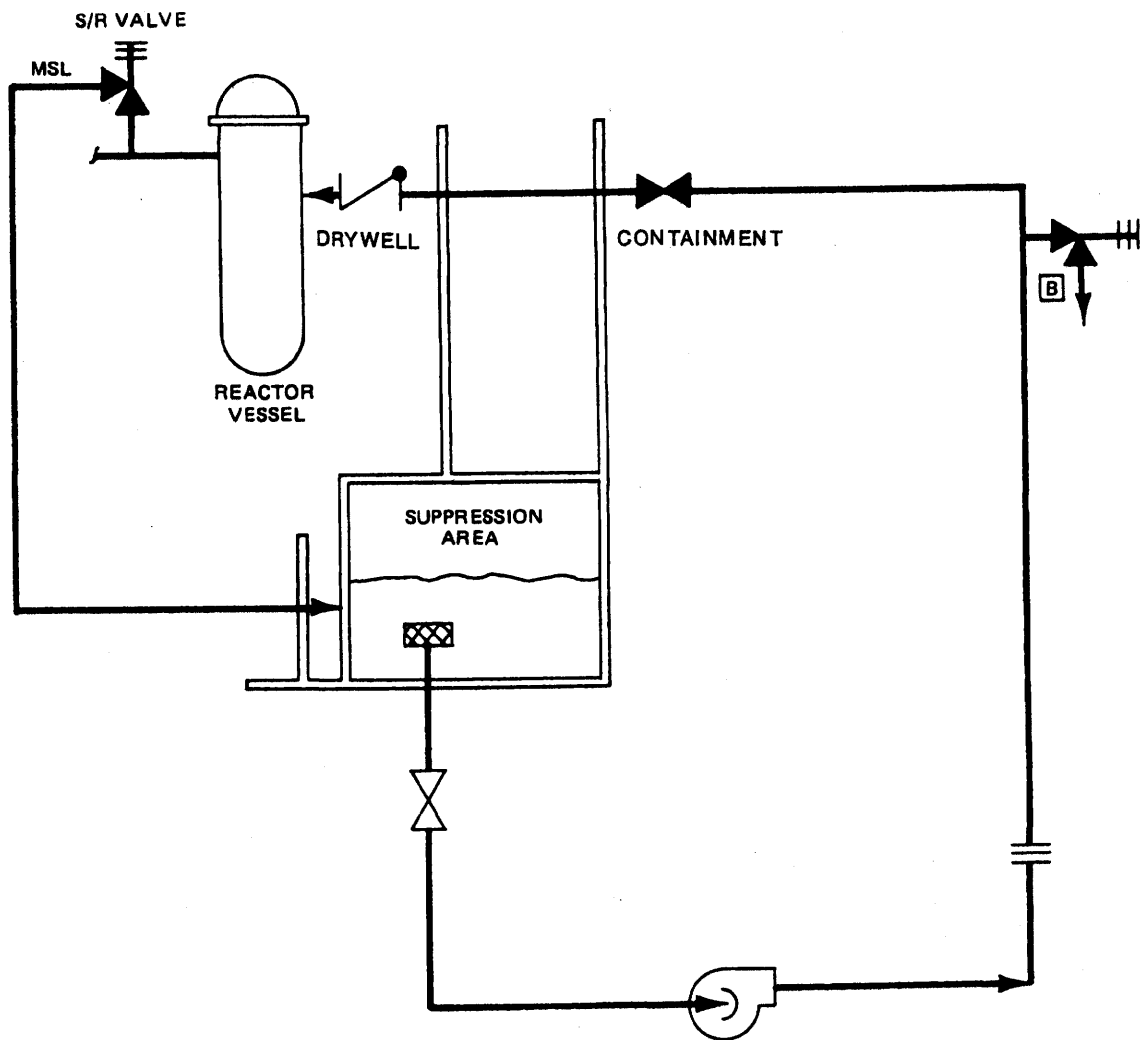


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PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

SUMMARY OF PATHS AVAILABLE
TO ACHIEVE COLD SHUTDOWN

FIGURE 15.2-12



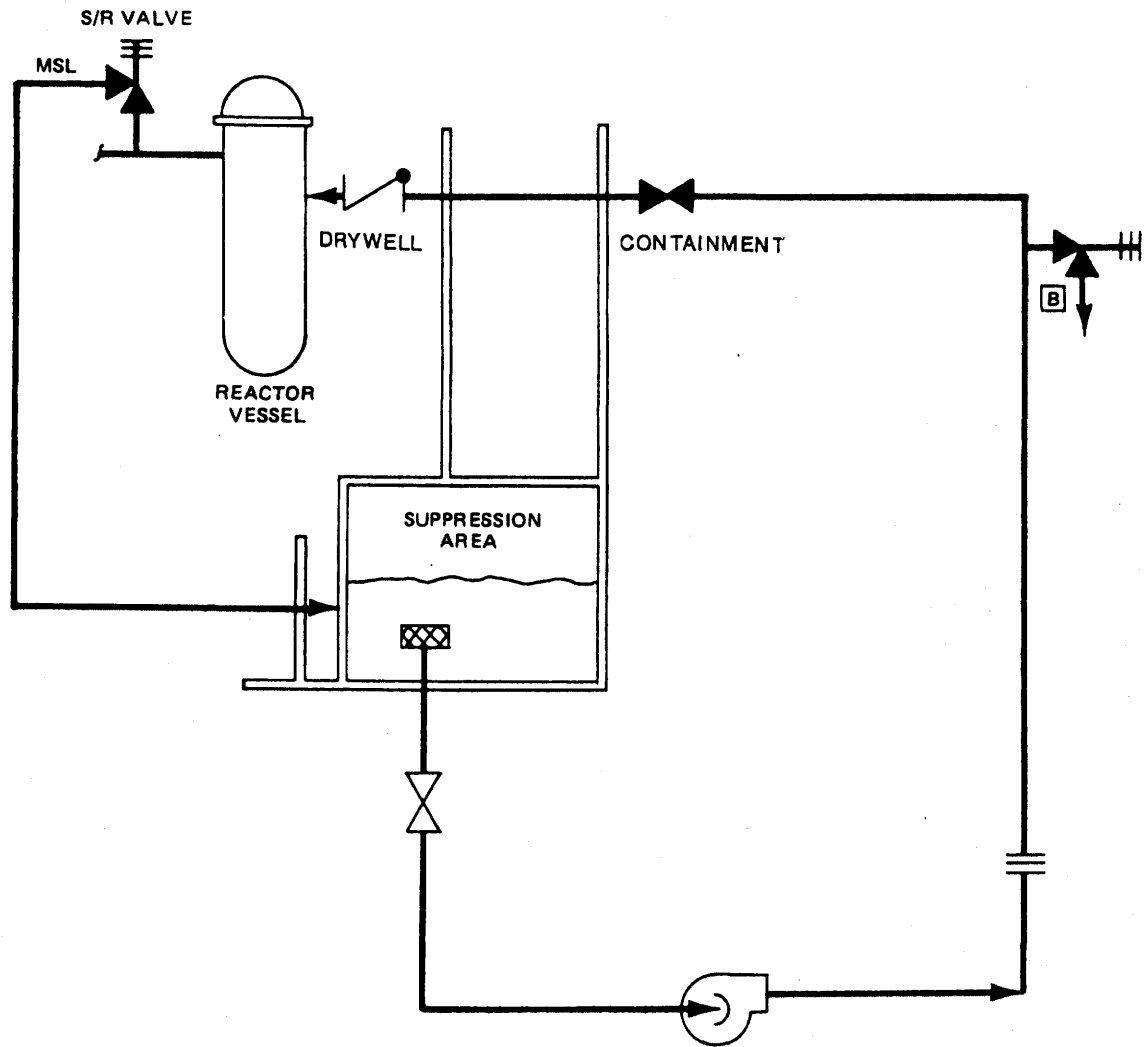
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PERRY NUCLEAR POWER PLANT

RHR Loop C

Figure 15.2-14 (Sheet 1 of 2)



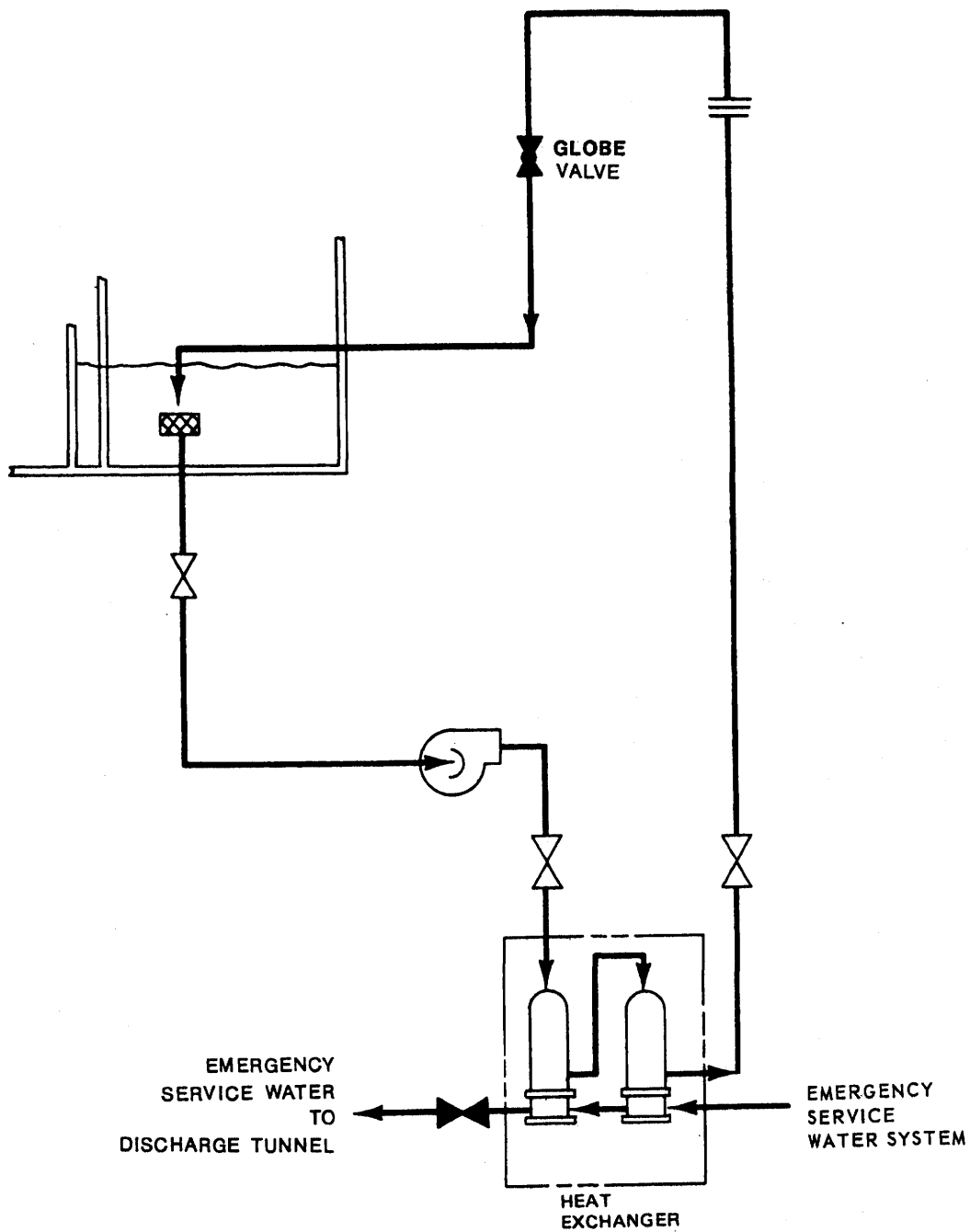
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PERRY NUCLEAR POWER PLANT

LPCS

Figure 15.2-14 (Sheet 2 of 2)



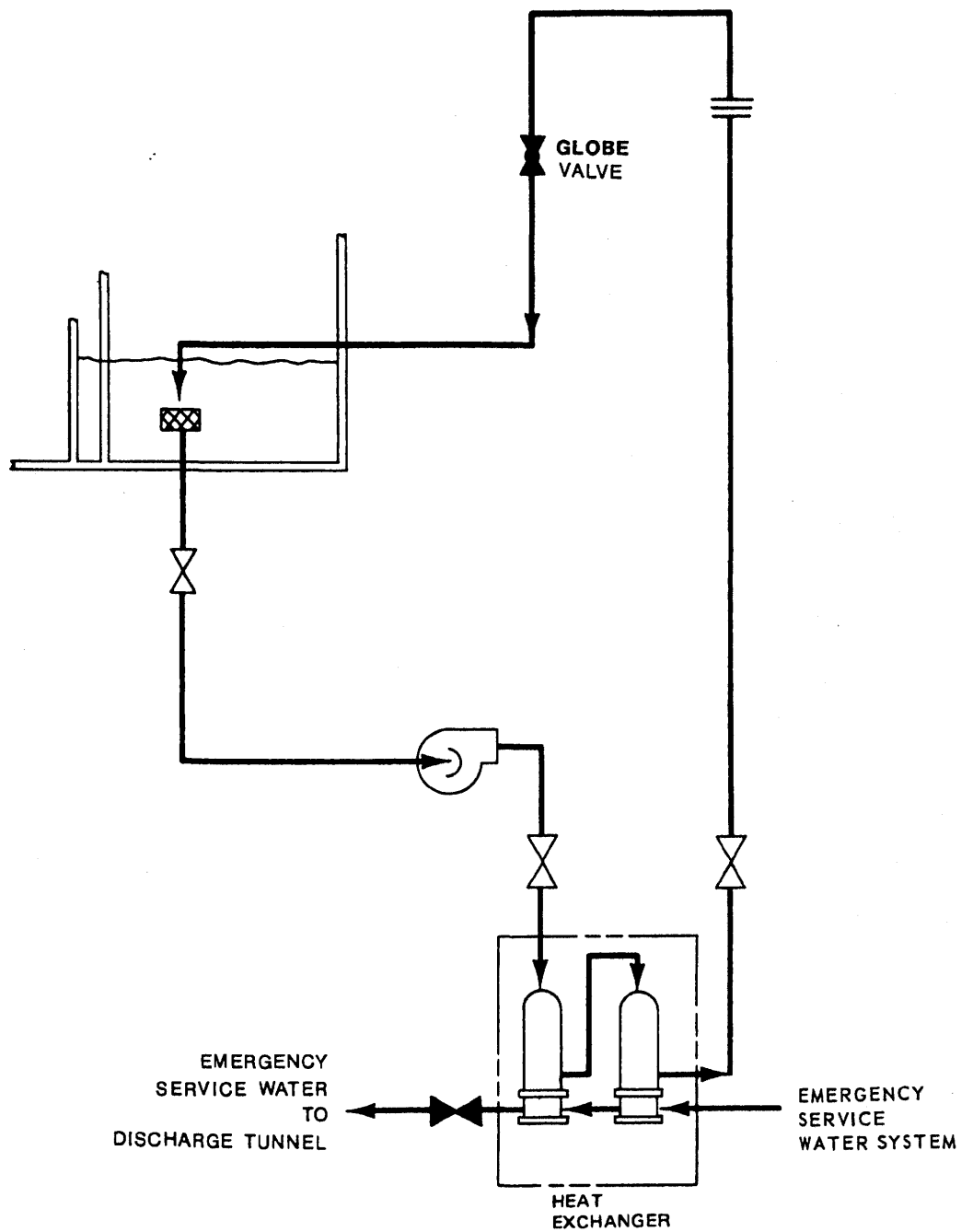
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PERRY NUCLEAR POWER PLANT

RHR Loop B (Suppression Pool
Cooling Mode)

Figure 15.2-15 (Sheet 1 of 2)



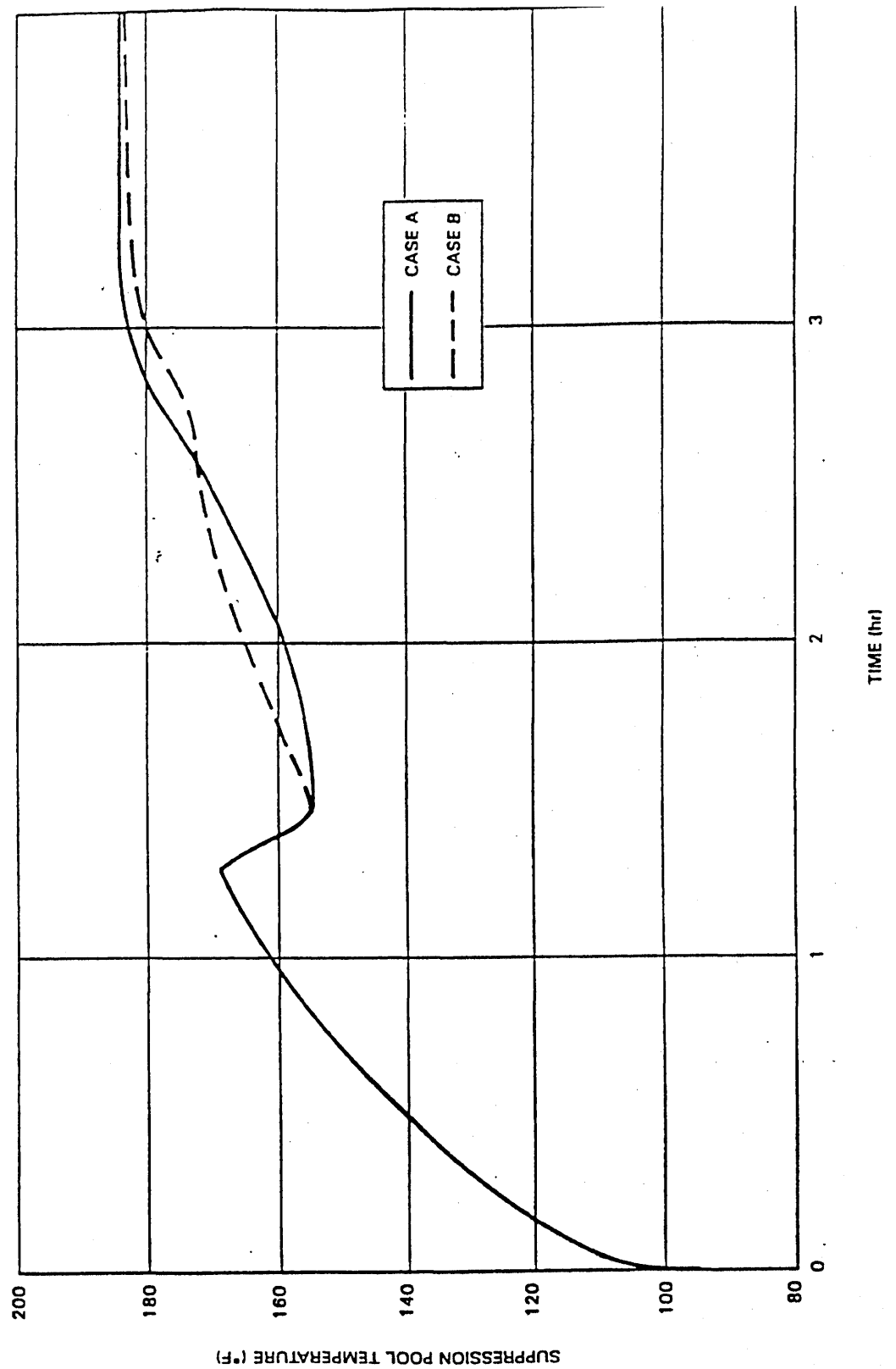
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PERRY NUCLEAR POWER PLANT

RHR Loop A (Suppression Pool
Cooling Mode)

Figure 15.2-15 (Sheet 2 of 2)



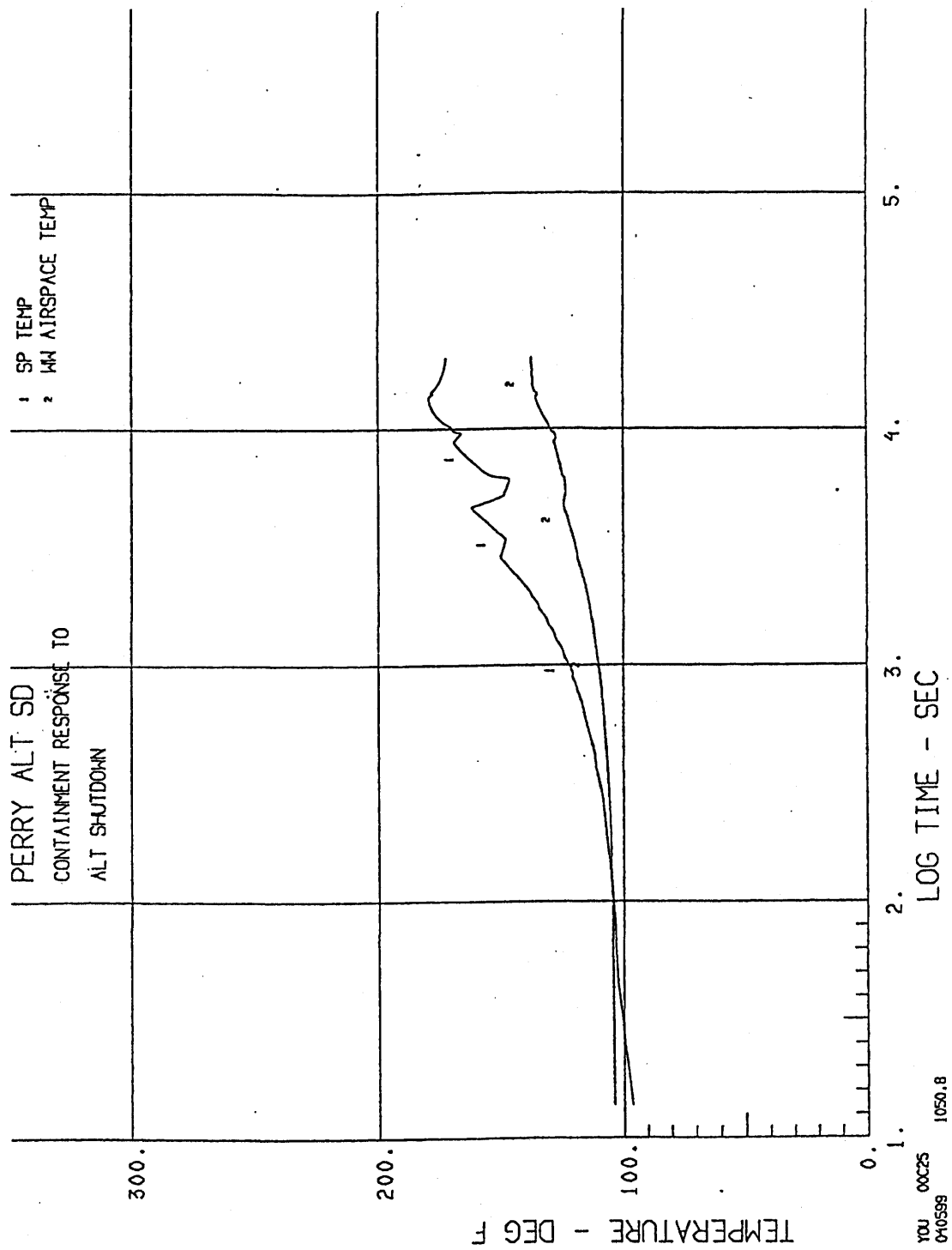
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PERRY NUCLEAR POWER PLANT

Suppression Pool Temperature
During Alternate Shutdown
(at 3729 MWt)

Figure 15.2-17



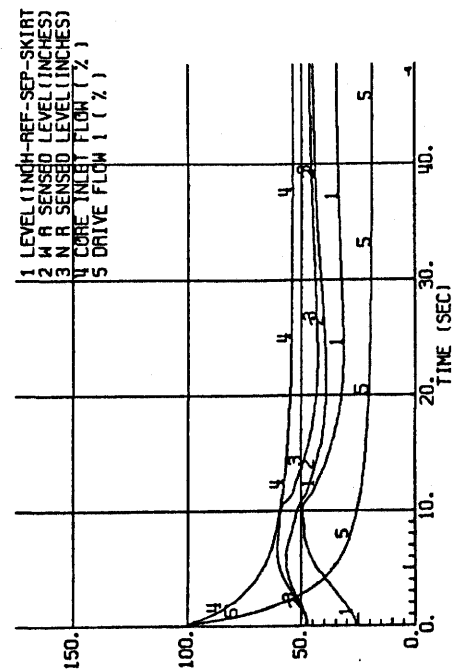
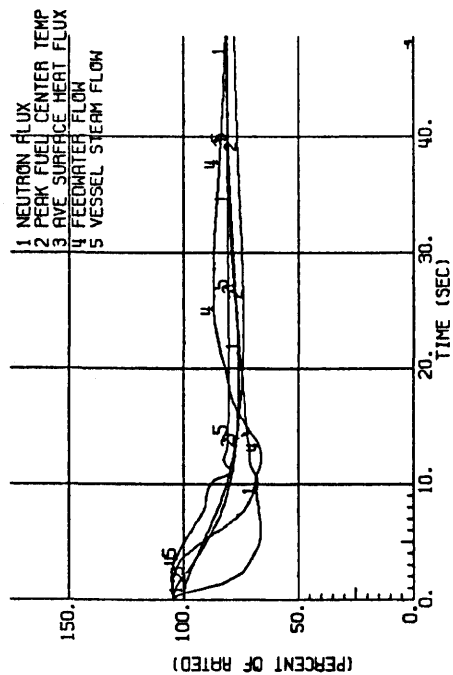
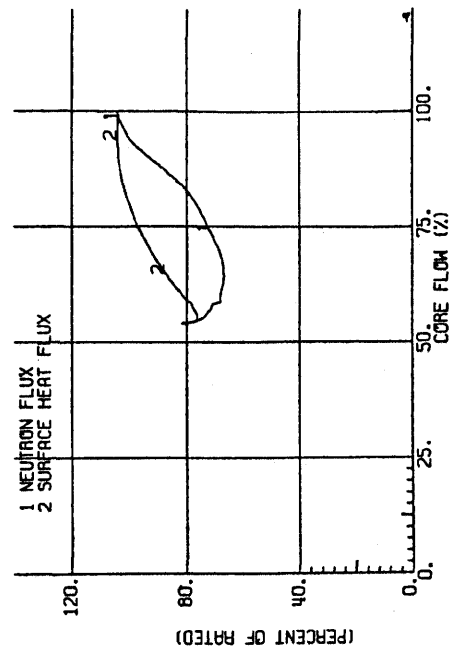
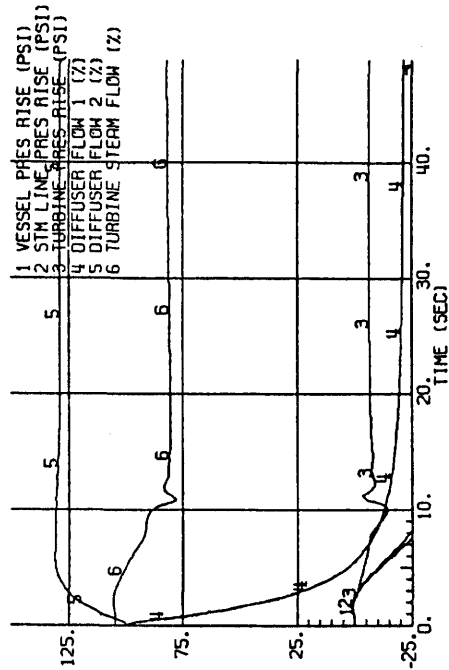
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PERRY NUCLEAR POWER PLANT

Suppression Pool Temperature
During Alternate Shutdown
(at 3833 MWt)

Figure 15.2-17a



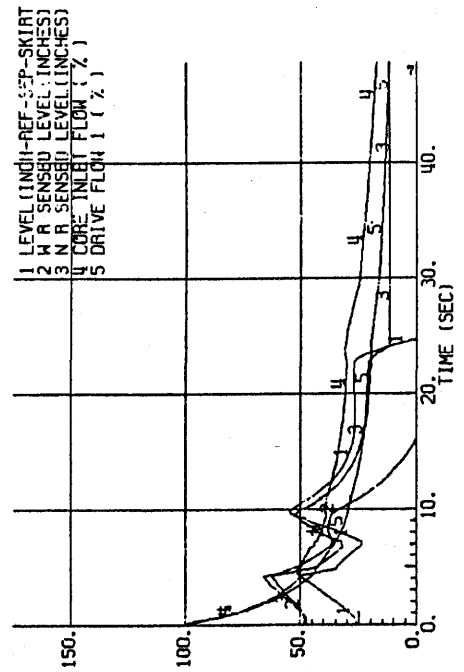
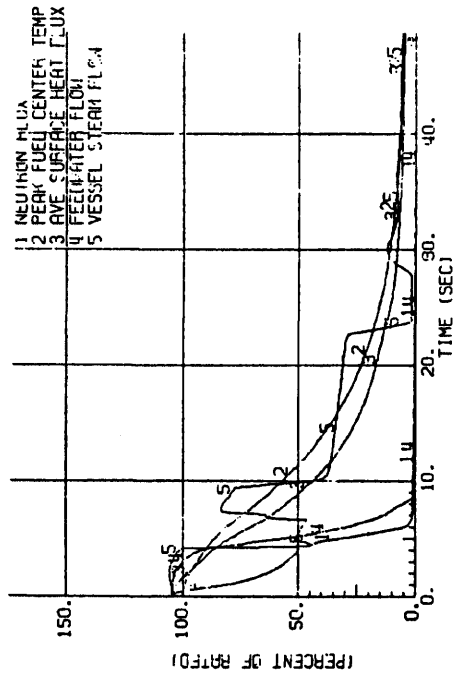
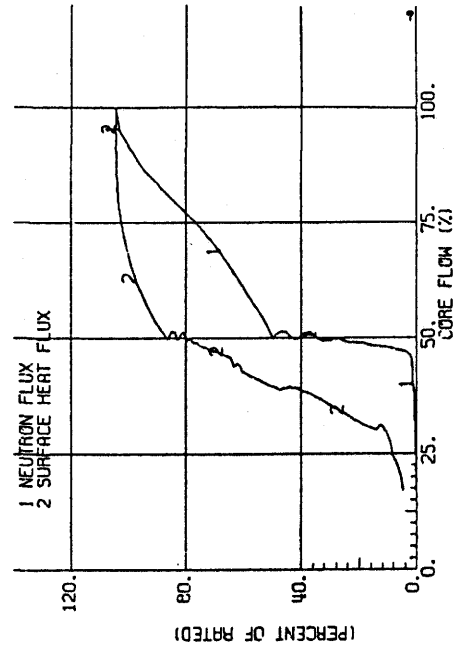
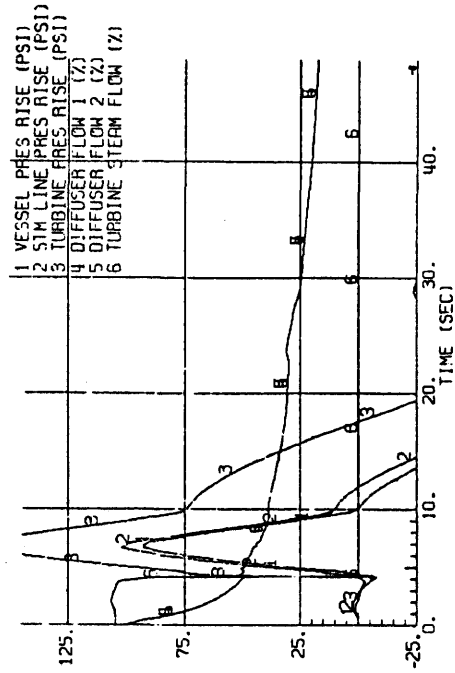
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PERRY NUCLEAR POWER PLANT

Trip of One Recirculation
Pump Motor

Figure 15.3-1



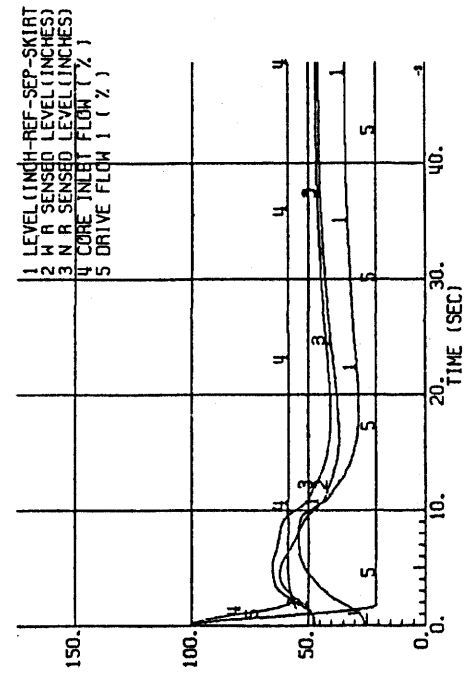
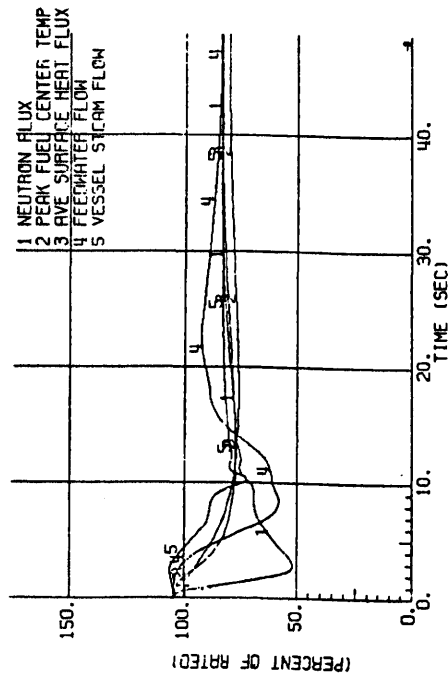
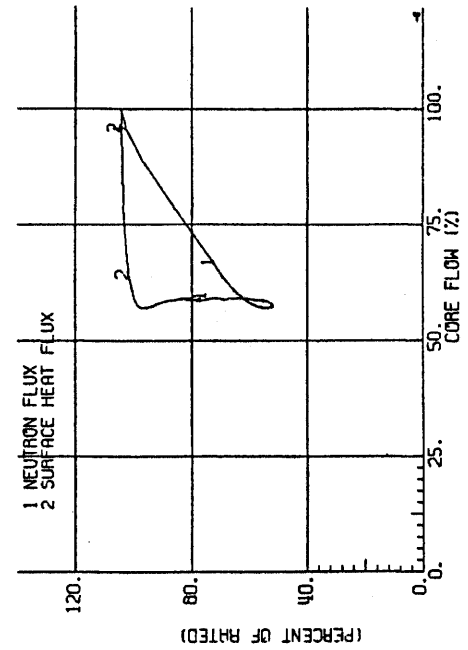
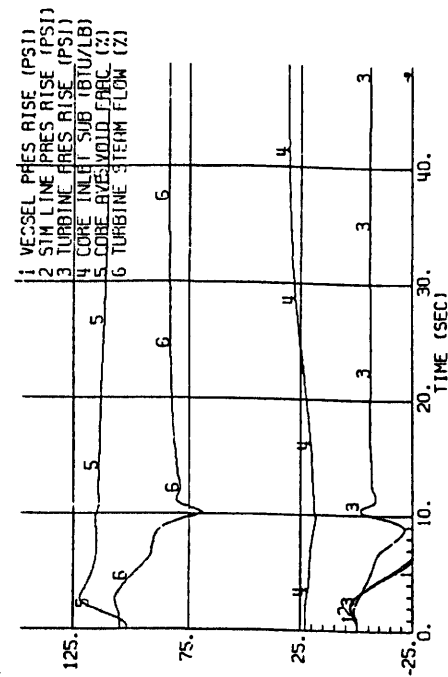
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PERRY NUCLEAR POWER PLANT

Trip of Both Recirculation
Pump Motors

Figure 15.3-2



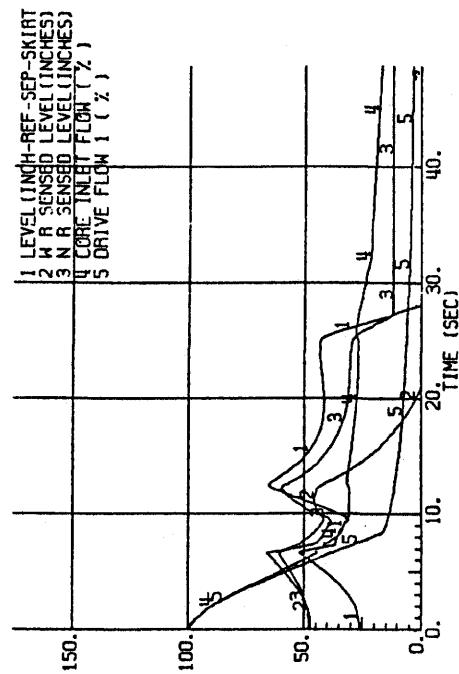
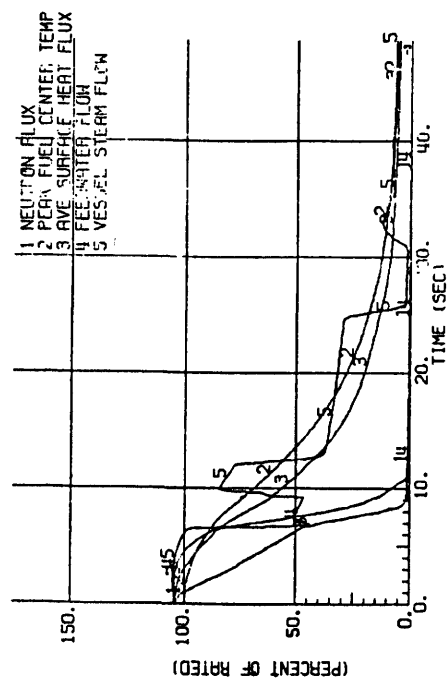
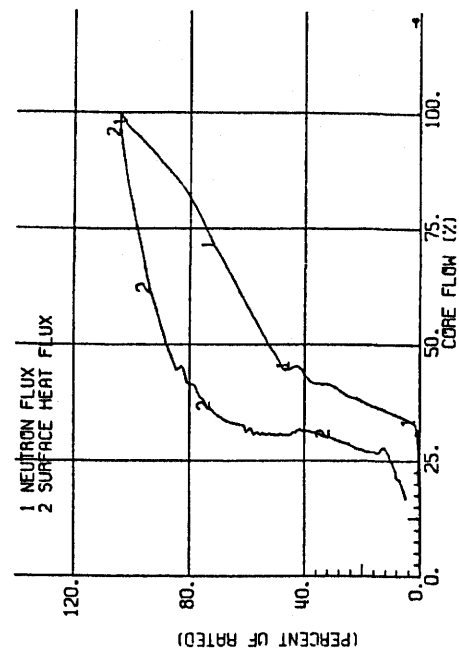
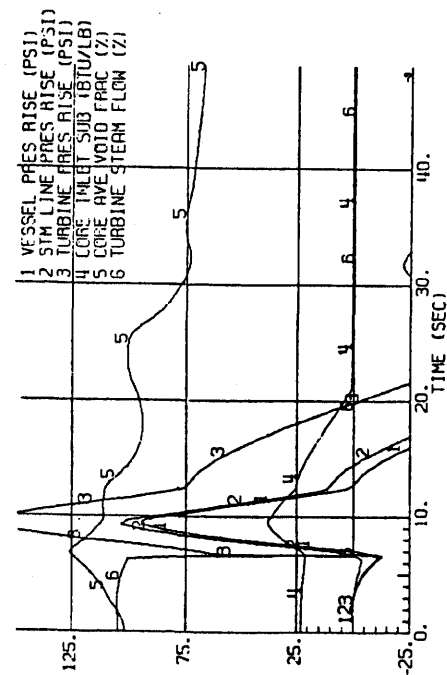
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PERRY NUCLEAR POWER PLANT

Fast Closure of One Main
Recirculation Valve at
60% Per Second

Figure 15.3-3



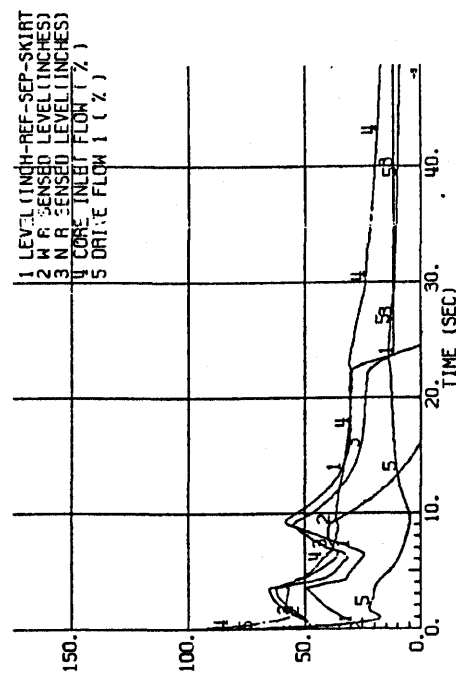
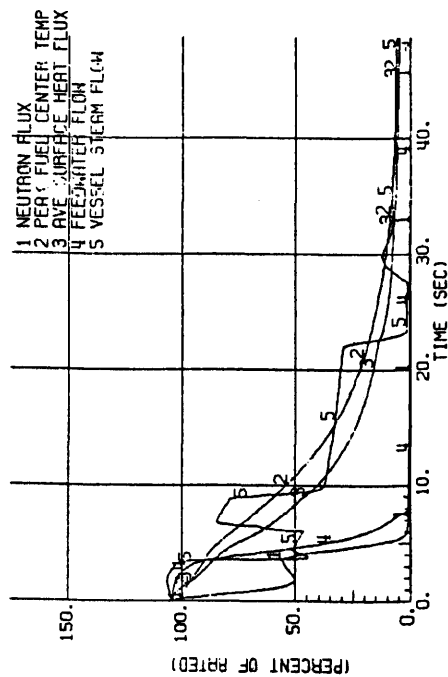
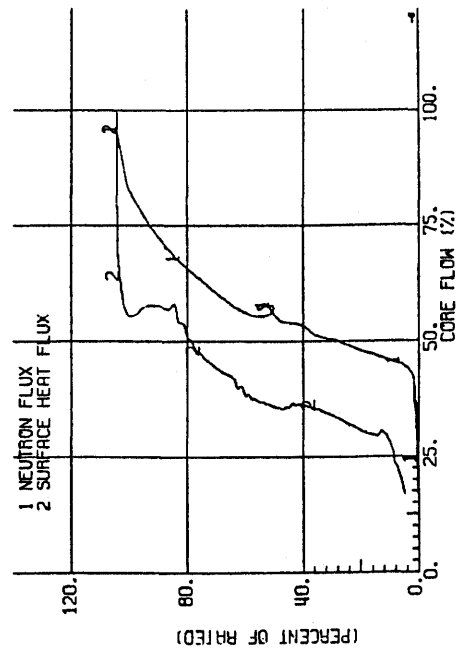
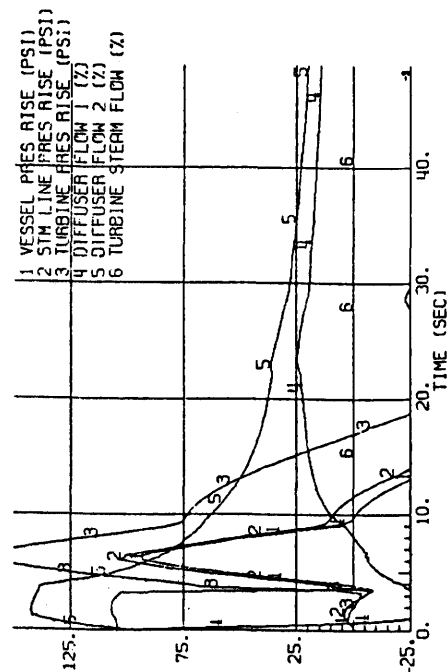
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PERRY NUCLEAR POWER PLANT

Fast Closure of Both Main
Recirculation Valves at
11% Per Second

Figure 15.3-4



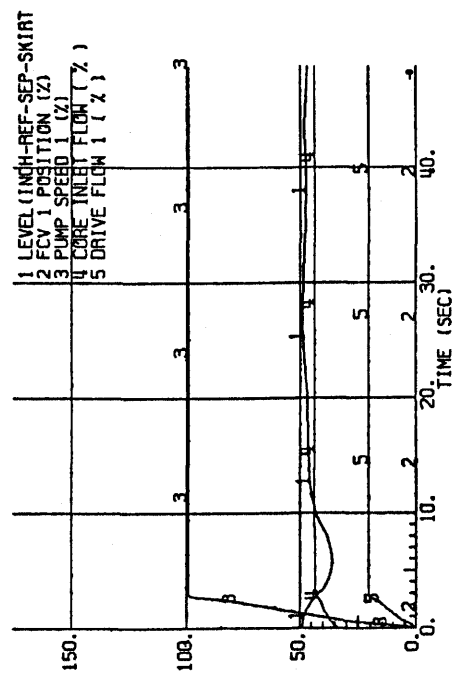
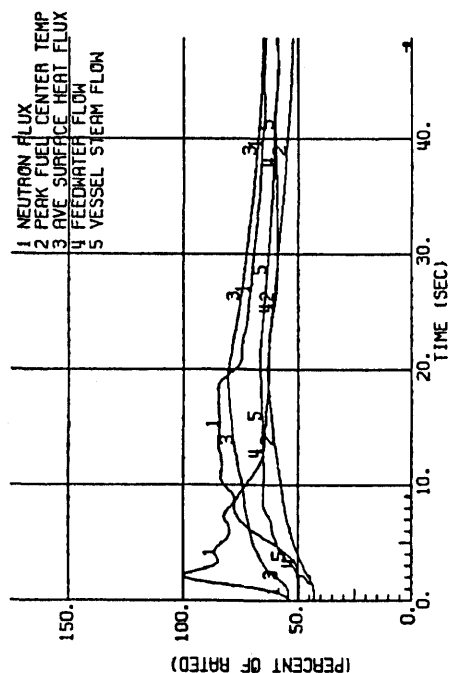
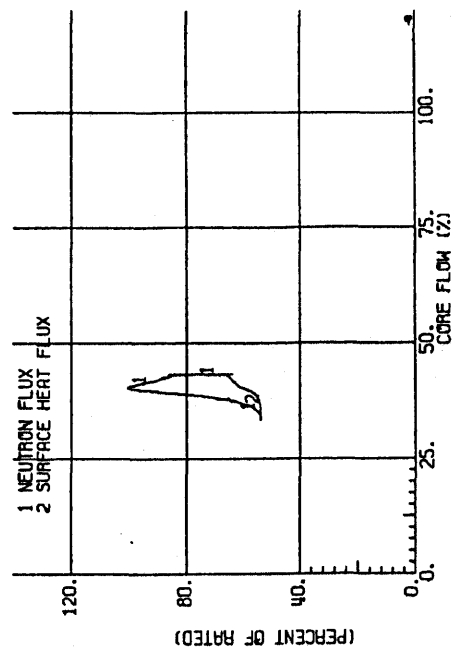
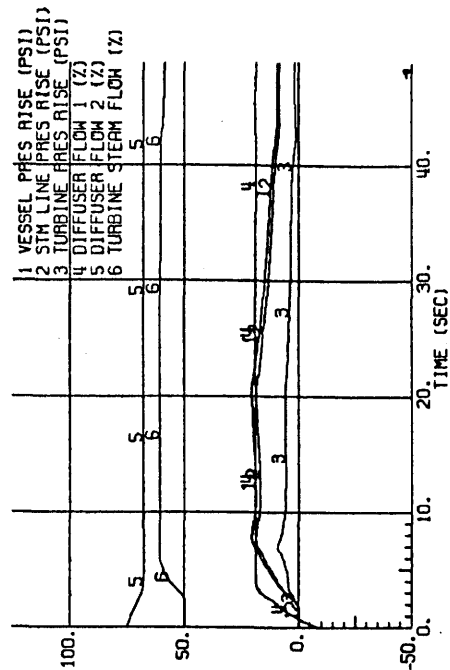
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PERRY NUCLEAR POWER PLANT

Seizure of One Recirculation Pump

Figure 15.3-5



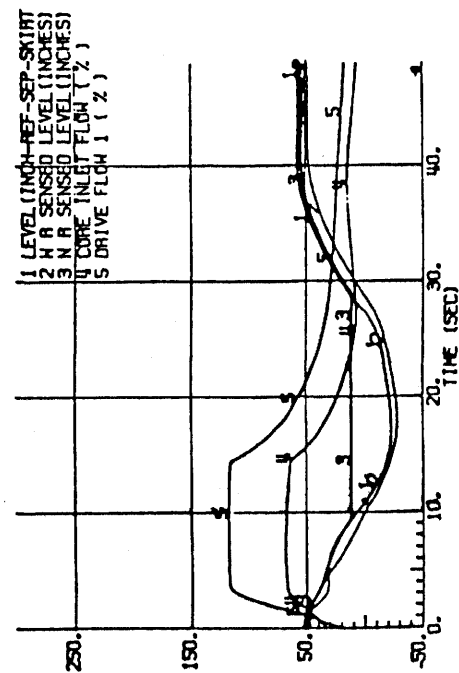
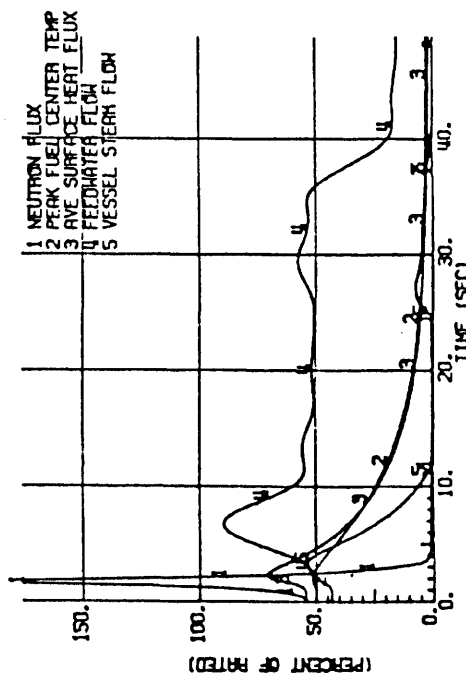
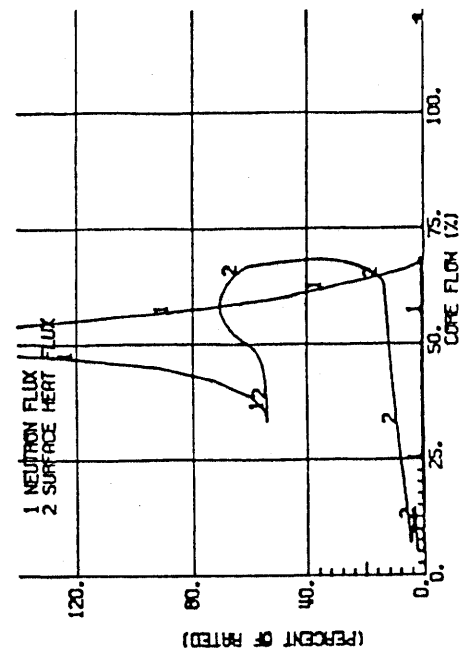
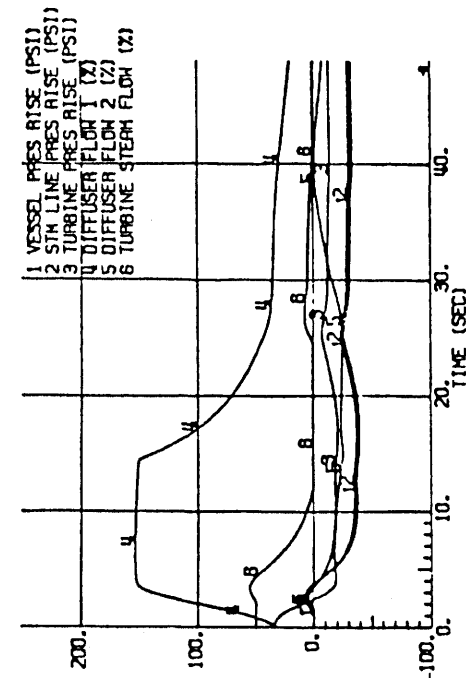
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PERRY NUCLEAR POWER PLANT

Abnormal Startup of Idle
Recirculation Loop Pump

Figure 15.4-1



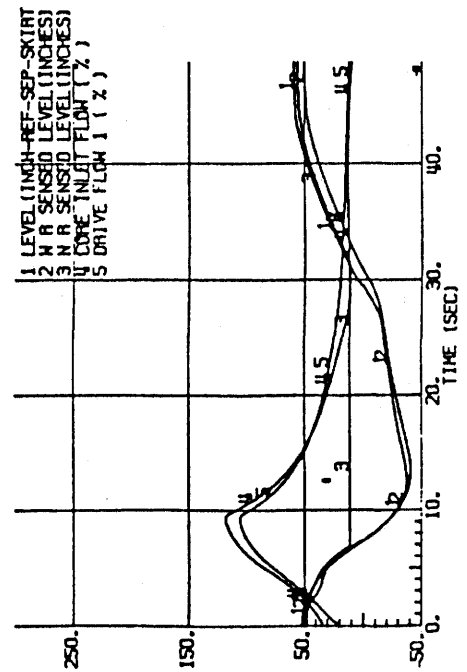
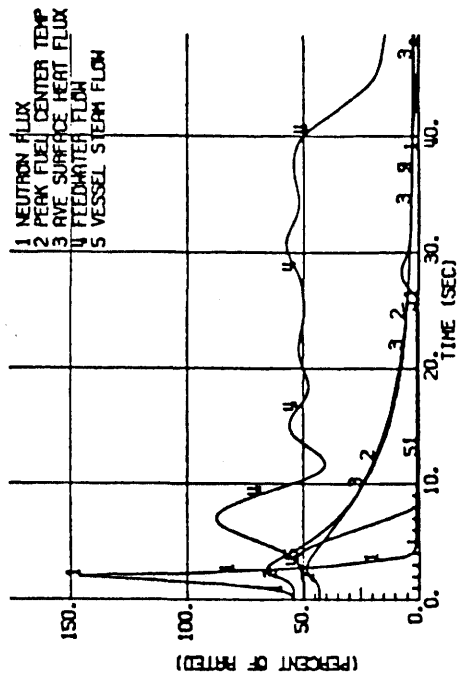
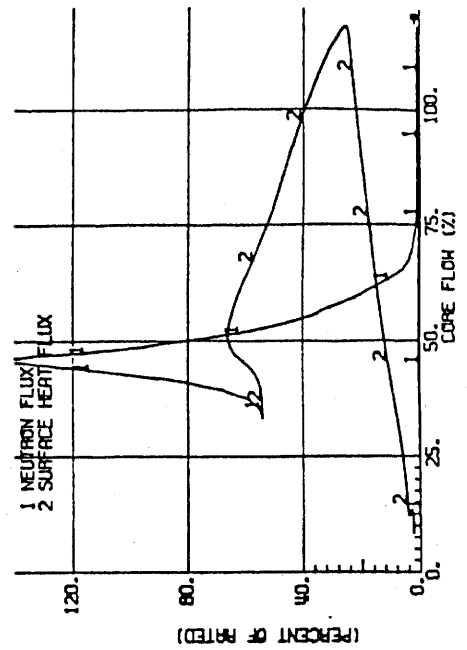
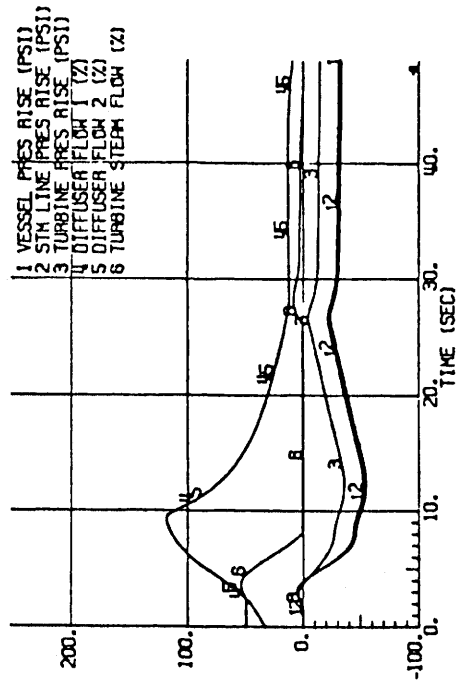
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PERRY NUCLEAR POWER PLANT

Fast Opening of One Main
Recirculation Loop Valve at
30% Per Second

Figure 15.4-2



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PERRY NUCLEAR POWER PLANT

Fast Opening of Both Main
Recirculation Loop Valves at
11% Per Second

Figure 15.4-3

AXIAL AVERAGE EXPOSURES		CONTROL ROD CONFIGURATION	
AXIAL K	C U R R E N T MWD/ST	TARGET MWD/ST	IN NOTCHES WITHDRAWN
25	0.1741 4480.6	5008.1	1 3 5 7 9 11 13 15 17 19 21 23 25 27 29
24	0.3266 7092.0	7809.5	59
23	0.7259 15858.8	17394.8	1
22	0.8882 19776.8	21617.6	3
21	1.0141 22992.4	25092.5	5
20	1.0944 25209.3	27461.2	5
19	1.1346 26819.6	29119.5	40
18	1.1423 28005.1	30370.2	40
17	1.1012 28727.6	31132.9	7
16	1.1266 26580.4	28843.6	7
15	1.1038 26989.6	29371.4	9
14	1.1010 27490.9	30094.0	40 14 40
13	1.1017 27930.5	30753.6	11
12	1.1079 28299.4	31334.5	13
11	1.1208 28563.5	31824.8	14
10	1.1388 28773.1	32155.0	6 14
9	1.1640 28828.6	32252.1	15
8	1.1904 29077.1	32591.1	17
7	1.2152 29408.2	32982.3	14
6	1.2366 29656.9	33252.8	19
5	1.2458 29635.0	33195.8	19
4	1.2206 28720.1	32225.5	21
3	1.1549 25875.0	29301.1	40 14 40
2	0.9177 19691.1	22552.2	23
1	0.2528 6033.2	6949.1	25
			27
			29
			2
			6 10 14 18 22 26 30 34 38 42 46 50 54 58

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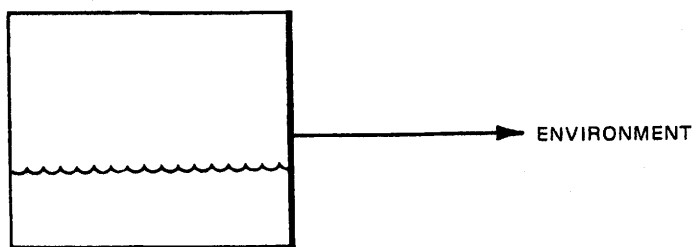


PERRY NUCLEAR POWER PLANT

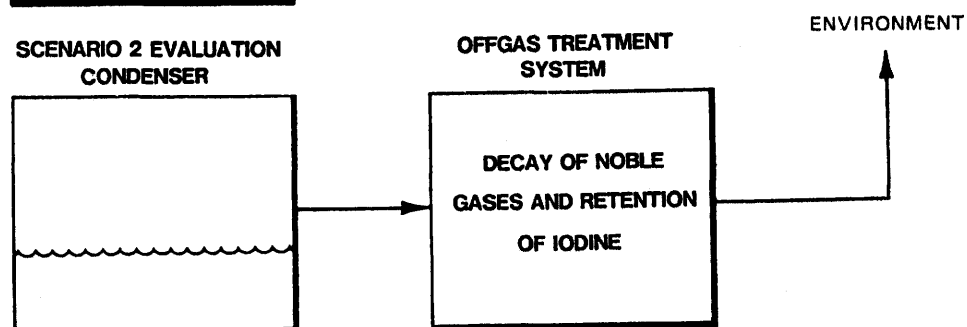
Critical Rod Pattern for
Misplaced Bundle Accident

Figure 15.4-4

1. **SCENARIO 1 EVALUATION
CONDENSER**



2. **SCENARIO 2 EVALUATION
CONDENSER**



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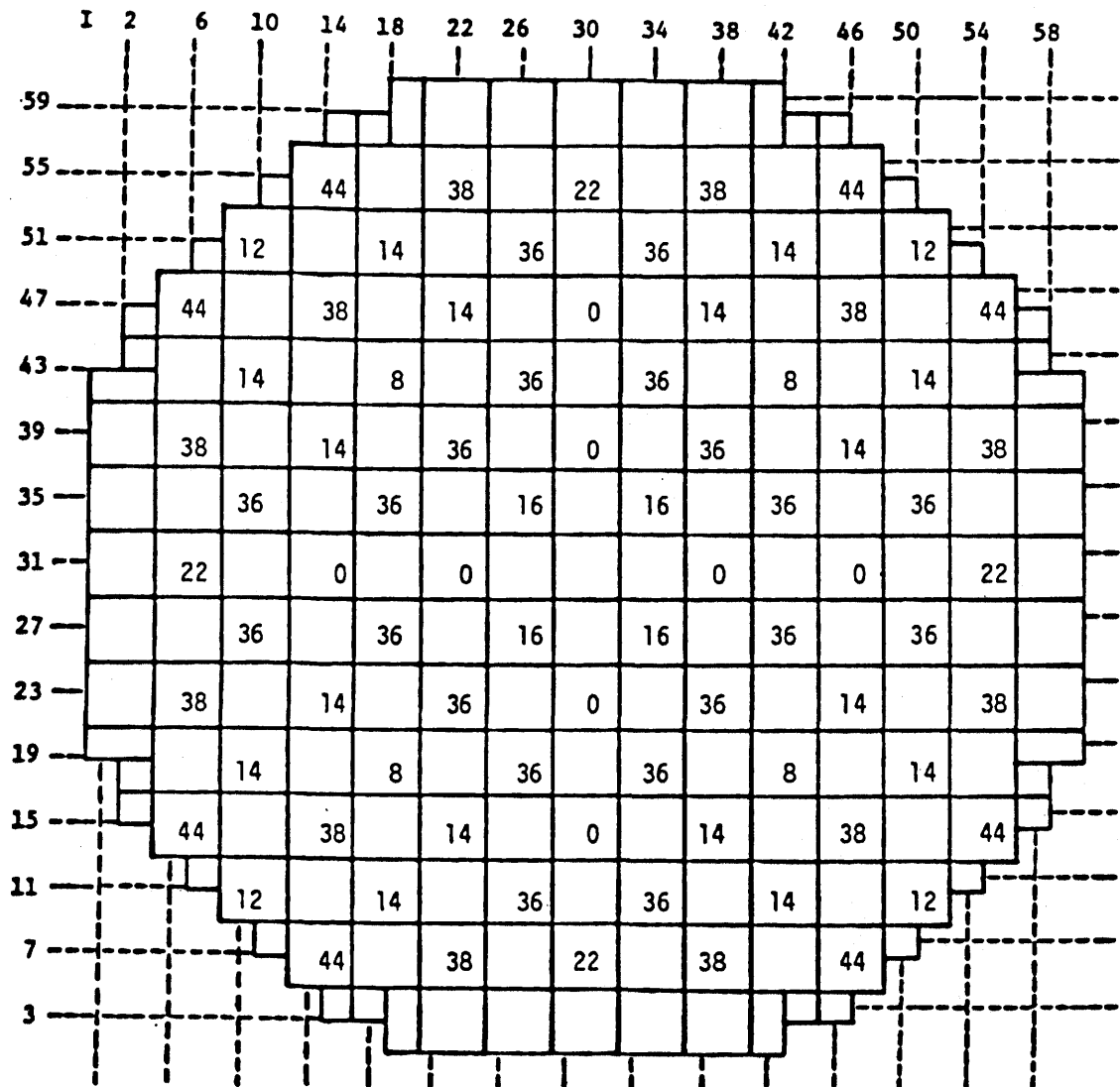


PERRY NUCLEAR POWER PLANT

Leakage Path Model for Rod
Drop Accident

Figure 15.4-5

PLANT
COORDINATE



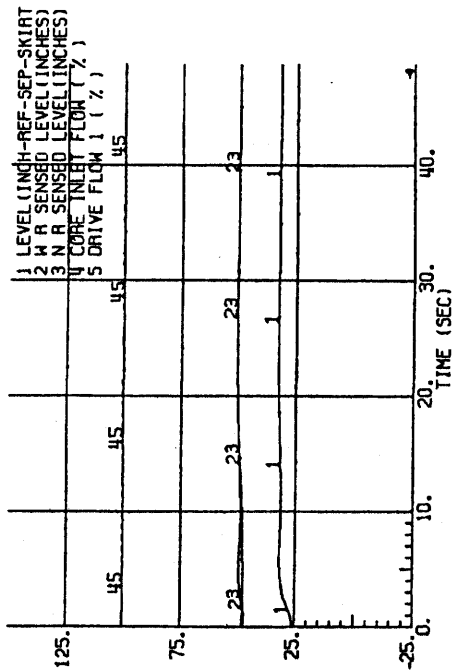
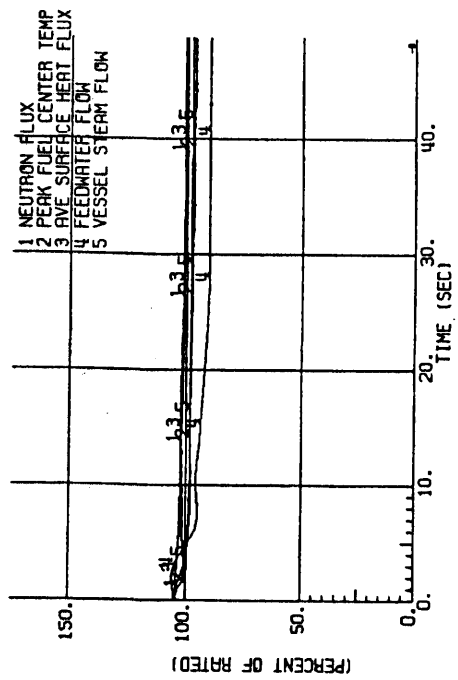
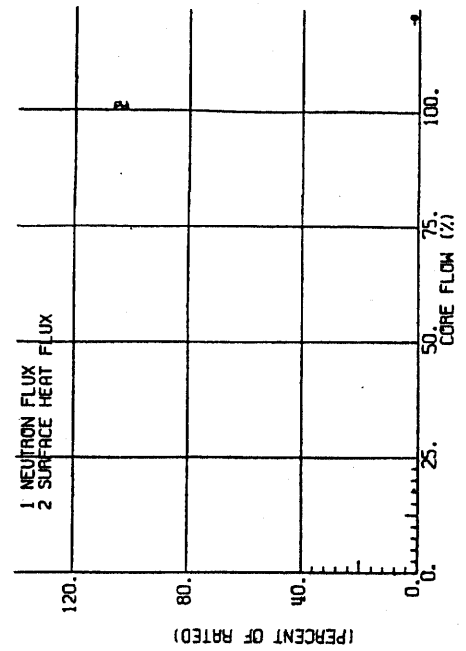
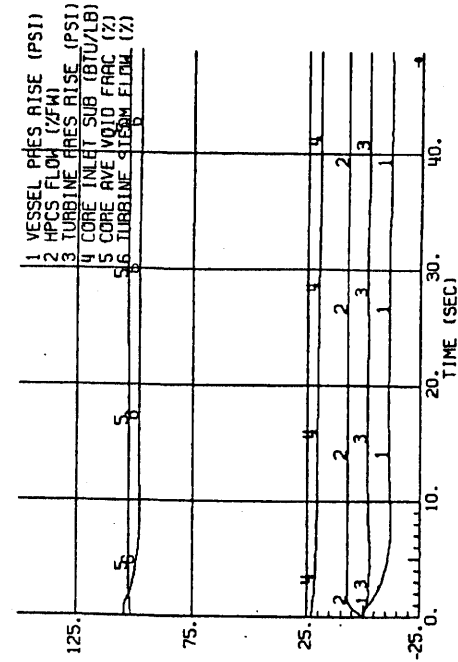
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PERRY NUCLEAR POWER PLANT

Rod Pattern for RWE Analysis $\frac{1}{4}$
Core Geometry (Rated Power Case)

Figure 15.4-6



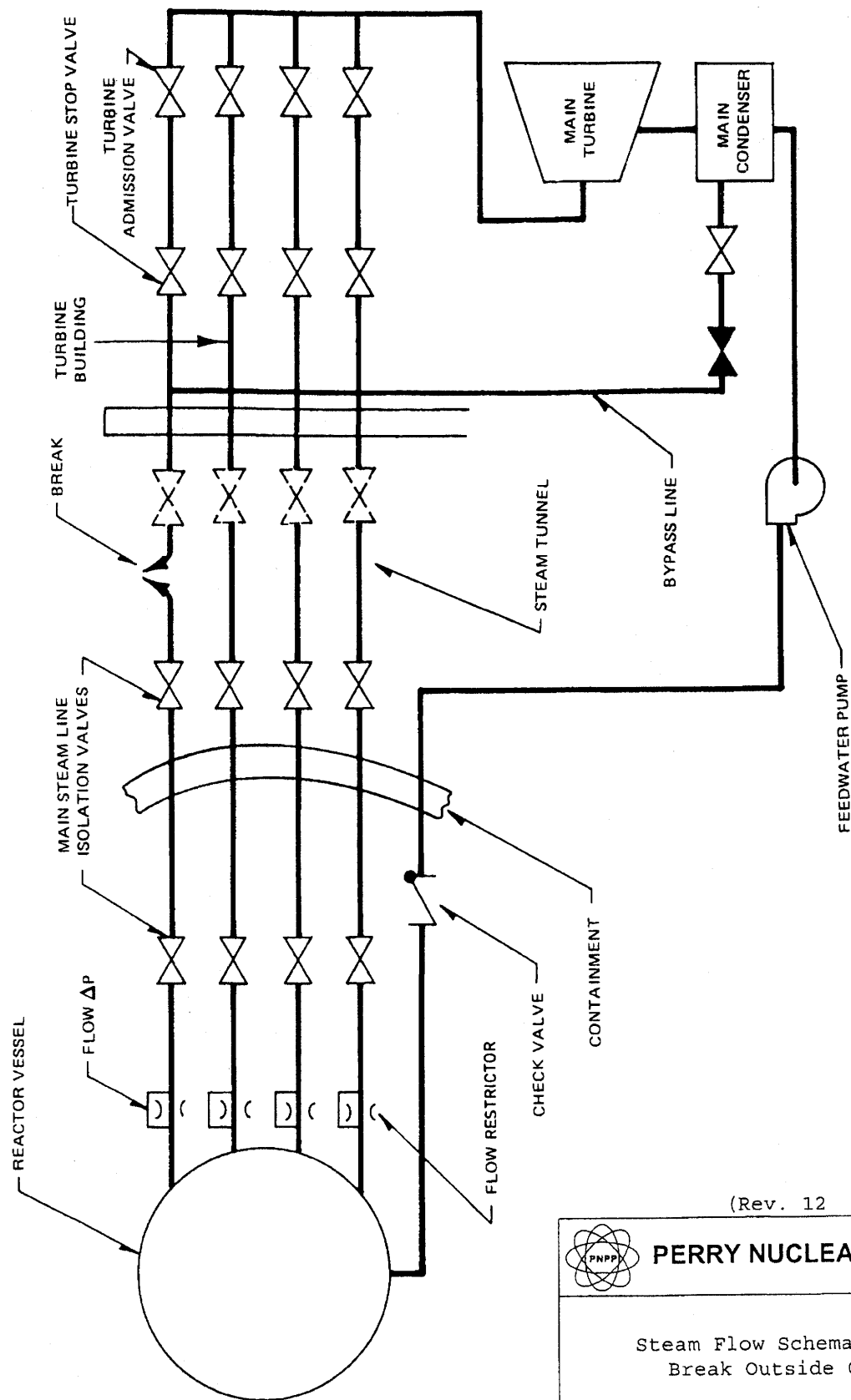
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PERRY NUCLEAR POWER PLANT

Inadvertent Startup of HPCS

Figure 15.5-1



(Rev. 12 1/03)

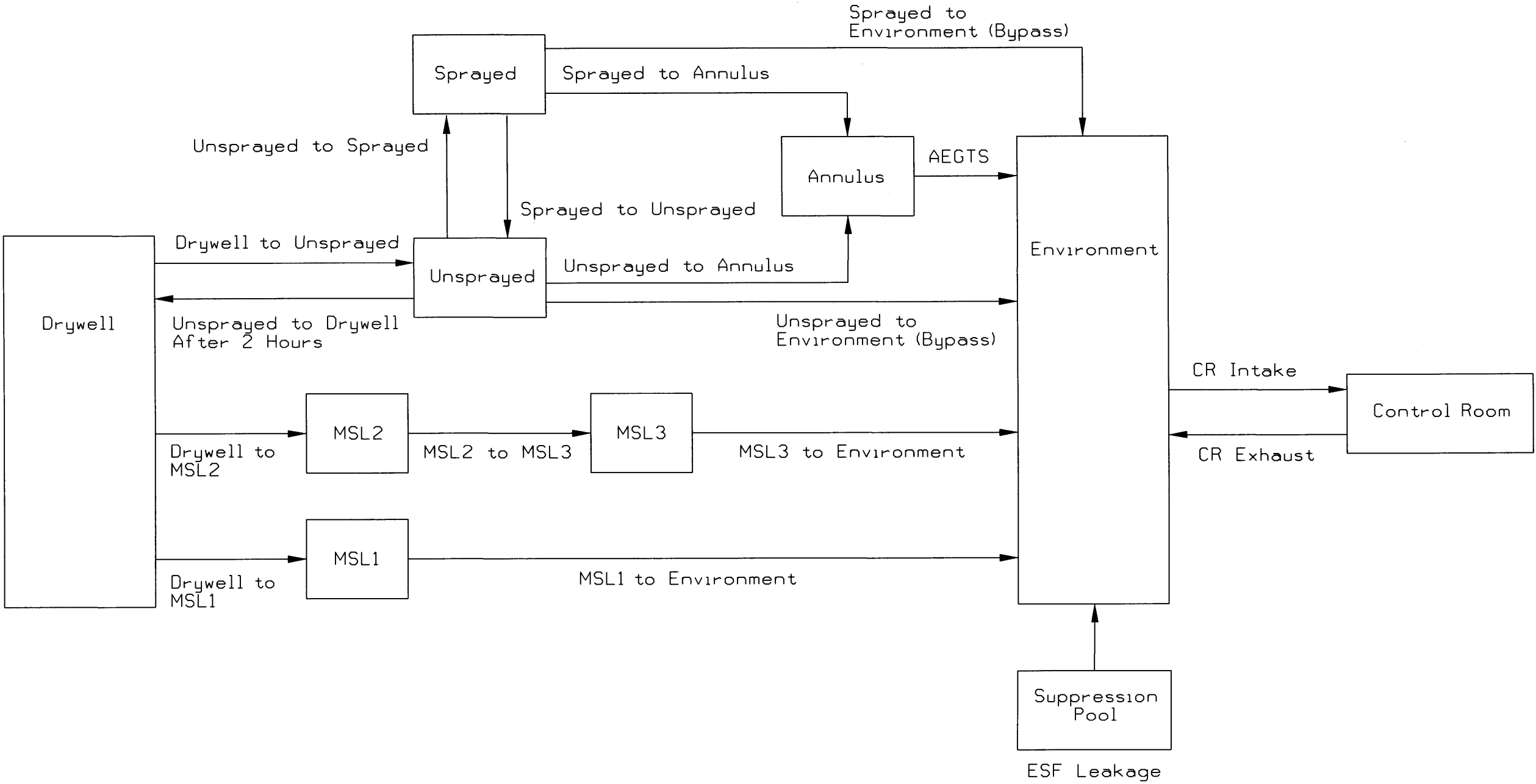


PERRY NUCLEAR POWER PLANT

Steam Flow Schematic For Steam
Break Outside Containment

Figure 15.6-1

Fission Product Transport Model
(MSIV and Containment Leakage Pathways)

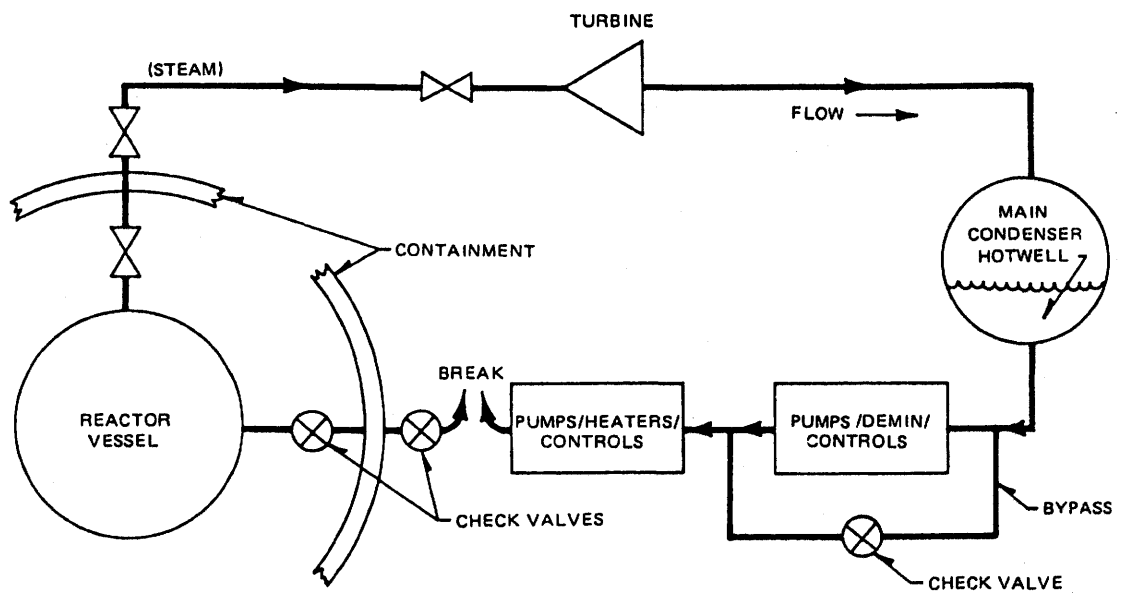


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PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

AIR LEAKAGE
FLOWPATH POST-LOCA

FIGURE 15.6-2



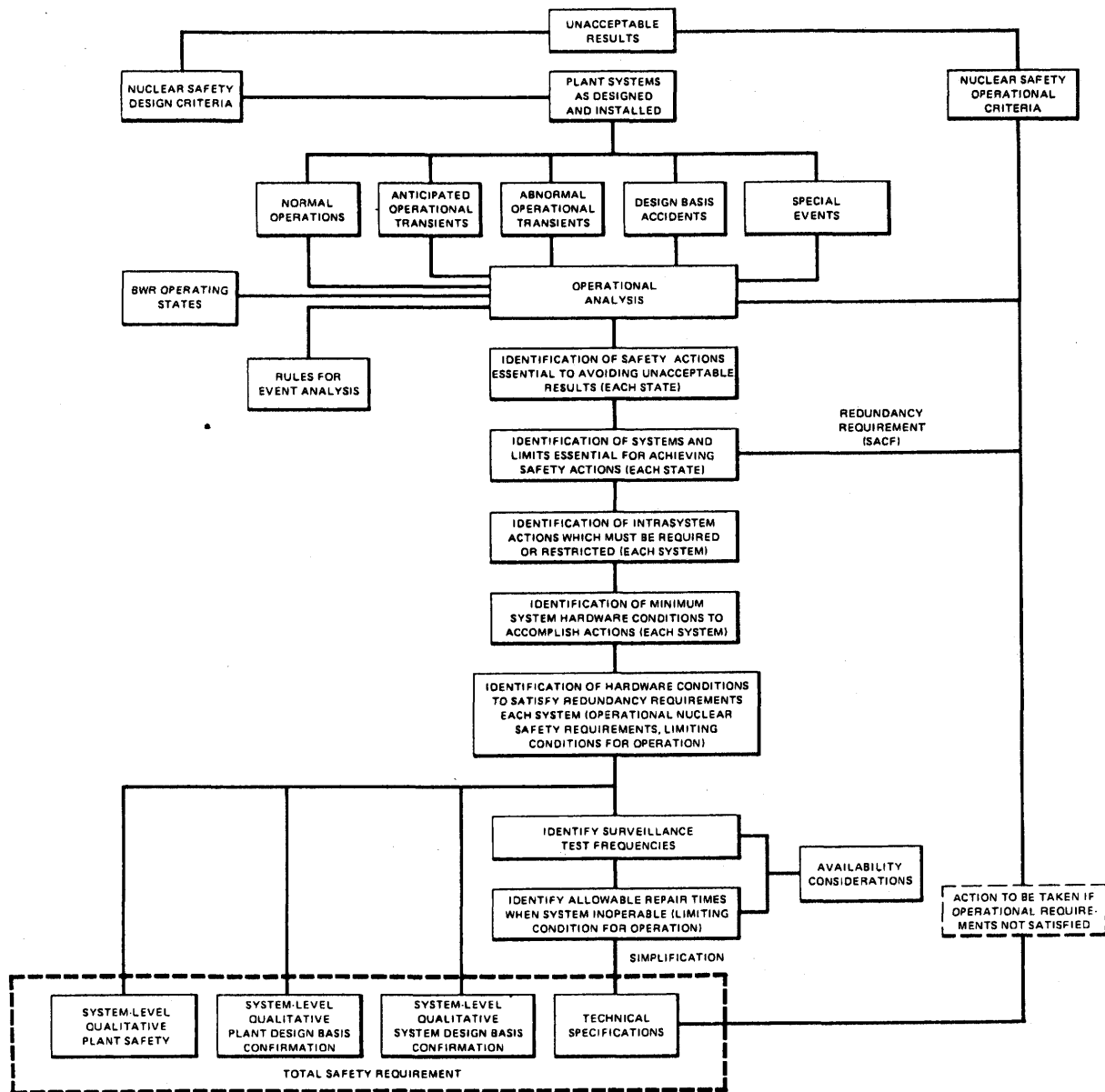
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PERRY NUCLEAR POWER PLANT

Leakage Path for Feedwater Line
Break Outside Containment

Figure 15.6-3

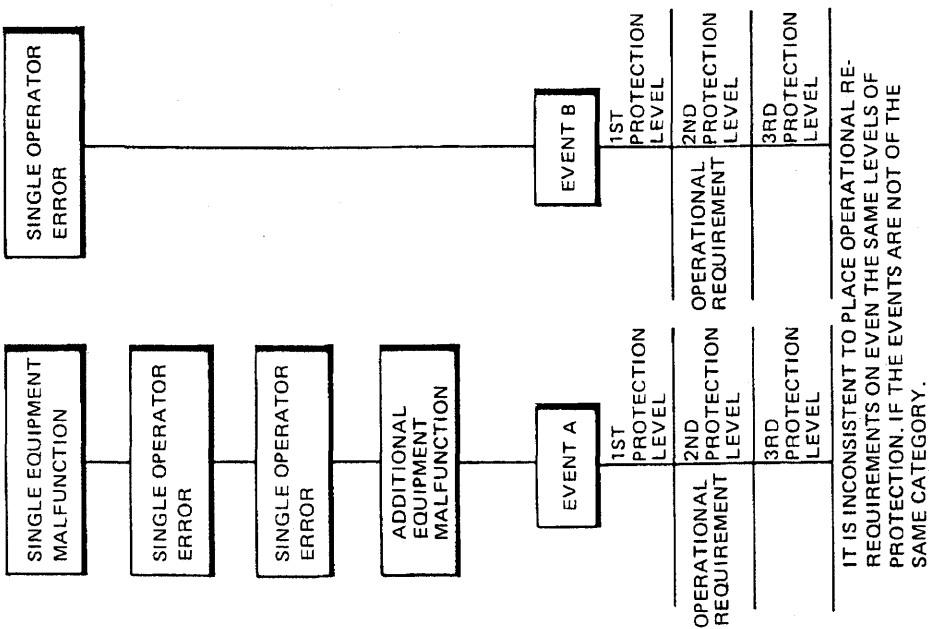


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PERRY NUCLEAR POWER PLANT

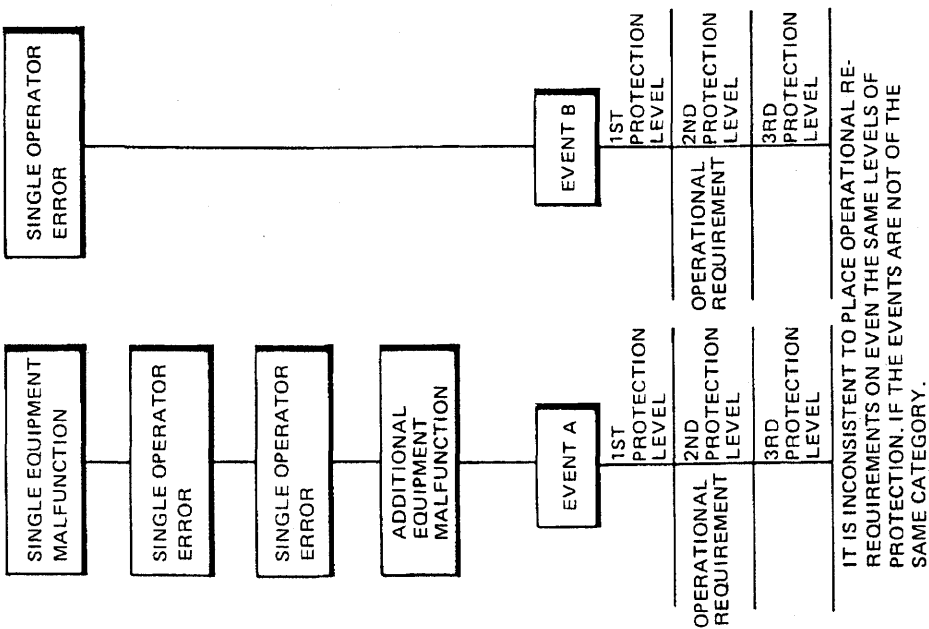
Block Diagram Used to Derive Nuclear
Safety Operational Requirement
System Level Qualitative Design
Basis Confirmation
Audits and Technical Specifications
Figure 15A.2-1



PANEL A

IT IS INCONSISTENT TO PLACE OPERATIONAL REQUIREMENTS ARBITRARILY ON SOME ACTION (SCRAM) IN ALL CASES OF ONE EVENT CATEGORY, BECAUSE THAT ACTION (SCRAM) MAY REPRESENT DIFFERENT LEVELS OF PROTECTION FOR THE VARIOUS CASES.

PANEL B



PANEL C

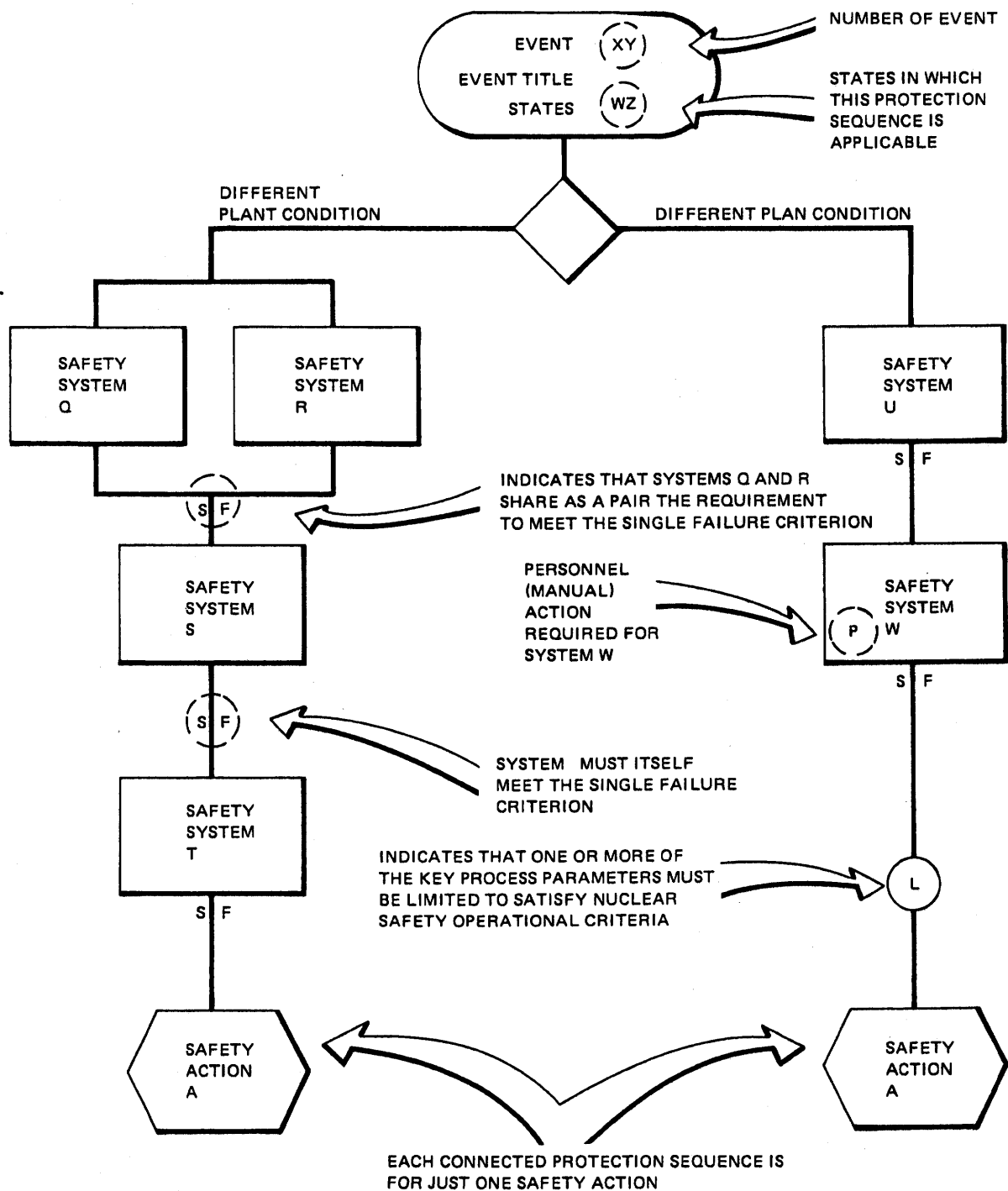
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PERRY NUCLEAR POWER PLANT

Possible Inconsistencies in the
Selection of Nuclear Safety
Operational Requirements

Figure 15A.2-2



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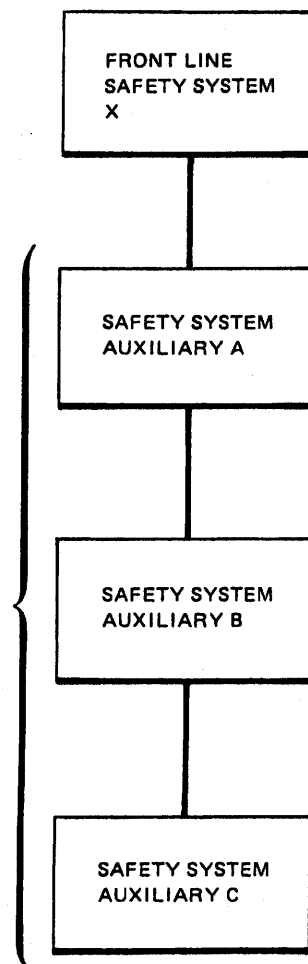


PERRY NUCLEAR POWER PLANT

Format for Protection
 Sequence Diagrams

Figure 15A.4-1

DIAGRAM INDICATES
THAT AUXILIARIES
A, B, AND C ARE
ESSENTIAL TO THE
OPERATION OF
THE FRONT LINE
SAFETY SYSTEM X.
NO CHRONOLOGY
OR ORDER OF
ACTION IS IMPLIED



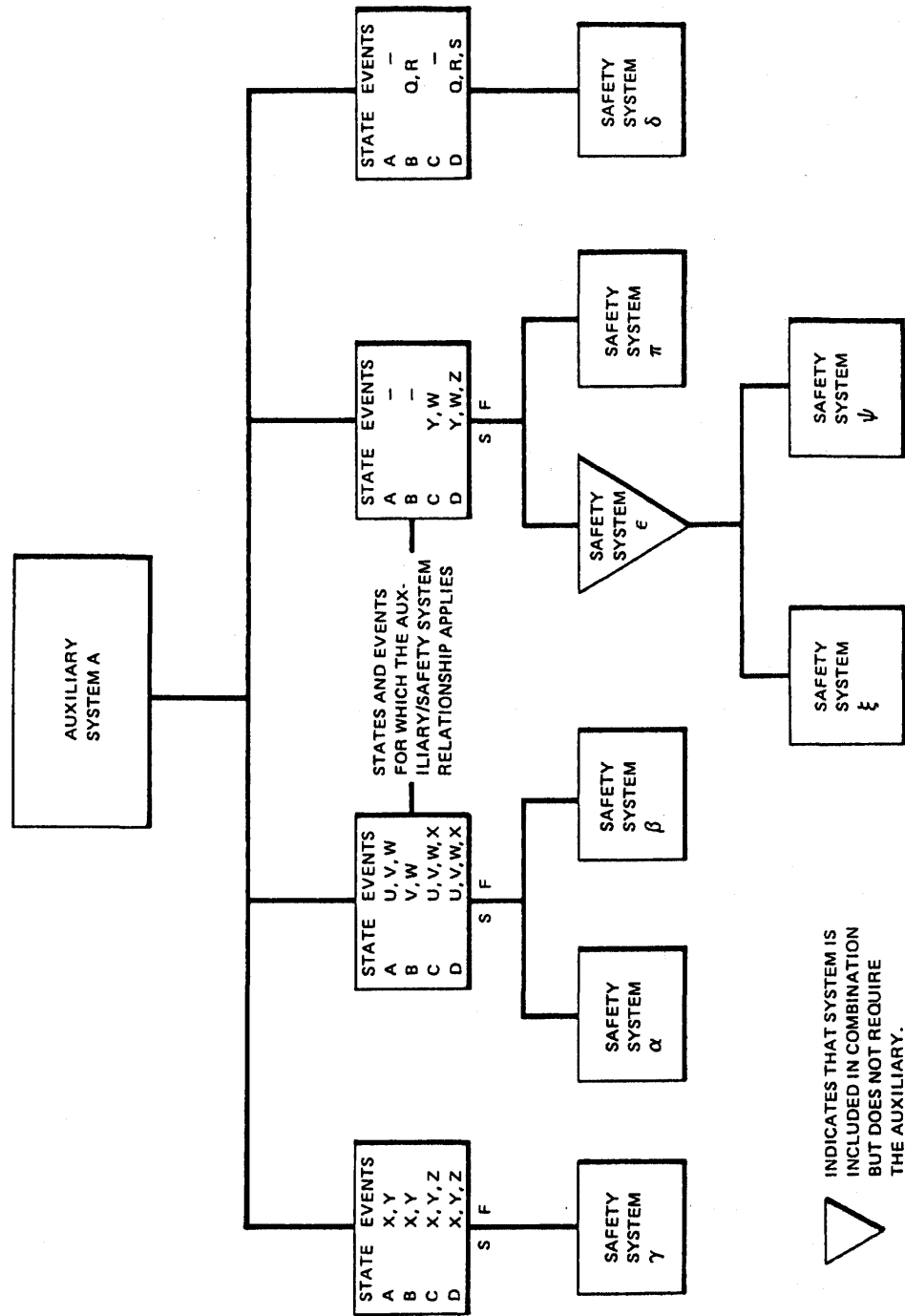
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PERRY NUCLEAR POWER PLANT

Format for Safety System
Auxiliary Diagrams

Figure 15A.4-2



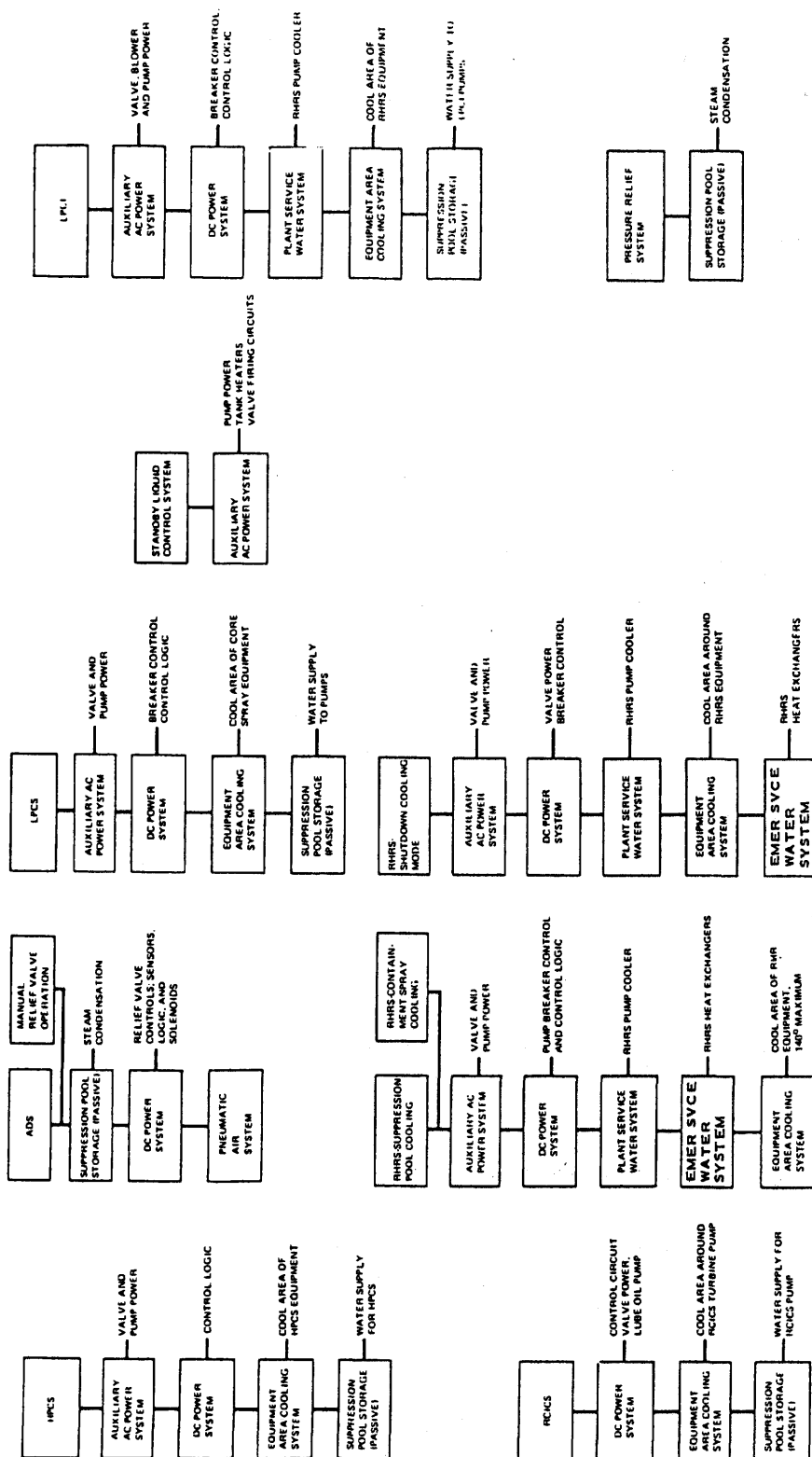
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PERRY NUCLEAR POWER PLANT

Format for Commonality of
Auxiliary Diagrams

Figure 15A.4-3

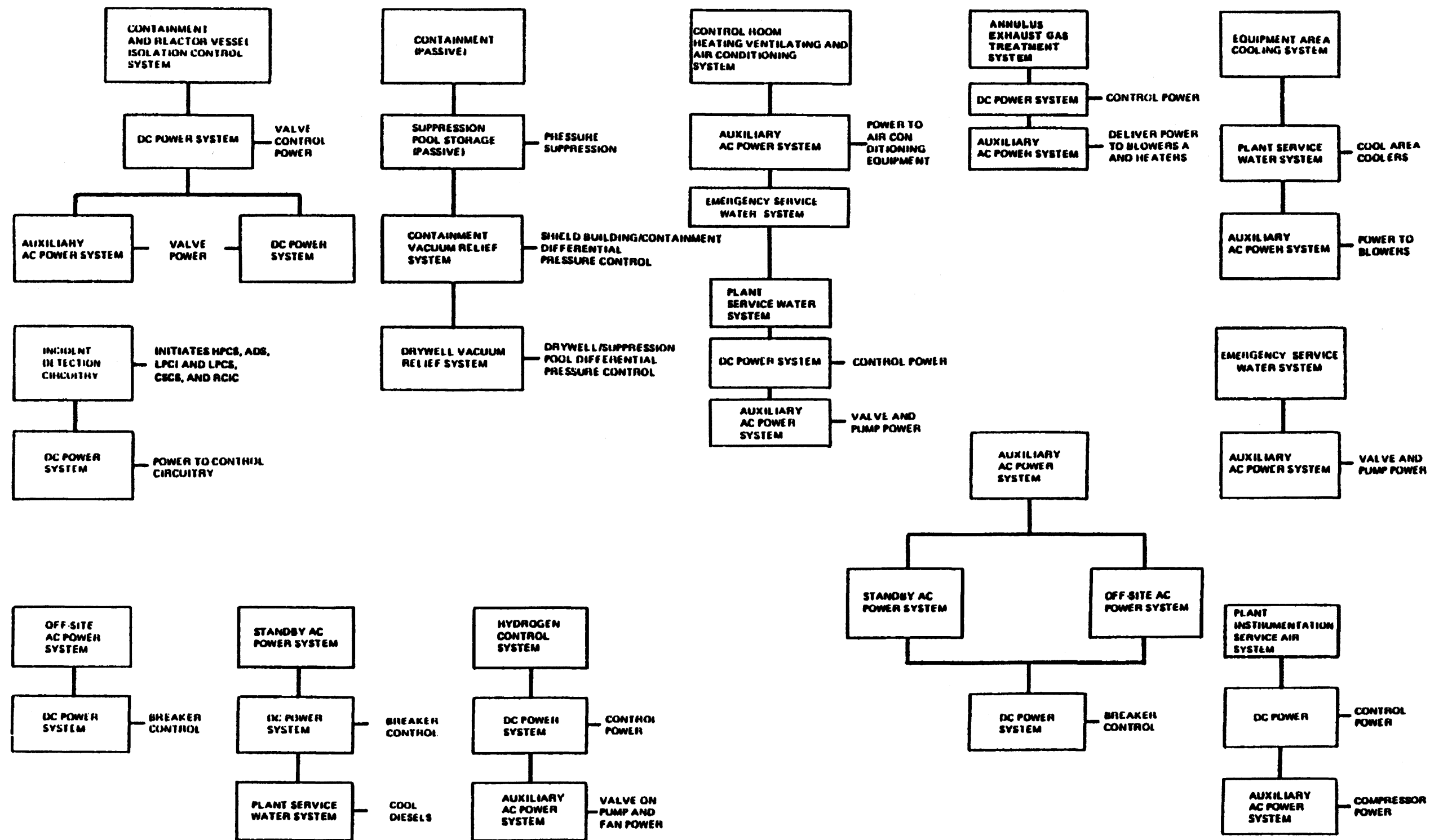


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PERRY NUCLEAR POWER PLANT

Safety System Auxiliaries

Figure 15A.6-1

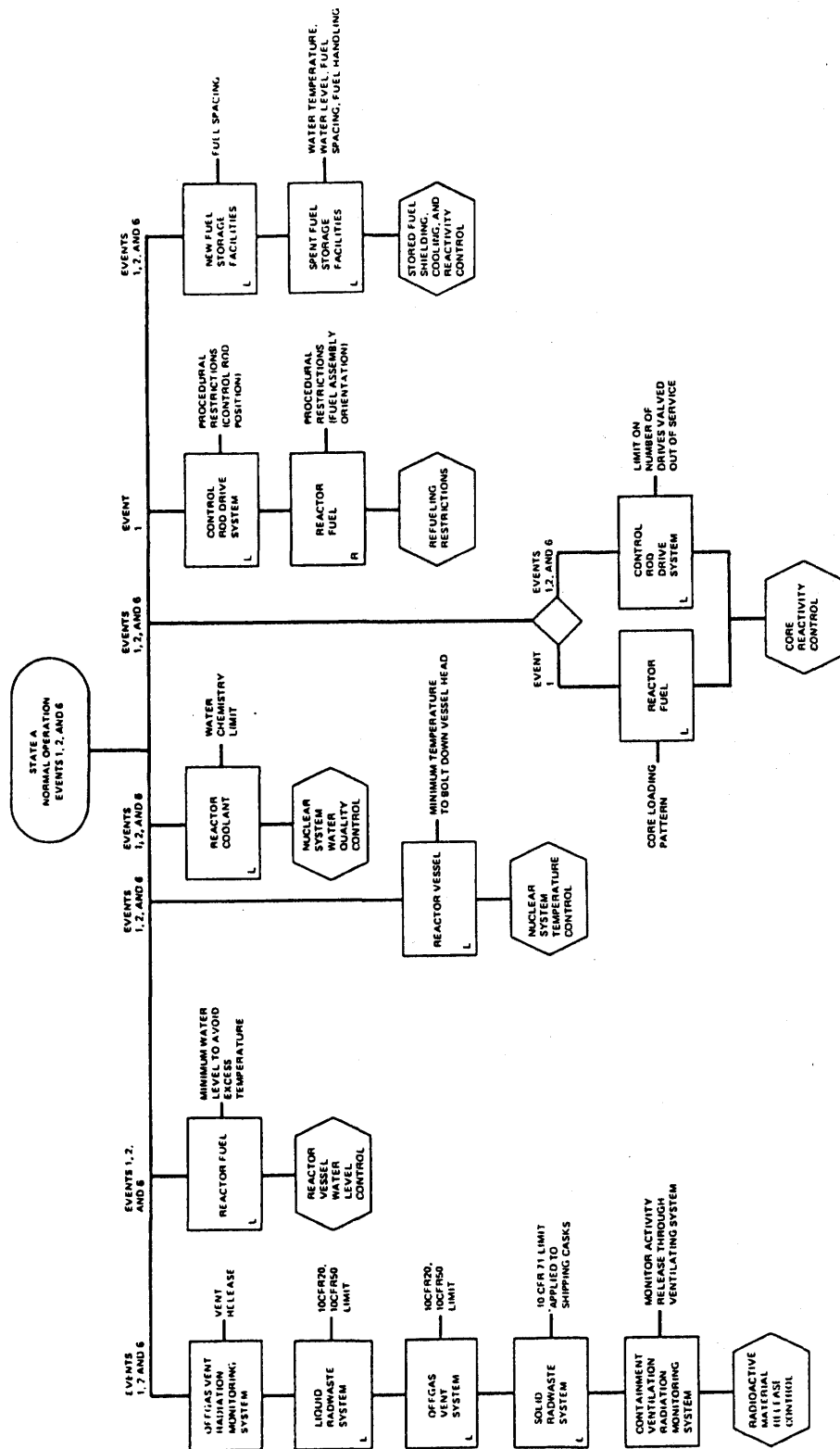


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PERRY NUCLEAR POWER PLANT
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SAFETY SYSTEM AUXILIARIES

FIGURE 15A.6-2



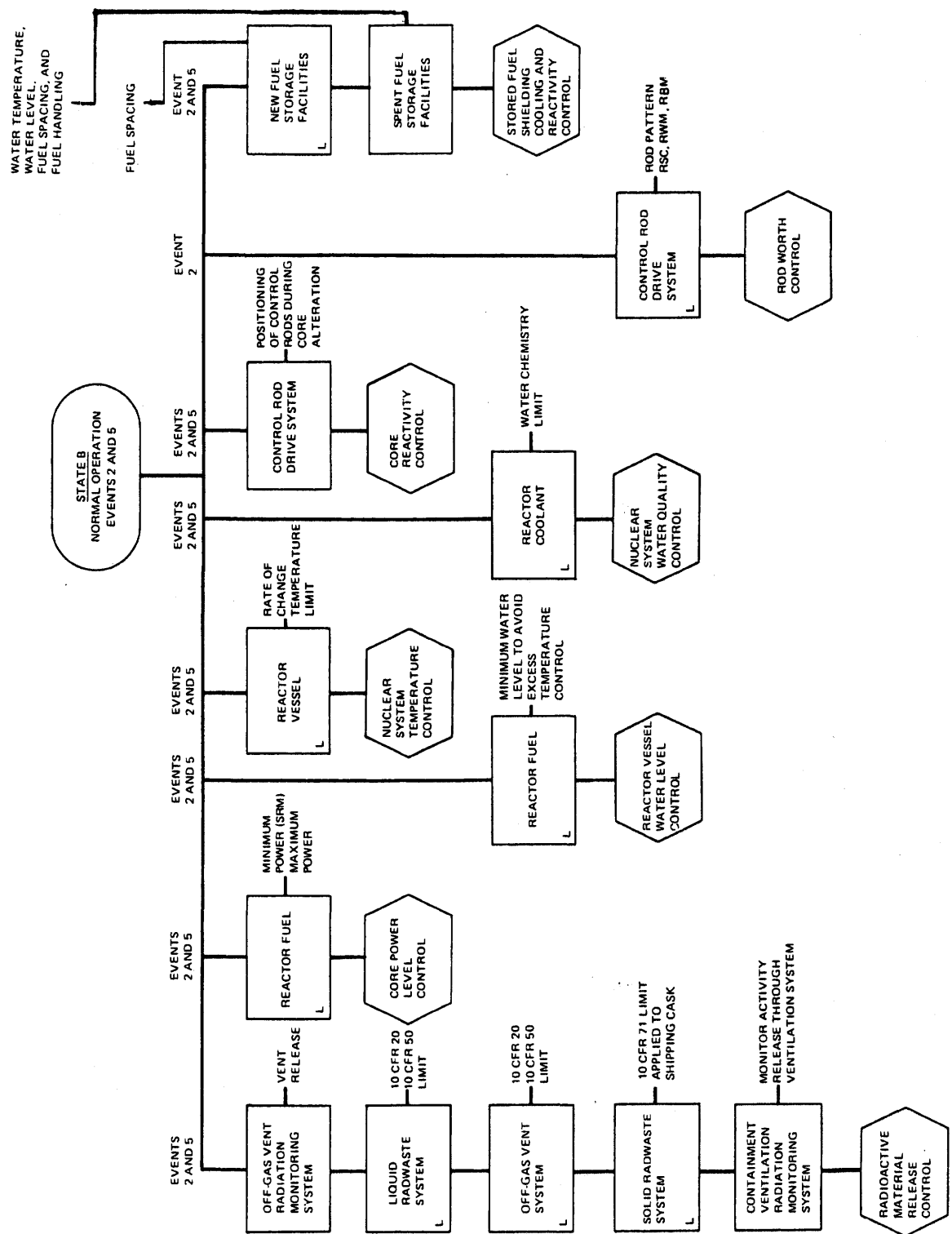
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PERRY NUCLEAR POWER PLANT

Safety Action Sequences for
Normal Operation in State A.

Figure 15A.6-3



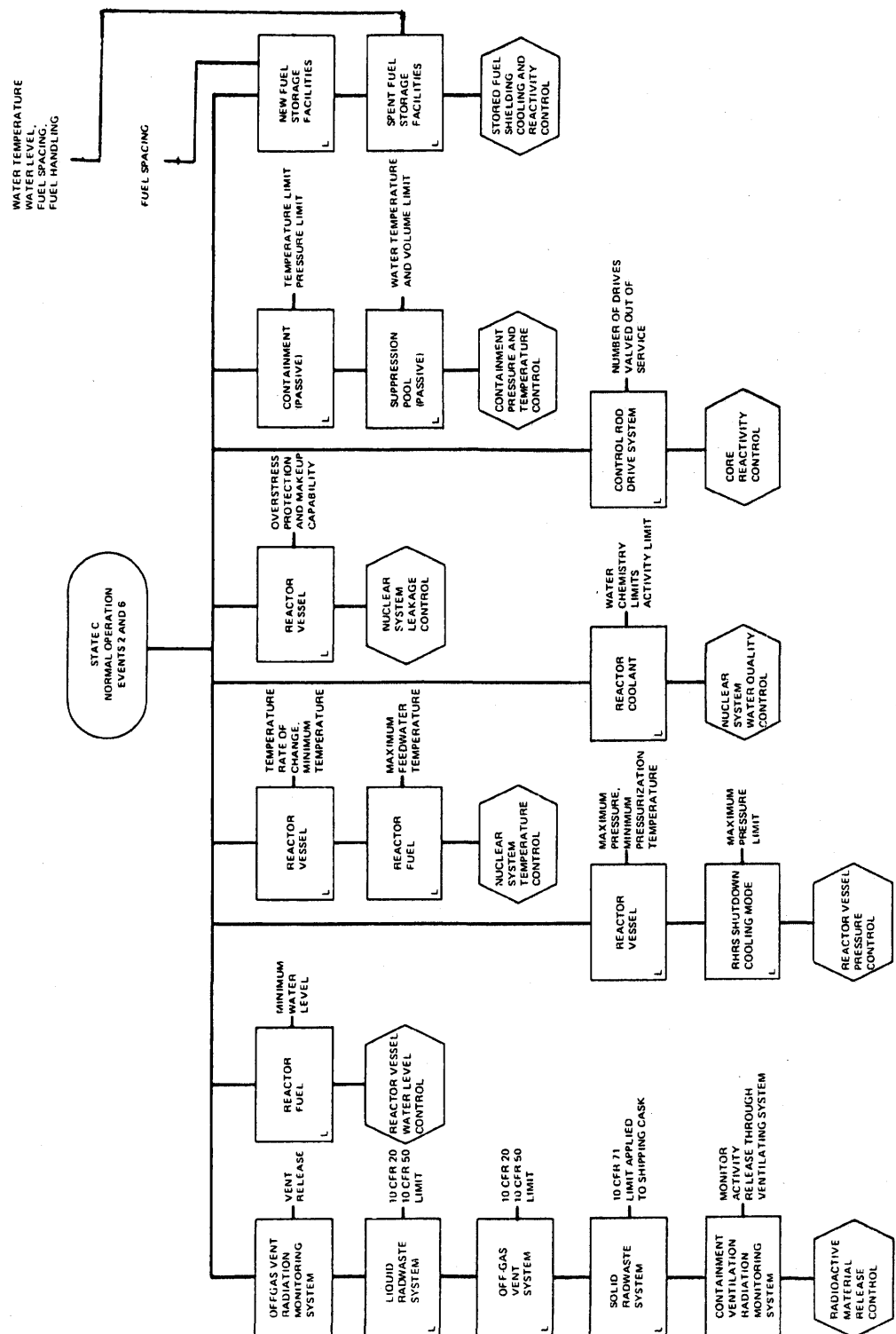
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PERRY NUCLEAR POWER PLANT

Safety Action Sequences for
Normal Operation in State B.

Figure 15A.6-4



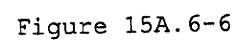
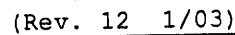
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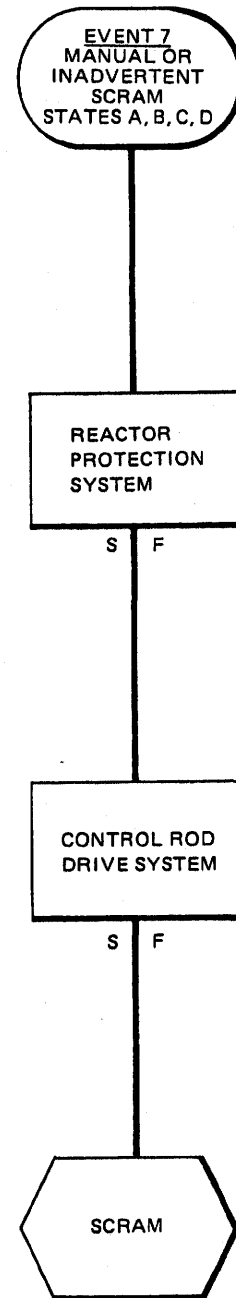


PERRY NUCLEAR POWER PLANT

Safety Action Sequences for
Normal Operation in State C.

Figure 15A.6-5





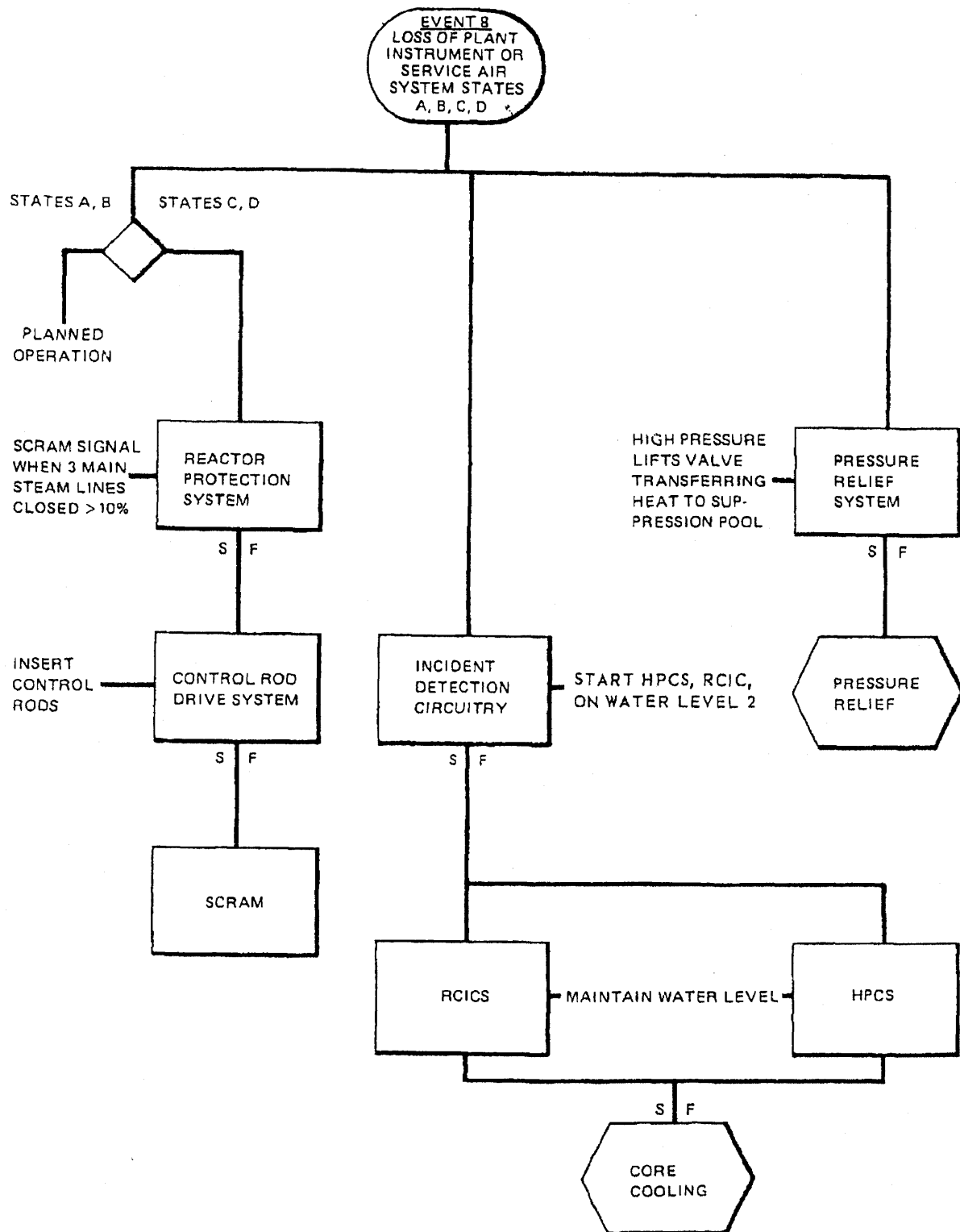
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PERRY NUCLEAR POWER PLANT

Protection Sequence for Manual
or Inadvertent Scram

Figure 15A.6-7



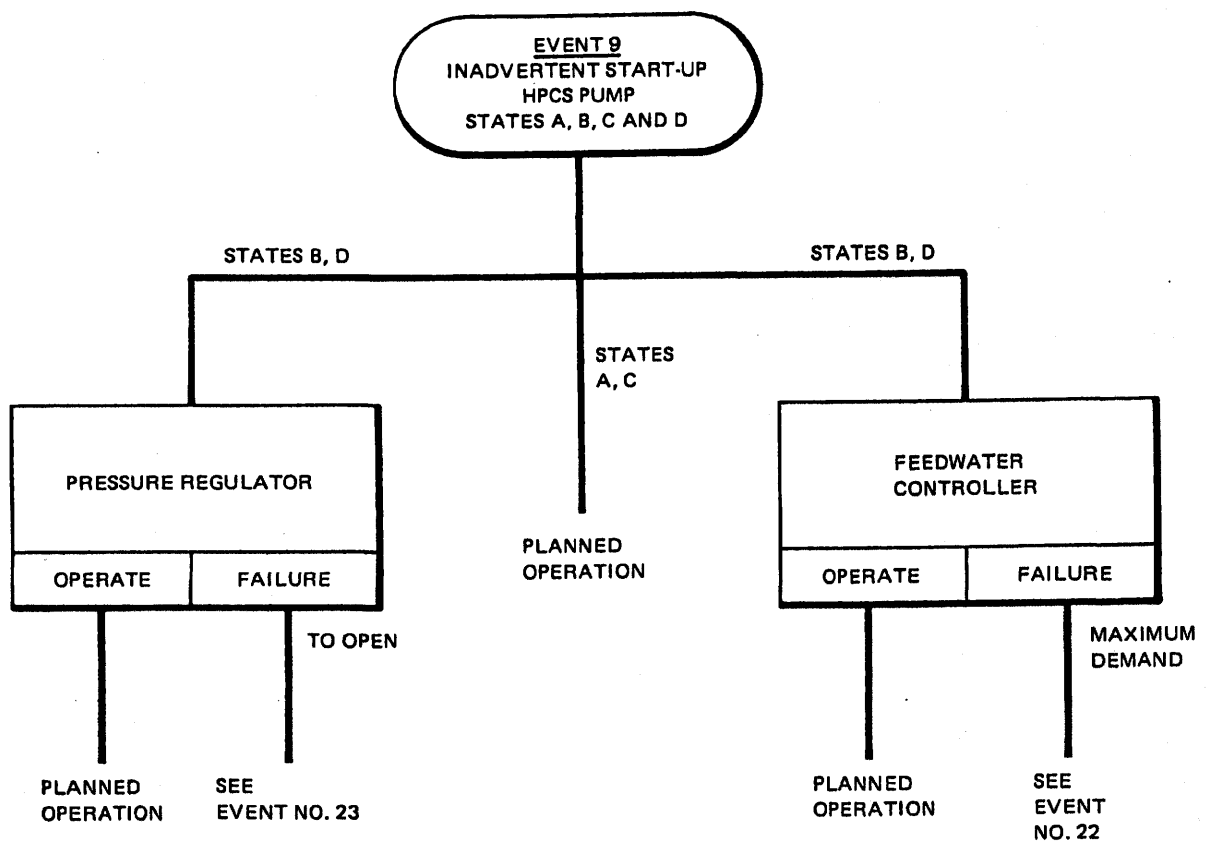
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Loss of
Plant Instrument or Service
Air System

Figure 15A.6-8



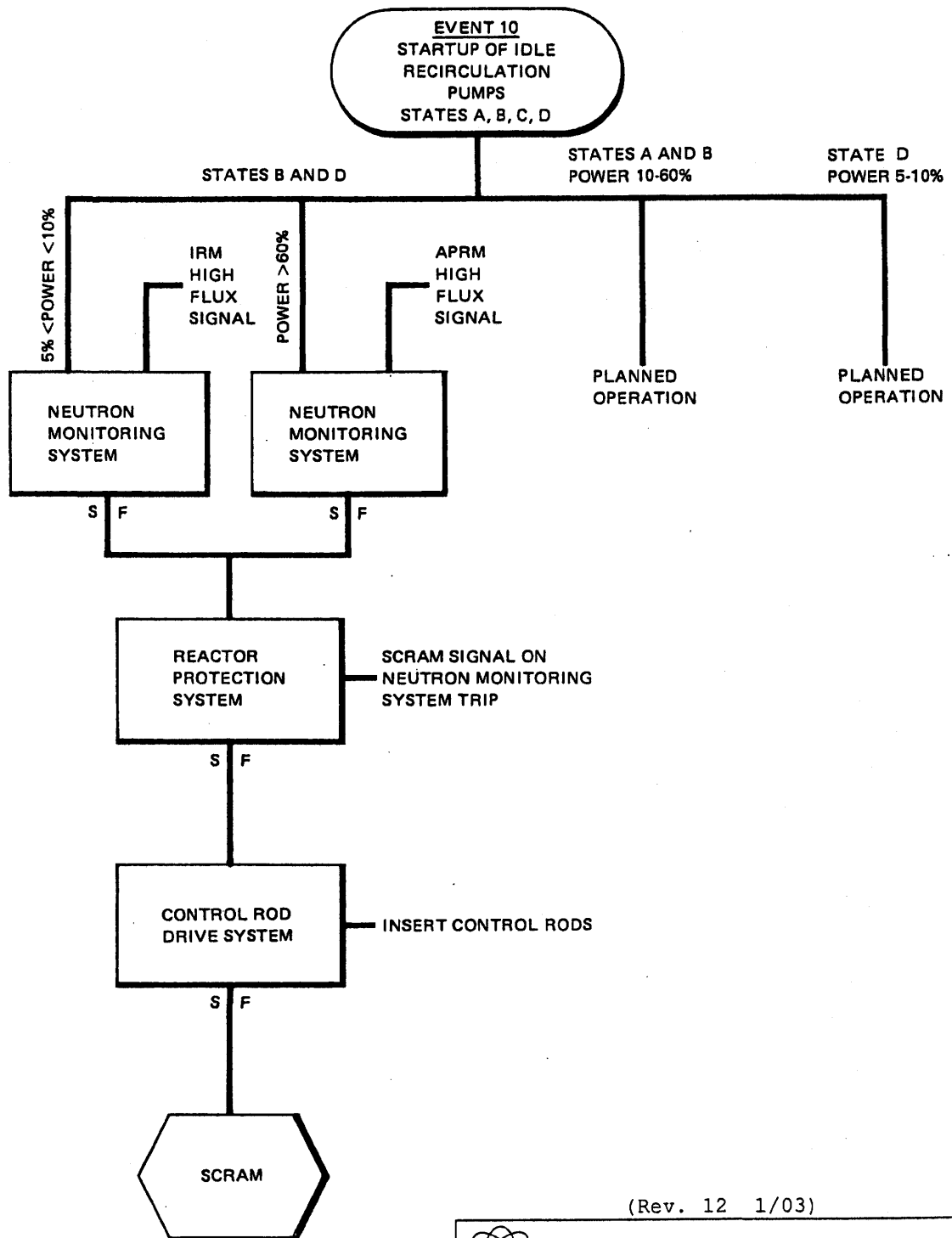
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PERRY NUCLEAR POWER PLANT

Protective Sequence for
Inadvertent Startup of HPCS Pumps

Figure 15A.6-9



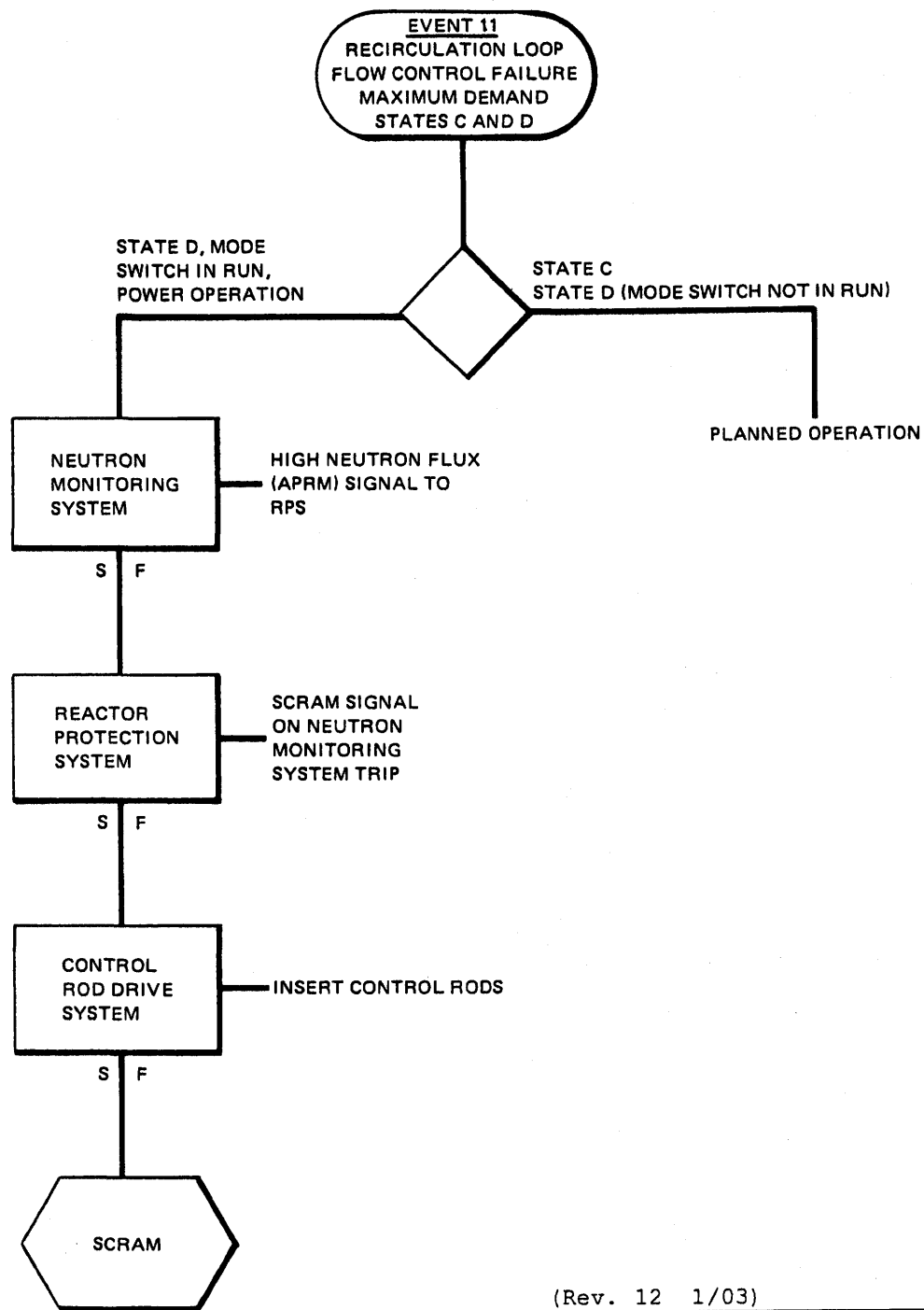
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Inadvertent
Startup of Idle
Recirculation Loop Pump

Figure 15A.6-10



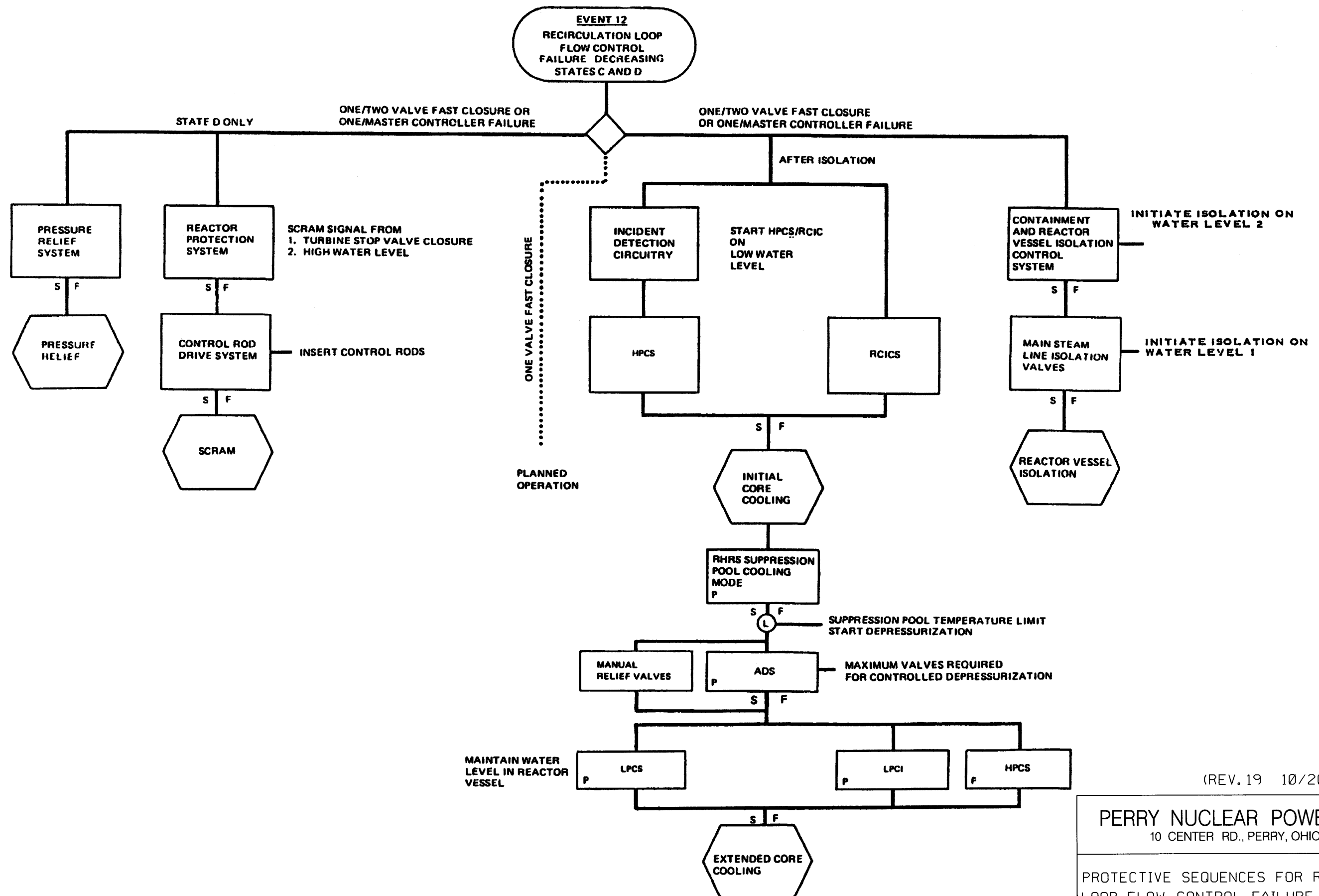
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PERRY NUCLEAR POWER PLANT

Protective Sequence for
Recirculation Loop Flow Control
Failure - Maximum Demand

Figure 15A.6-11

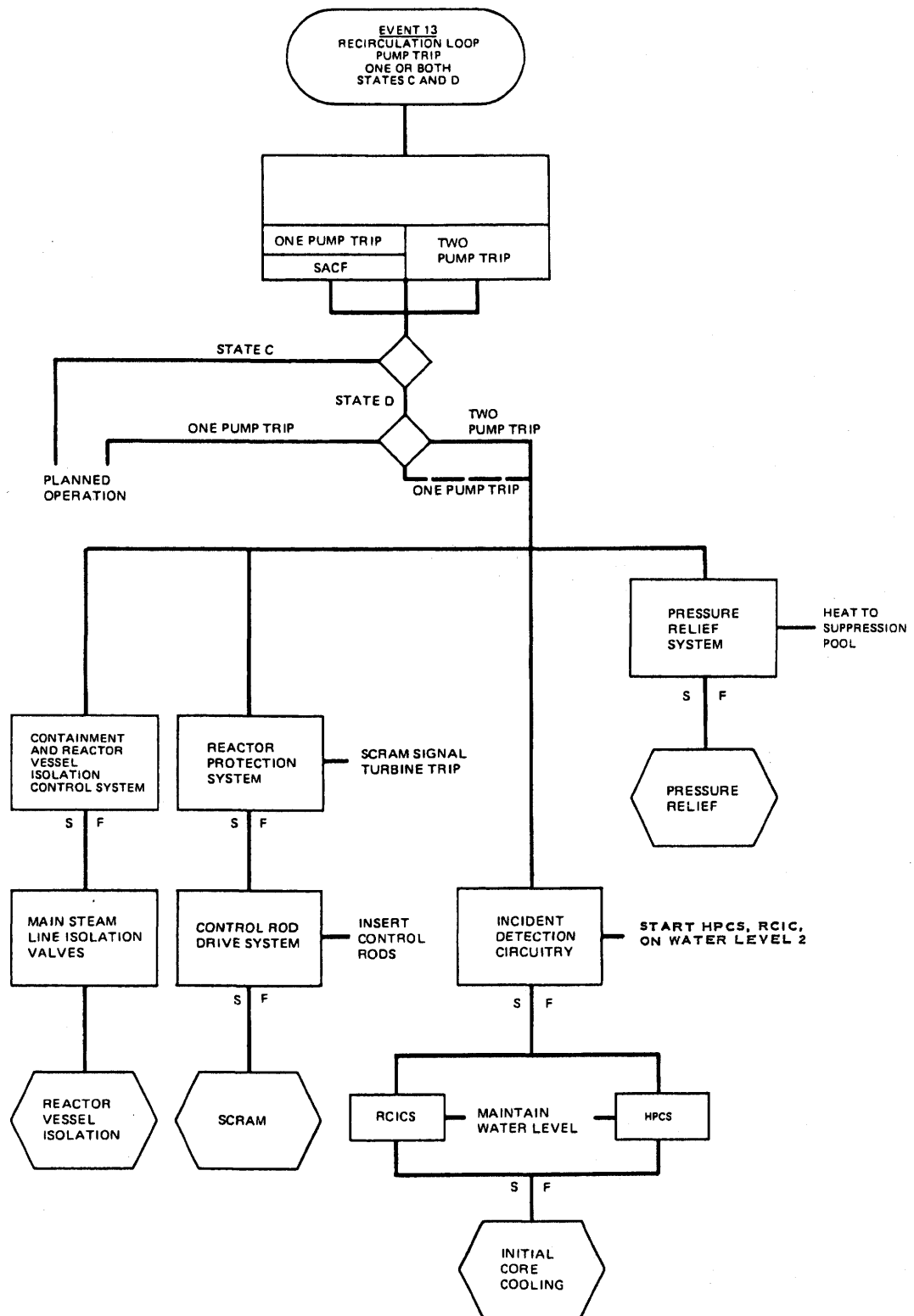


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PERRY NUCLEAR POWER PLANT
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PROTECTIVE SEQUENCES FOR RECIRCULATION
LOOP FLOW CONTROL FAILURE - DECREASING

FIGURE 15A.6-12



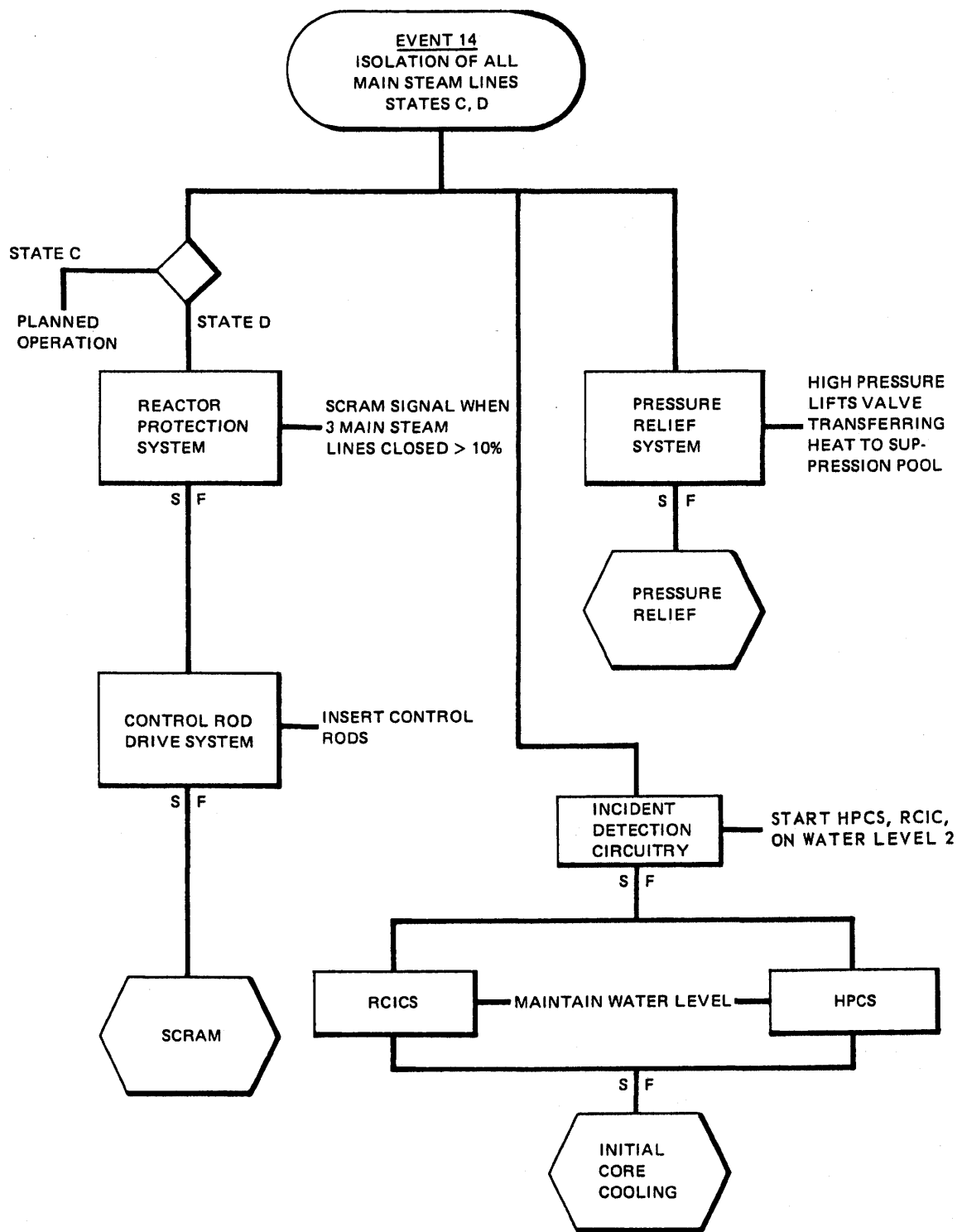
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PERRY NUCLEAR POWER PLANT

Recirculation Loop Pump Trip -
one or both

Figure 15A.6-13



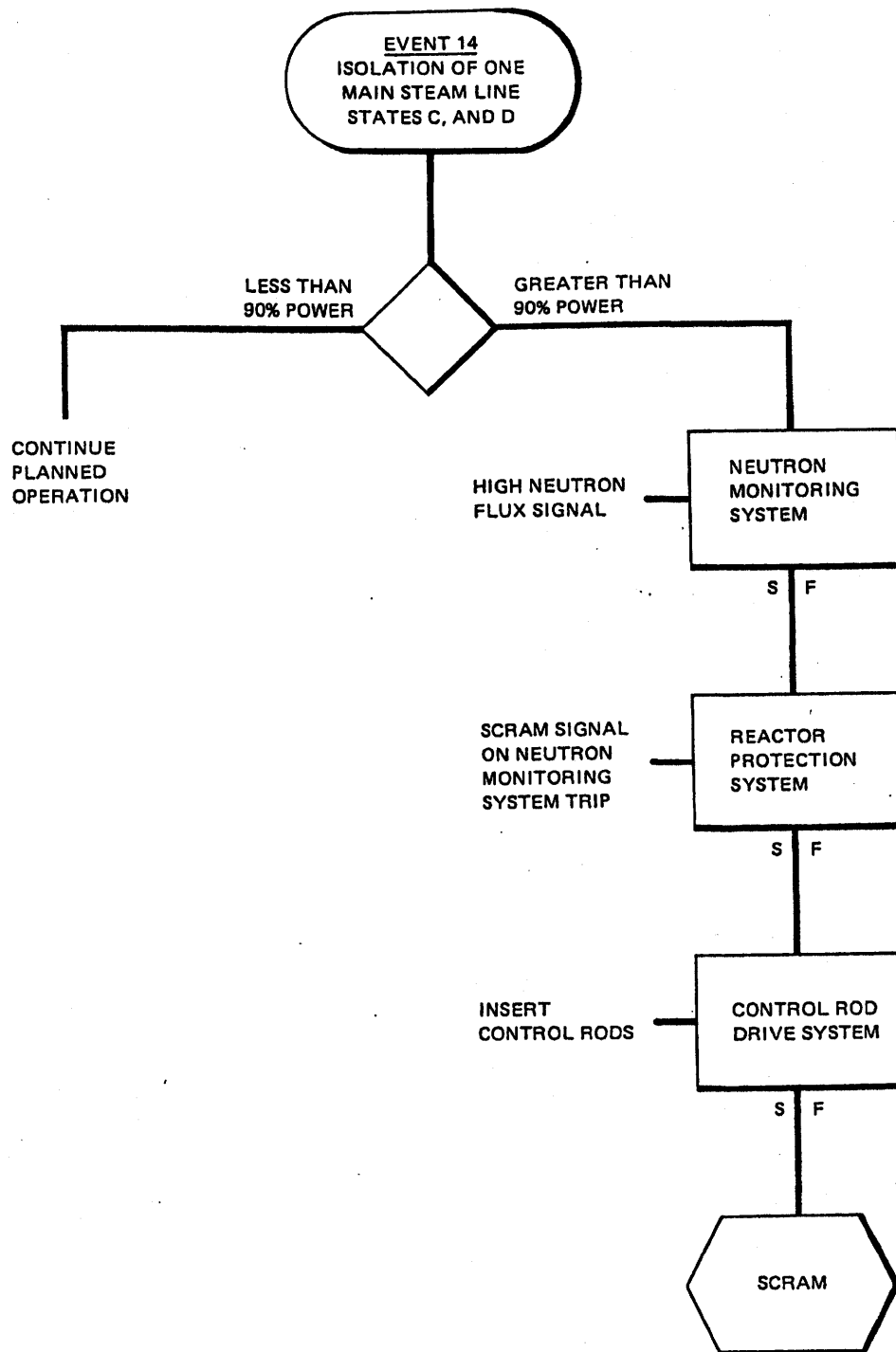
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Isolation
of All Main Steam Lines

Figure 15A.6-14 (Sheet 1 of 2)



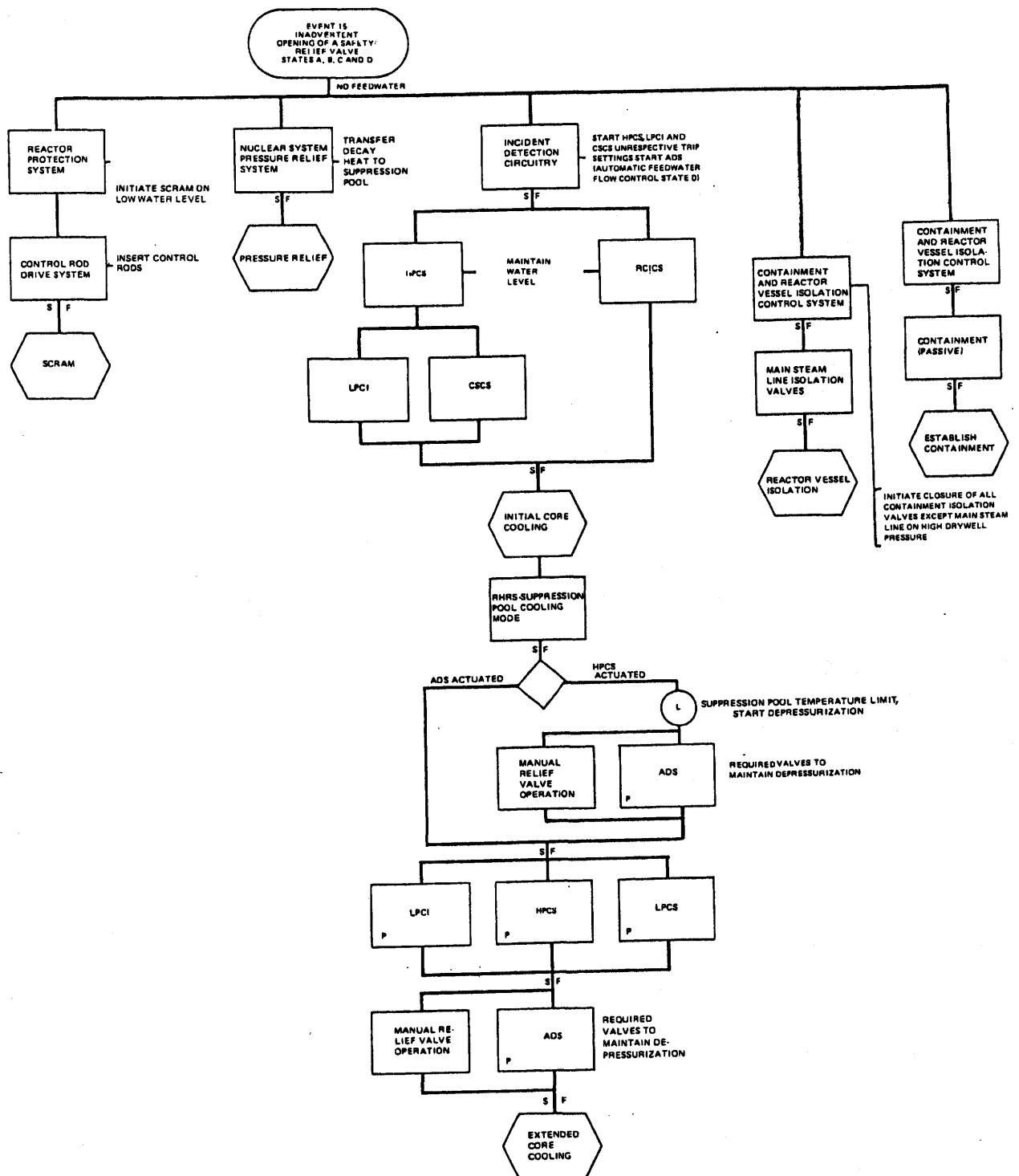
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Isolation
of One Main Steam Line

Figure 15A.6-14 (Sheet 2 of 2)



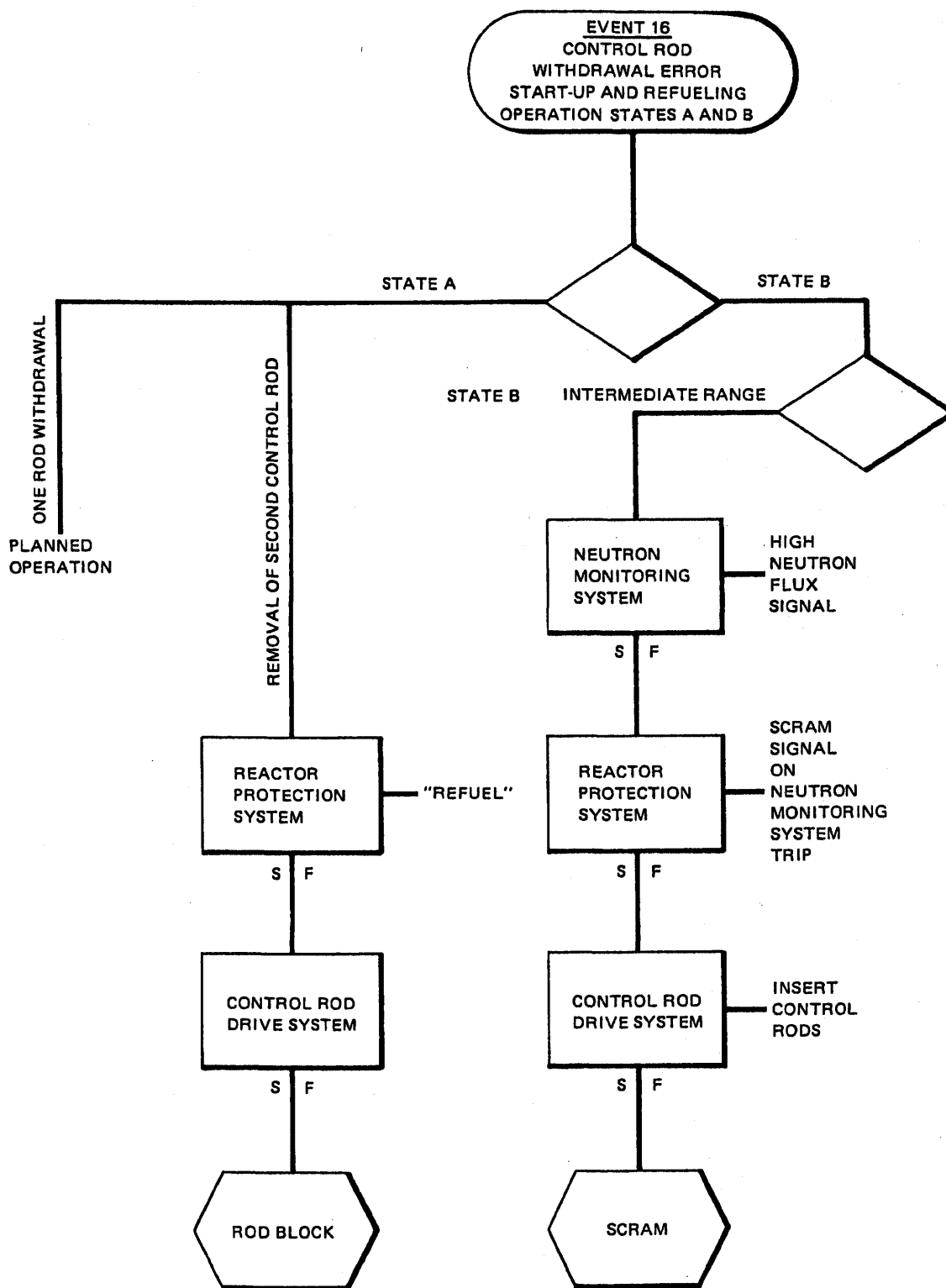
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PERRY NUCLEAR POWER PLANT

Protective Sequences for
Inadvertent Opening of a Safety/
Relief Valve

Figure 15A.6-15



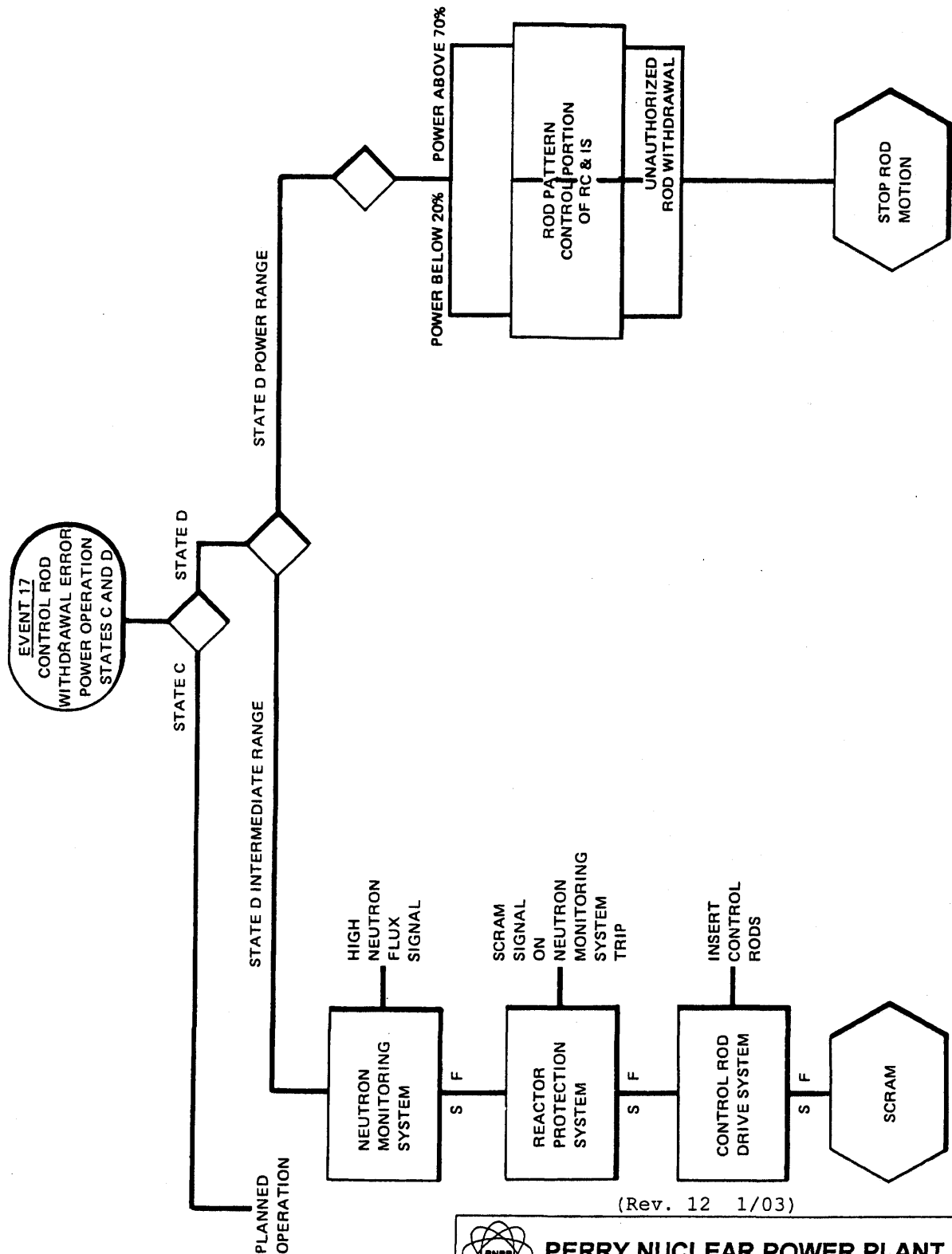
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Control Rod
Withdrawal Error for Startup and
Refueling Operations

Figure 15A.6-16



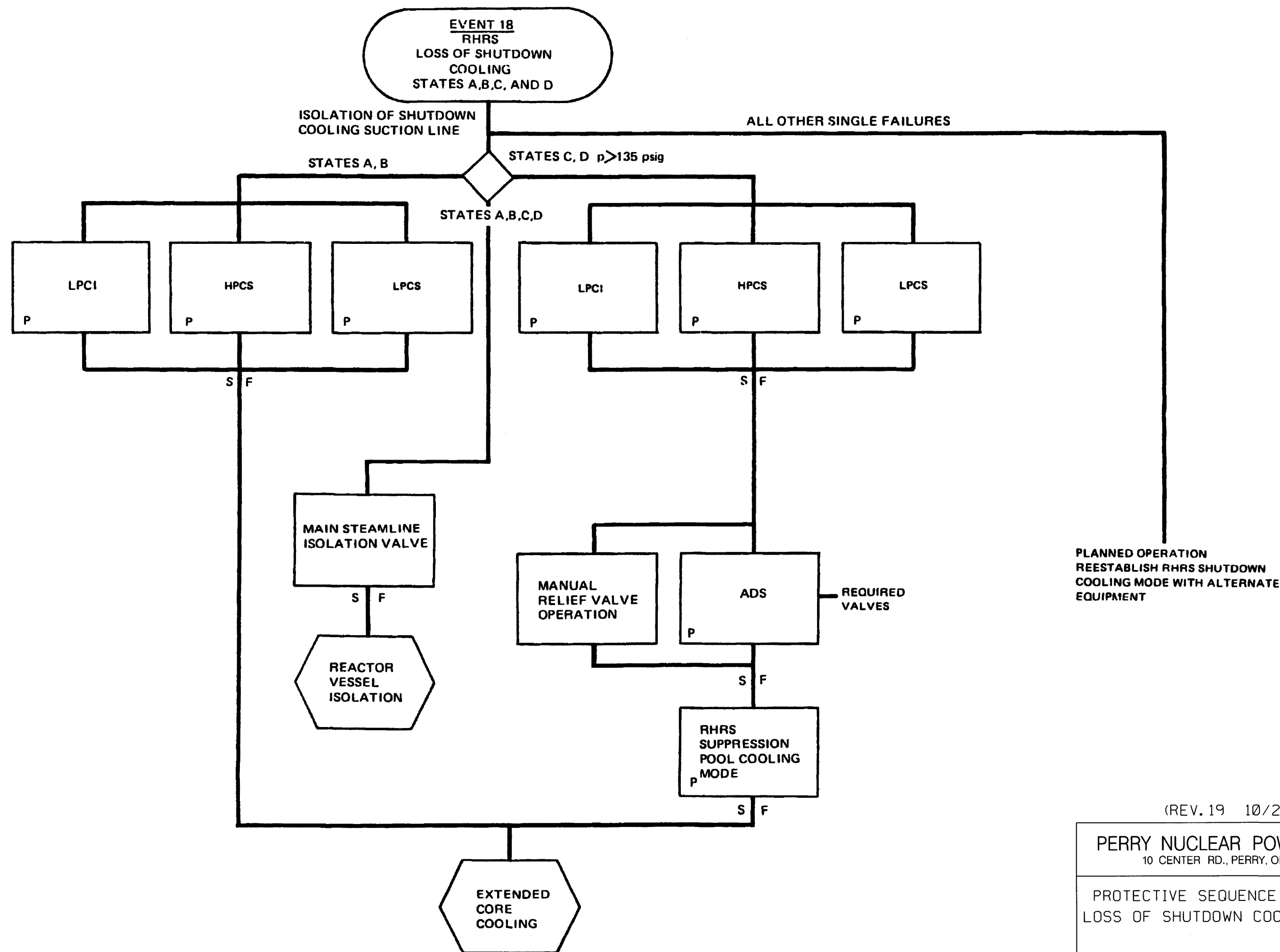
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Control Rod
Withdrawal Error for
Power Operation

Figure 15A.6-17

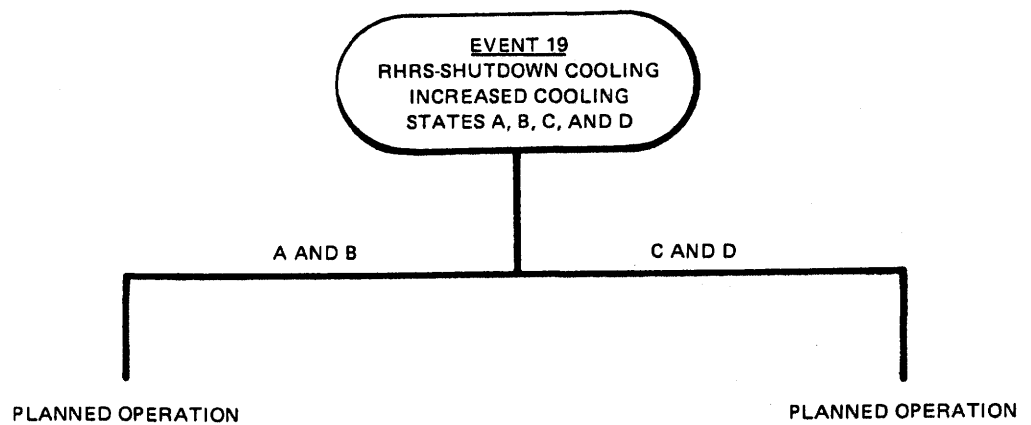


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PERRY NUCLEAR POWER PLANT
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PROTECTIVE SEQUENCE FOR RHRs -
LOSS OF SHUTDOWN COOLING FAILURE

FIGURE 15A.6-18



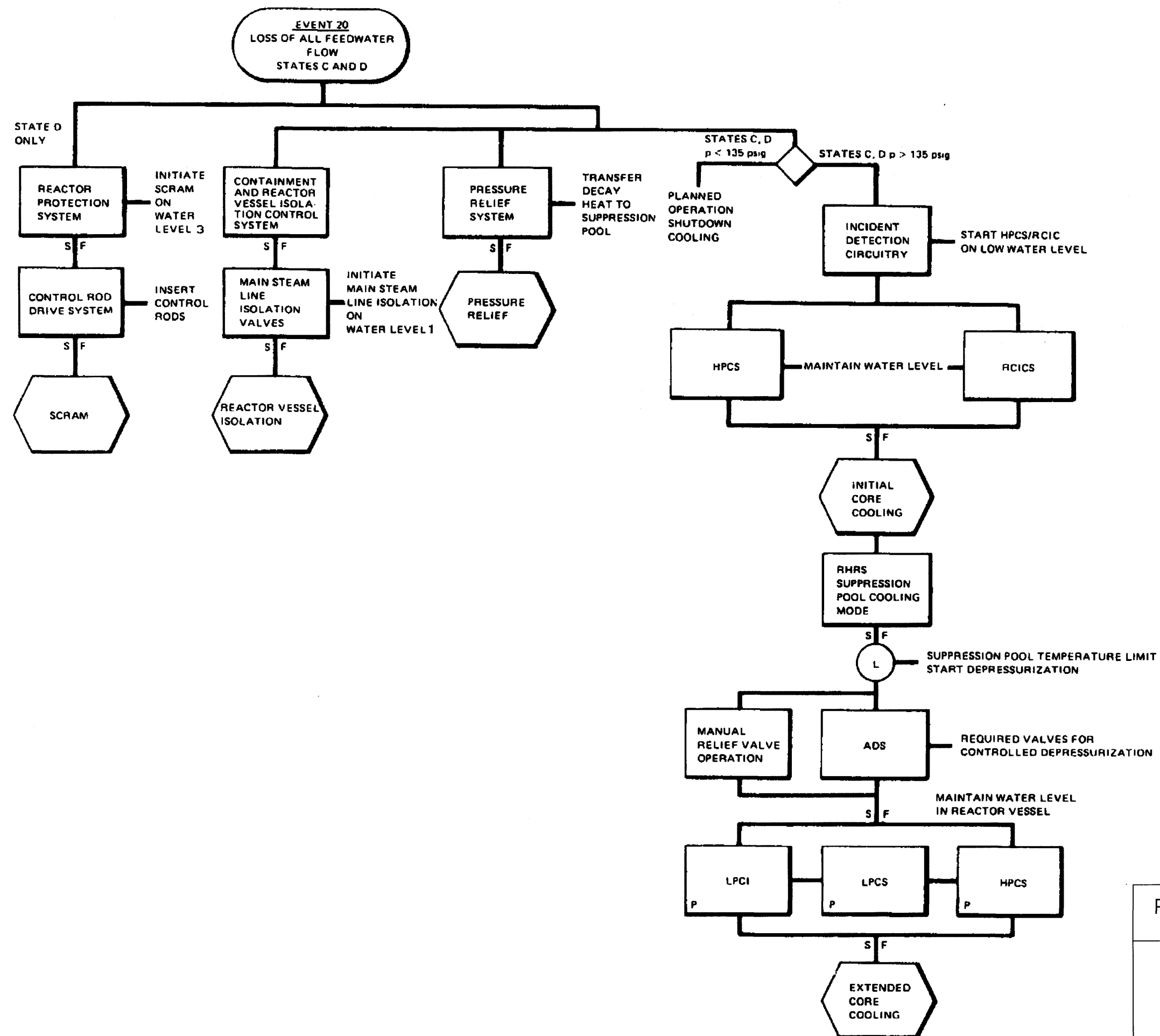
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PERRY NUCLEAR POWER PLANT

RHRS - Shutdown Cooling Failure -
Increased Cooling

Figure 15A.6-19

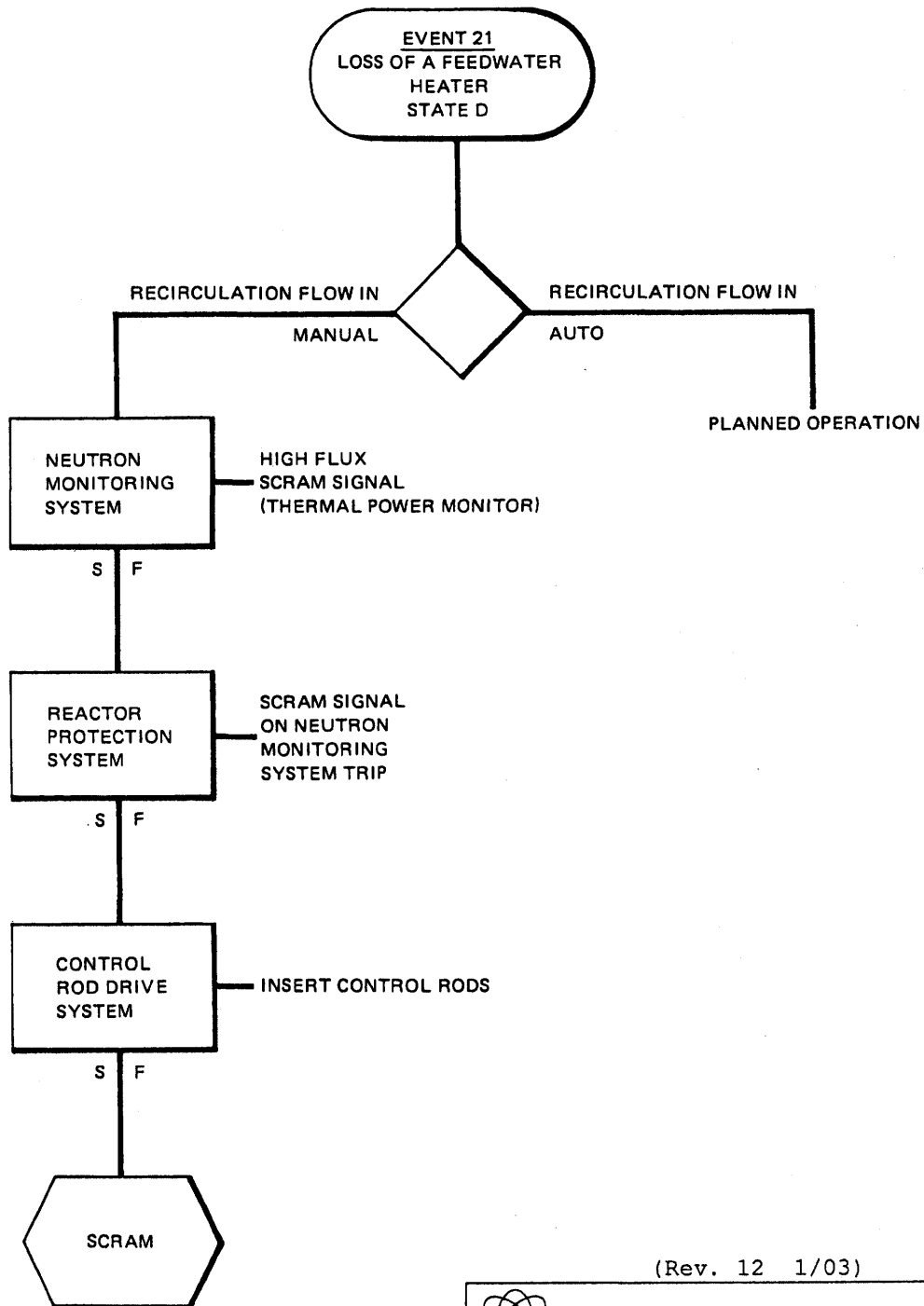


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PERRY NUCLEAR POWER PLANT
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PROTECTIVE SEQUENCE FOR
LOSS OF FEEDWATER FLOW

FIGURE 15A.6-20



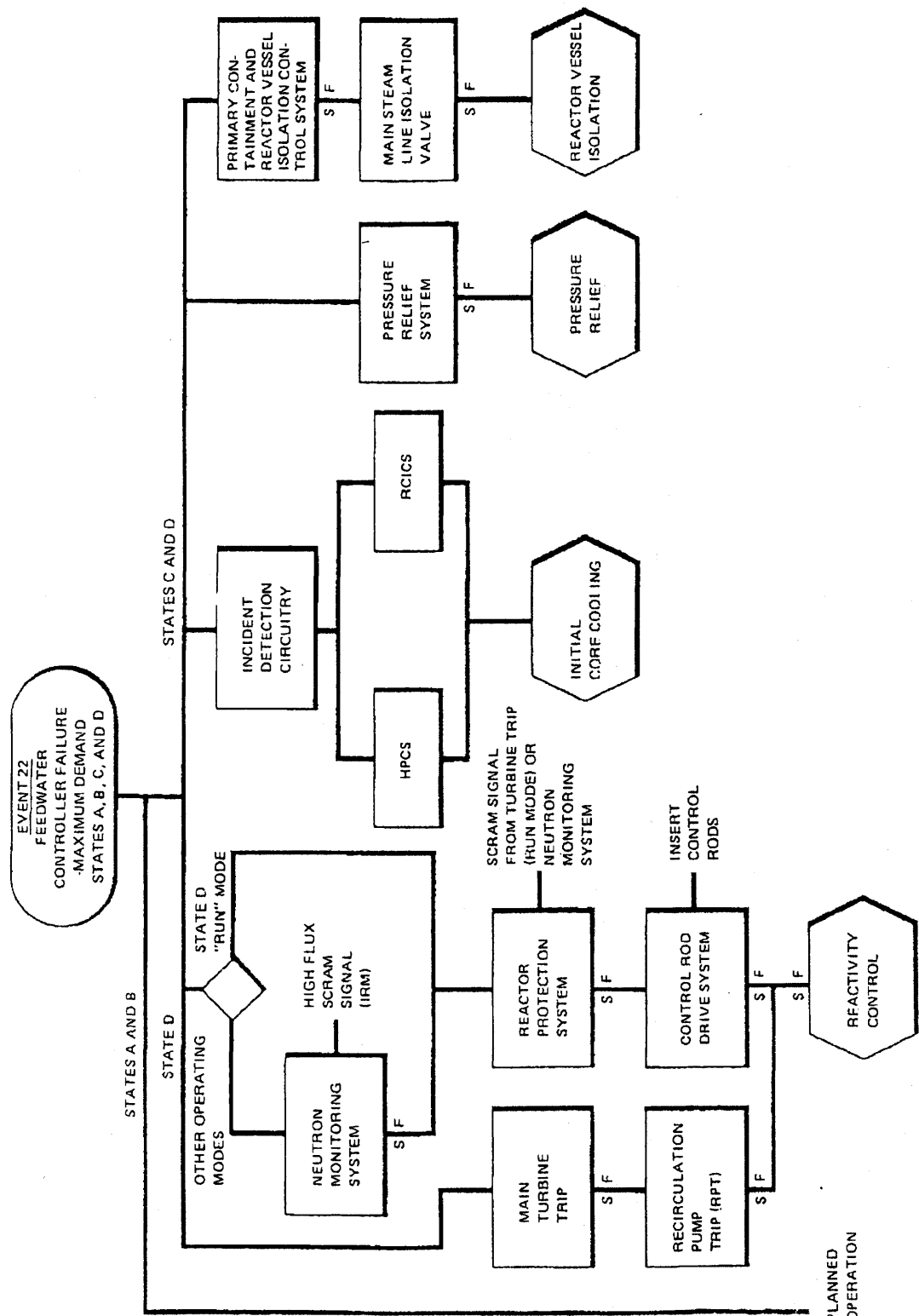
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Loss of a
Feedwater Heater

Figure 15A.6-21



(Rev. 12 1/03)




PERRY NUCLEAR POWER PLANT

Protective Sequences for
Feedwater Controller Failure -
Maximum Demand

Figure 15A.6-22



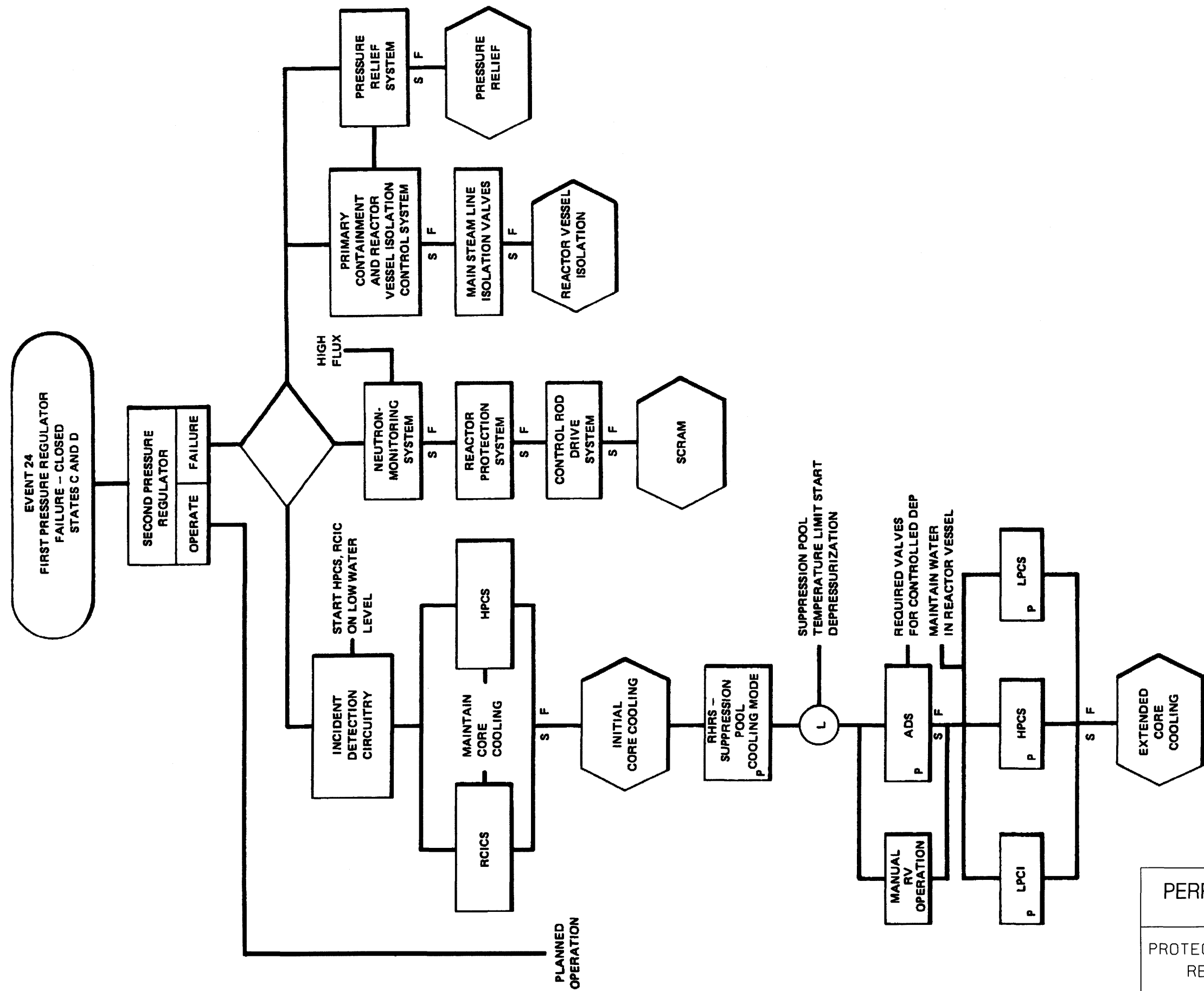
1. THIS PATH IS RELATED ONLY TO CONTROLLING THE COOLDOWN SO AS NOT TO EXCEED THE RATE OF 100°F IN ANY ONE-HOUR PERIOD.



PERRY NUCLEAR POWER PLANT

Protective Sequences for
Pressure Regulator Failure - Open

Figure 15A.6-23

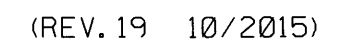


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PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

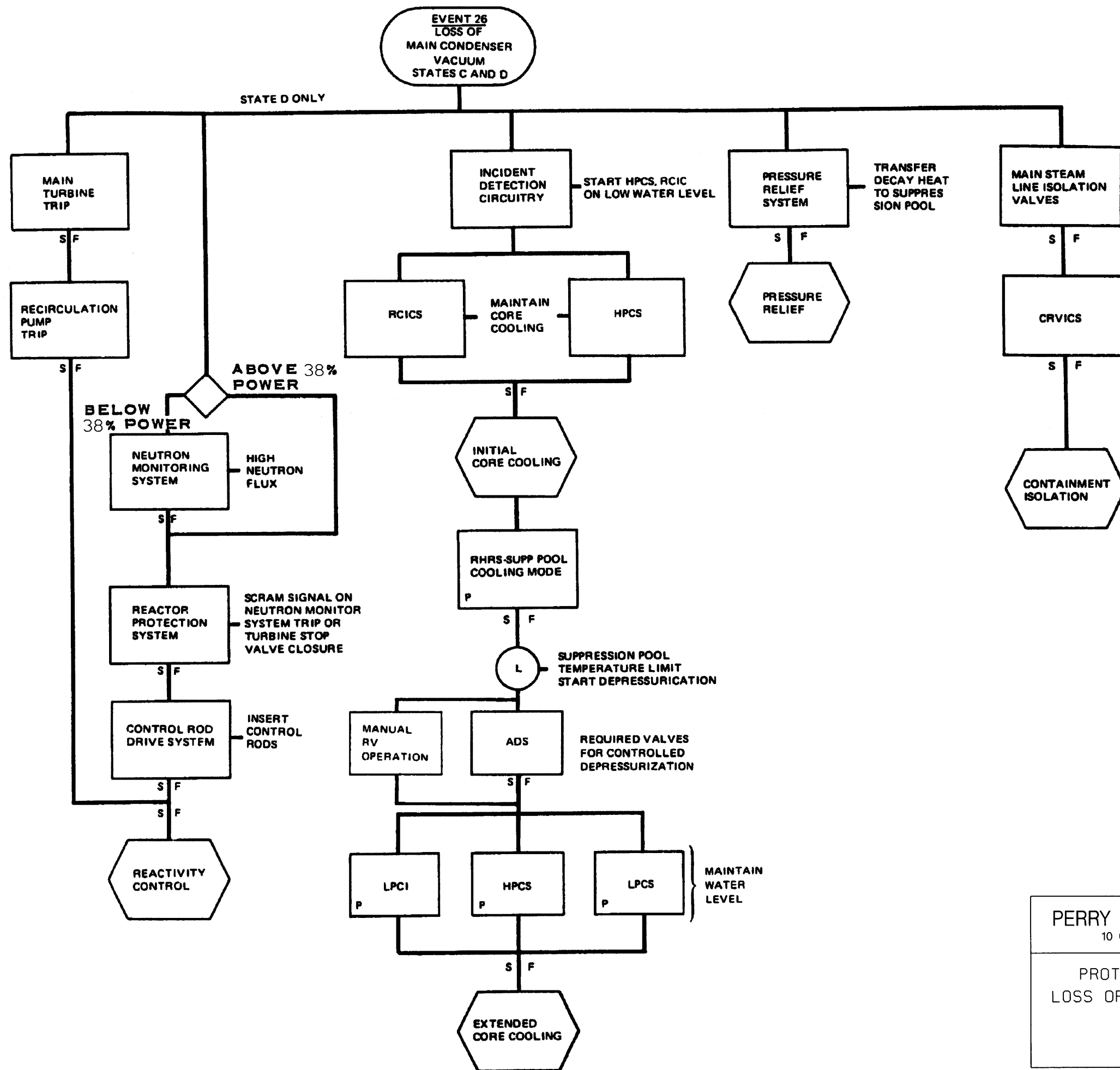
PROTECTIVE SEQUENCES FOR PRESSURE
REGULATOR FAILURE - CLOSED

FIGURE 15A.6-24



PROTECTIVE SEQUENCES FOR MAIN TURBINE TRIP WITH BYPASS

FIGURE 15A.6-25

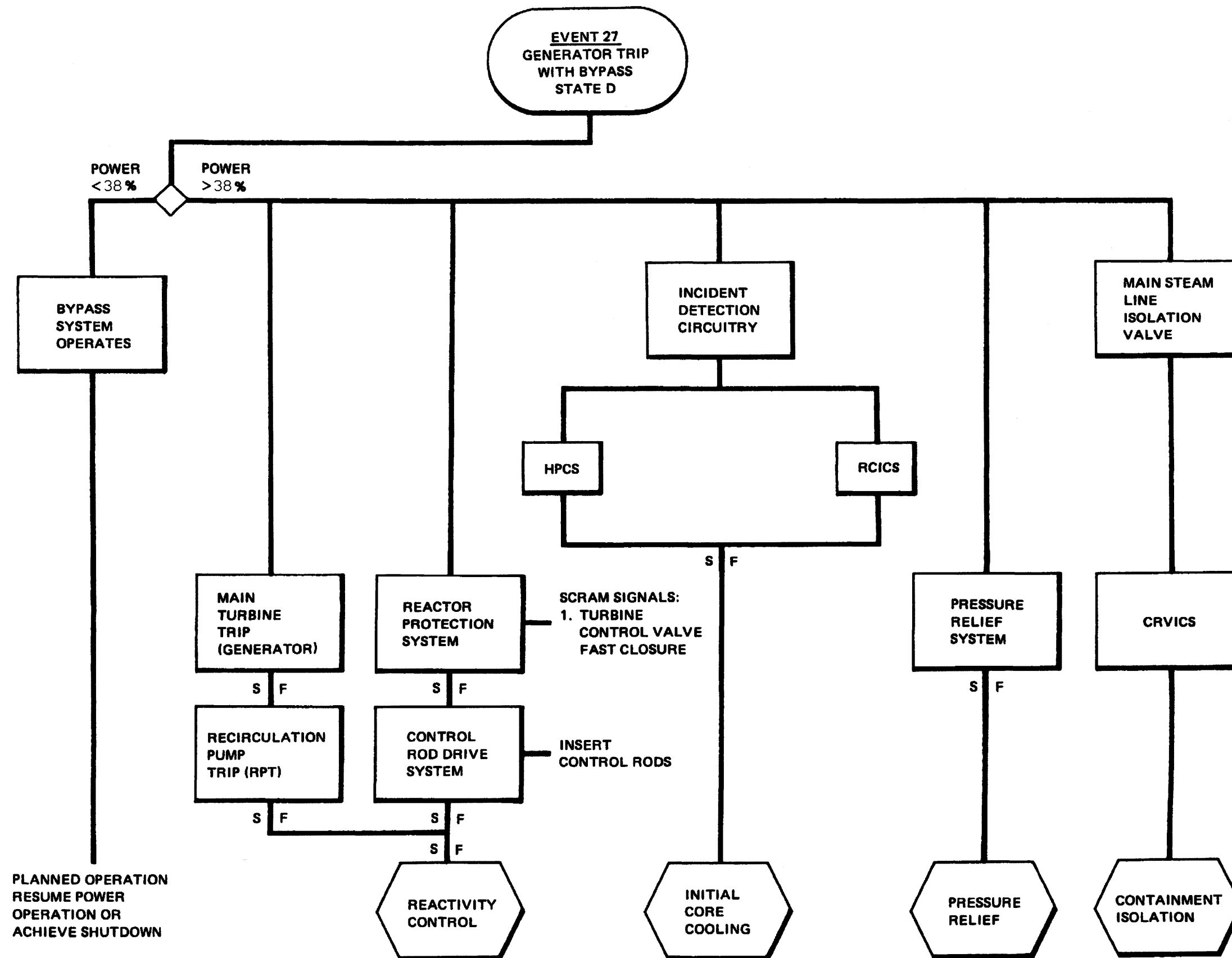


(REV. 19 10/2015)

PERRY NUCLEAR POWER PLANT
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PROTECTIVE SEQUENCES FOR
LOSS OF MAIN CONDENSER VACUUM

FIGURE 15A.6-26

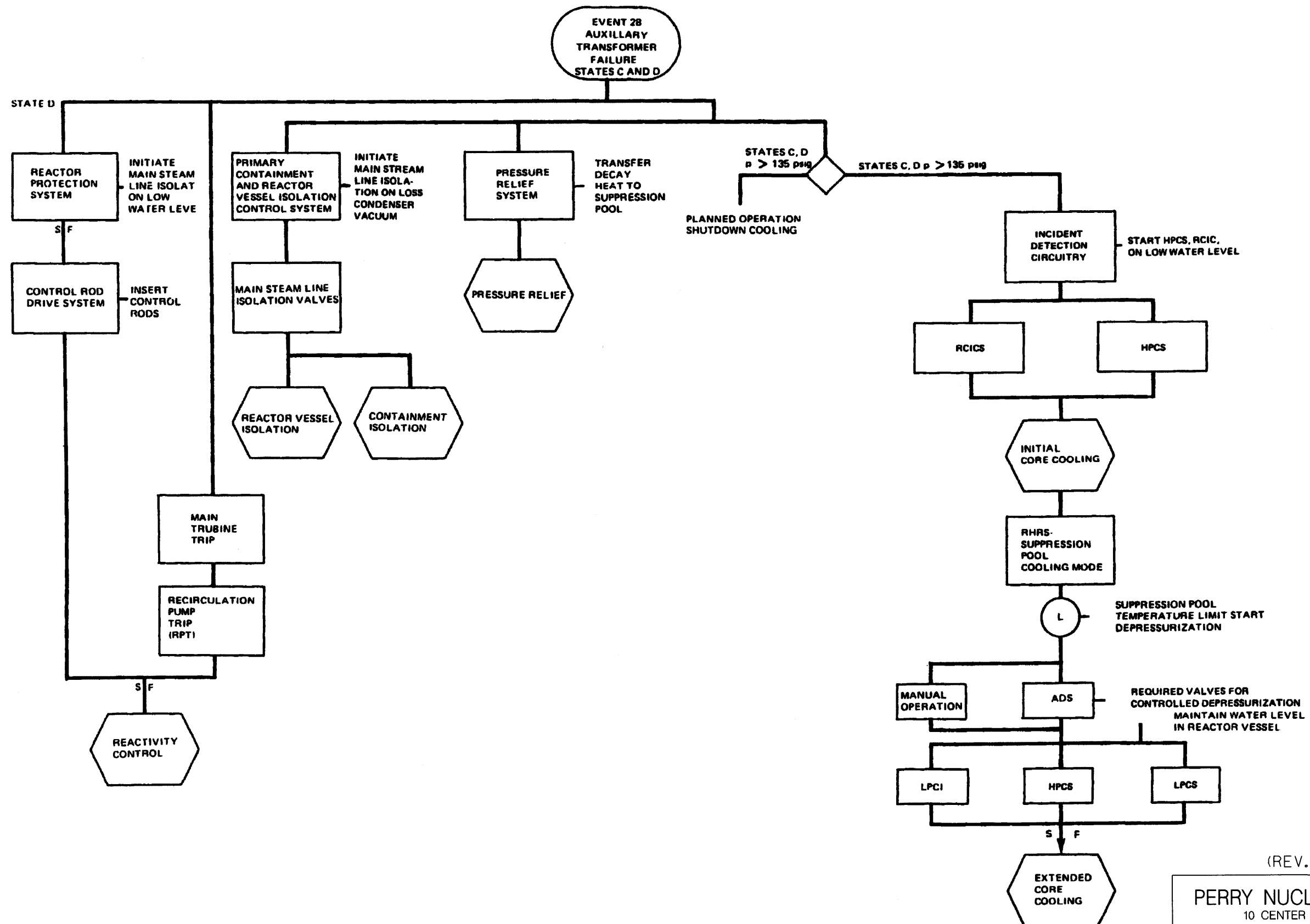


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PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

PROTECTIVE SEQUENCES FOR MAIN
GENERATOR TRIP WITH BYPASS

FIGURE 15A.6-27

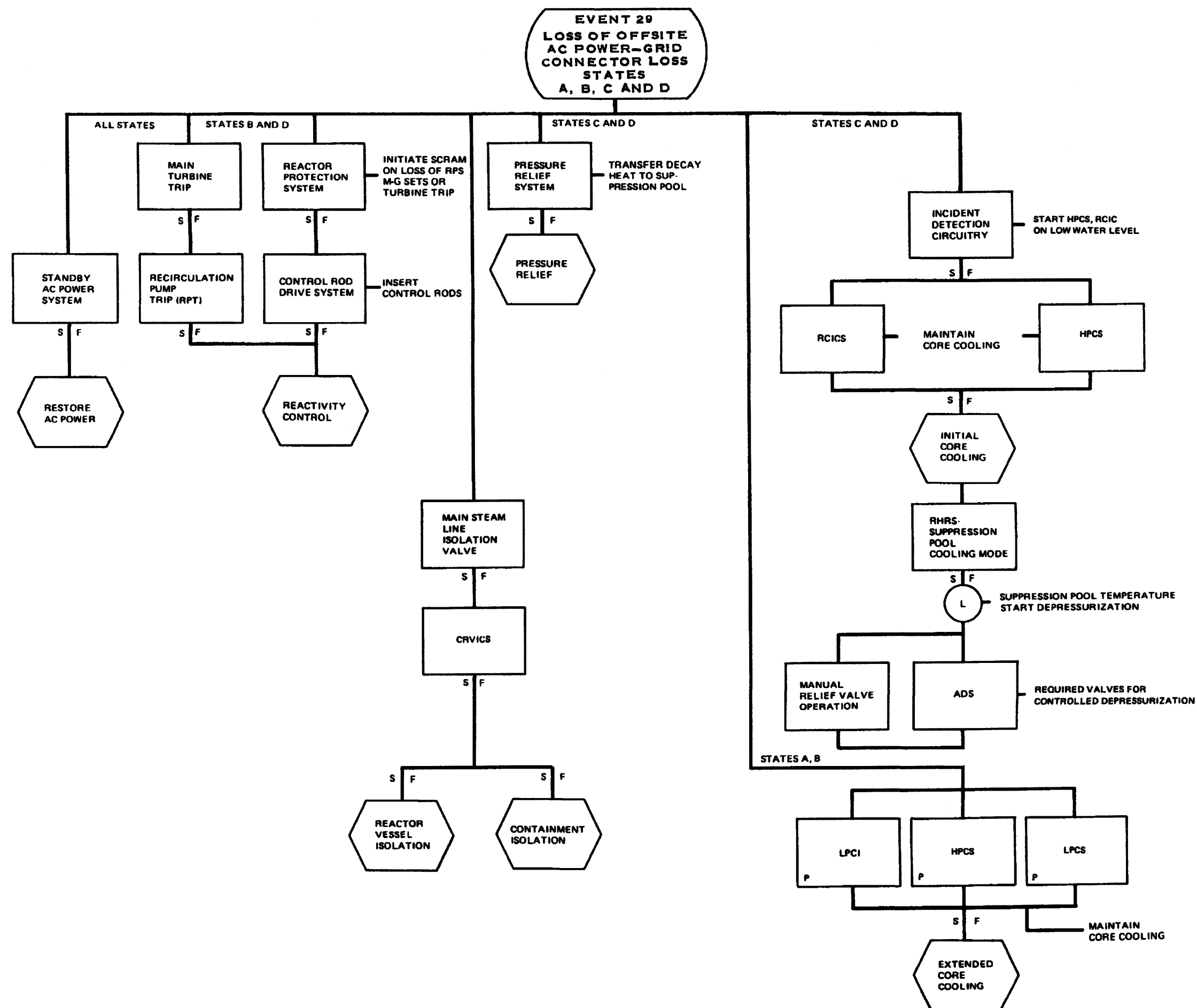


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PERRY NUCLEAR POWER PLANT
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PROTECTIVE SEQUENCE FOR
AUXILIARY TRANSFORMER FAILURE

FIGURE 15A.6-28

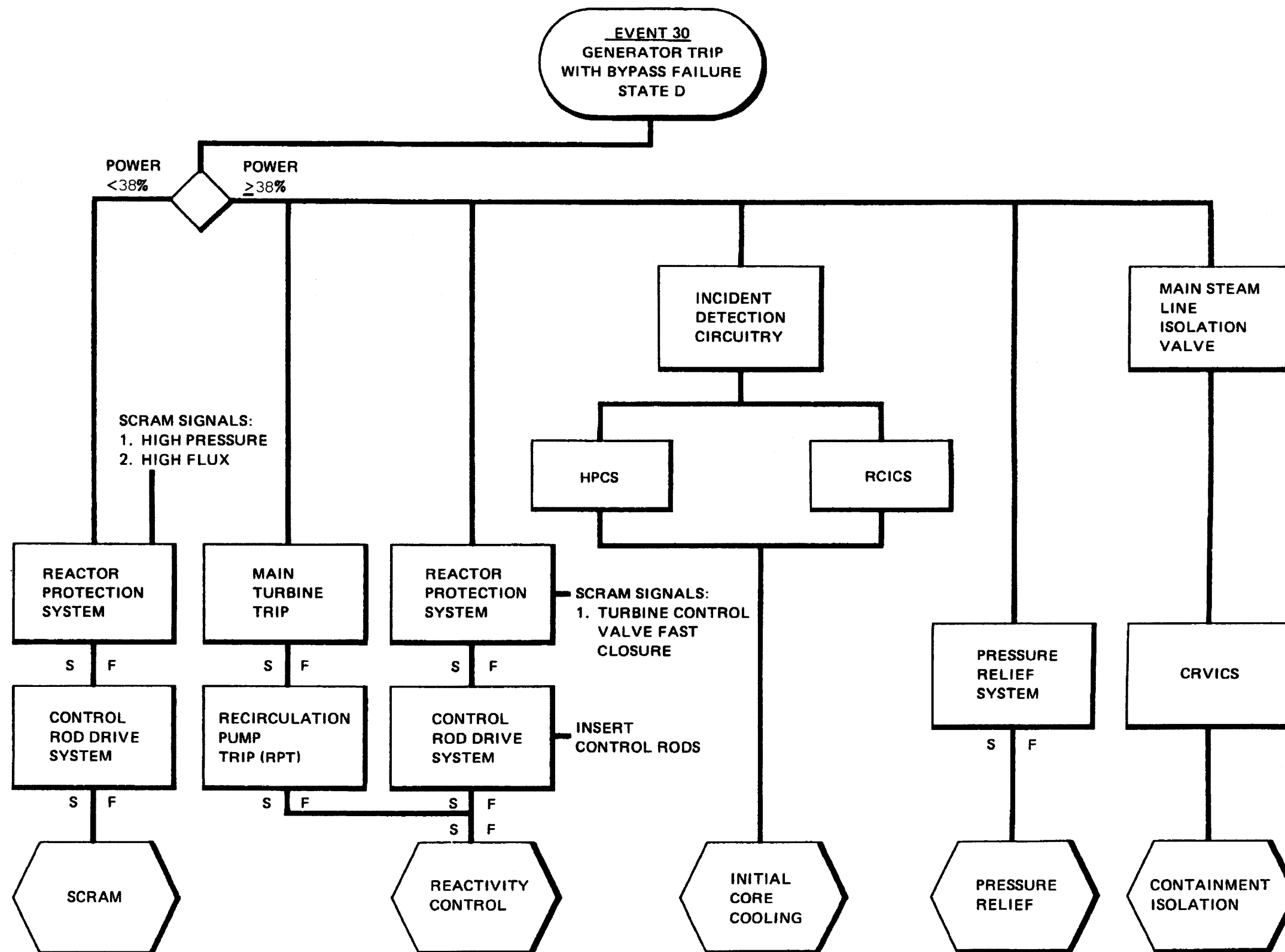


(REV. 19 10/2015)

PERRY NUCLEAR POWER PLANT
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PROTECTIVE SEQUENCES FOR LOSS OF
AC POWER - GRID CONNECTION LOSS

FIGURE 15A.6-29

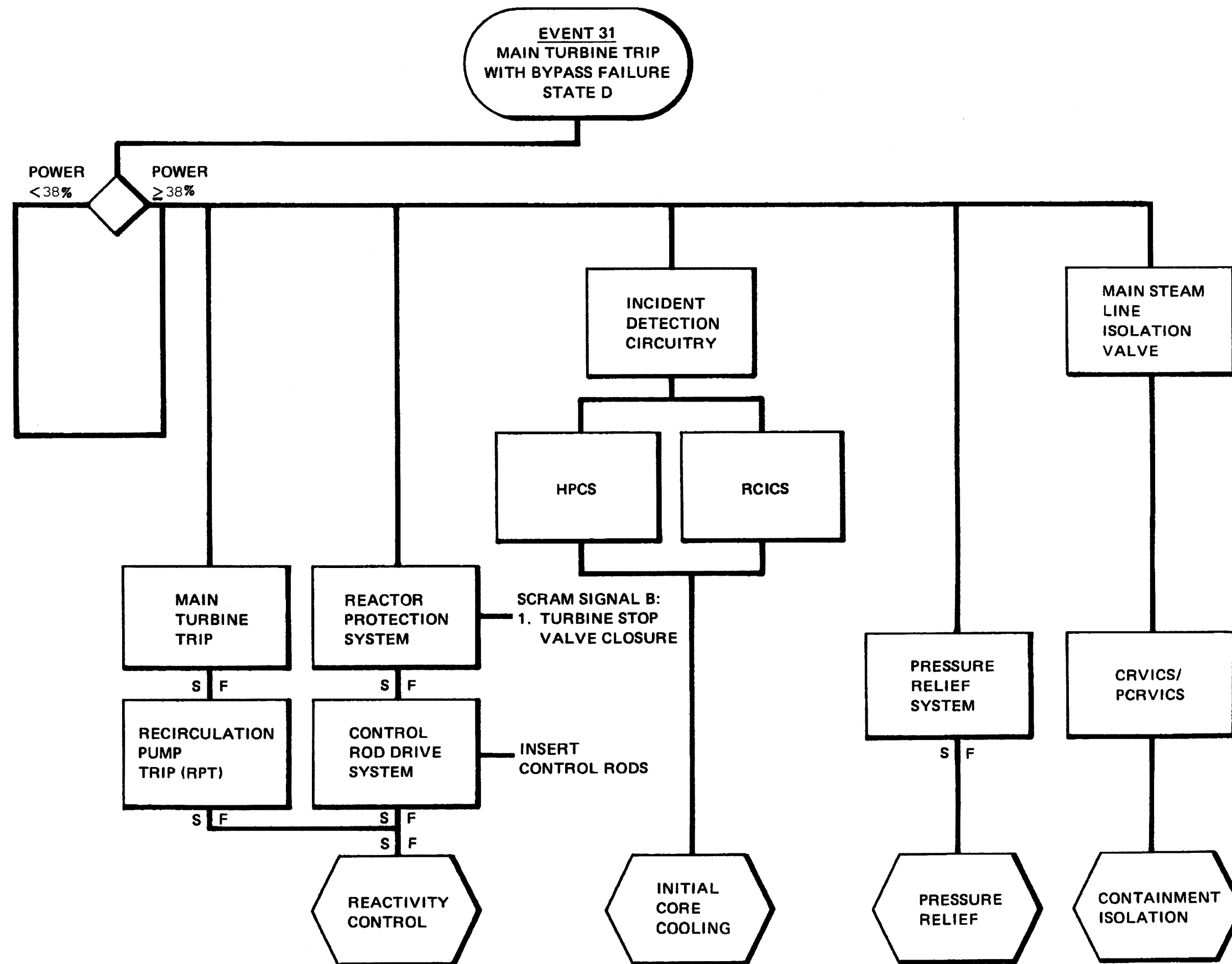


(REV. 19 10/2015)

PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

PROACTIVE SEQUENCES FOR MAIN
GENERATOR TRIP - WITH BYPASS FAILURE

FIGURE 15A.6-30



(REV. 19 10/2015)

PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

PROTECTIVE SEQUENCES FOR TURBINE
TRIP - WITH BYPASS FAILURE

FIGURE 15A.6-31

EVENT 32
INADVERTENT LOADING AND
OPERATION – FUEL ASSEMBLY
IN IMPROPER POSITION
STATES A, B, C, D

PLANNED
OPERATION

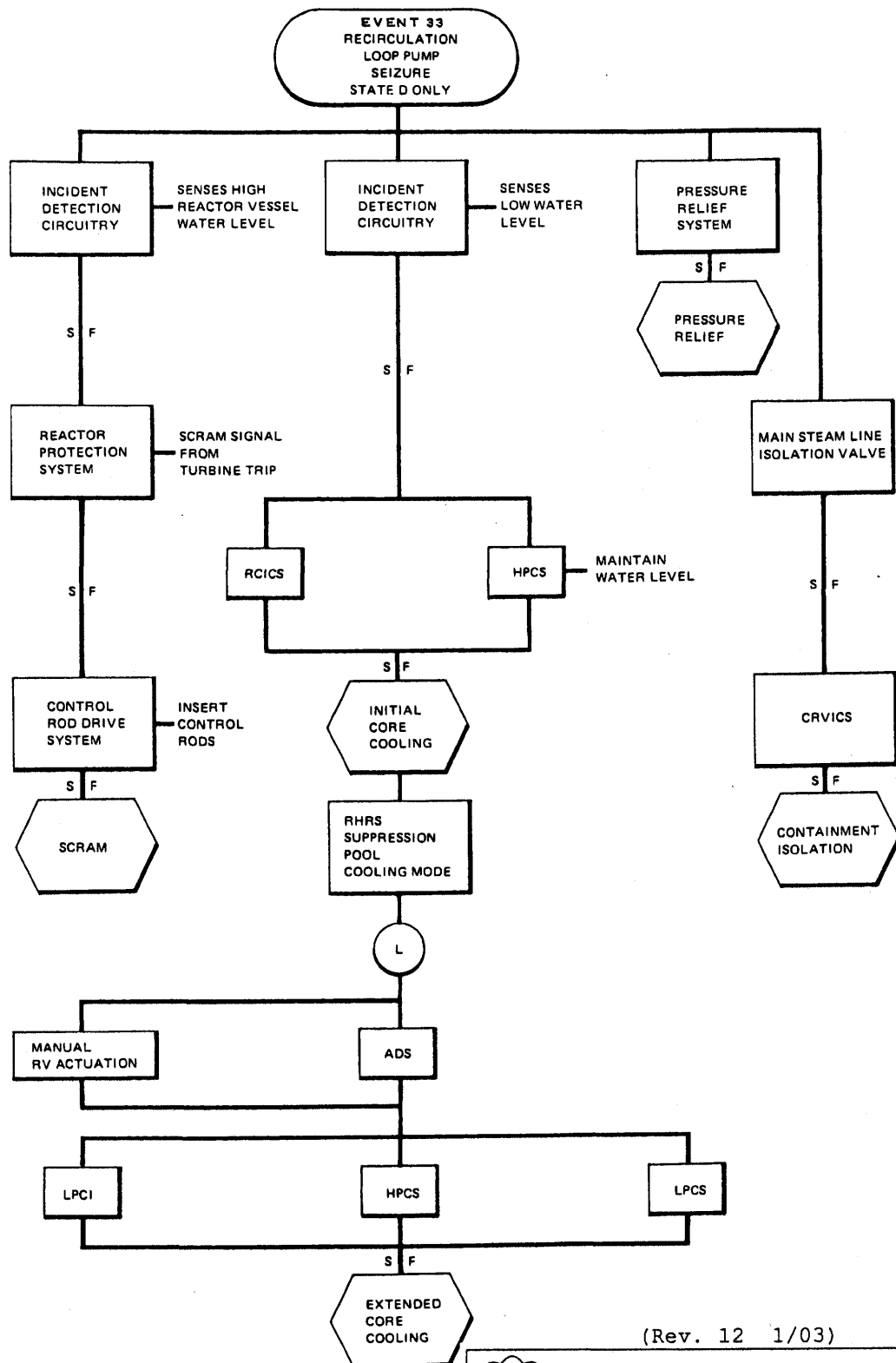
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Protective Sequence for
Inadvertent Loading and Operation
of Fuel Assembly in
Improper Position

Figure 15A.6-32



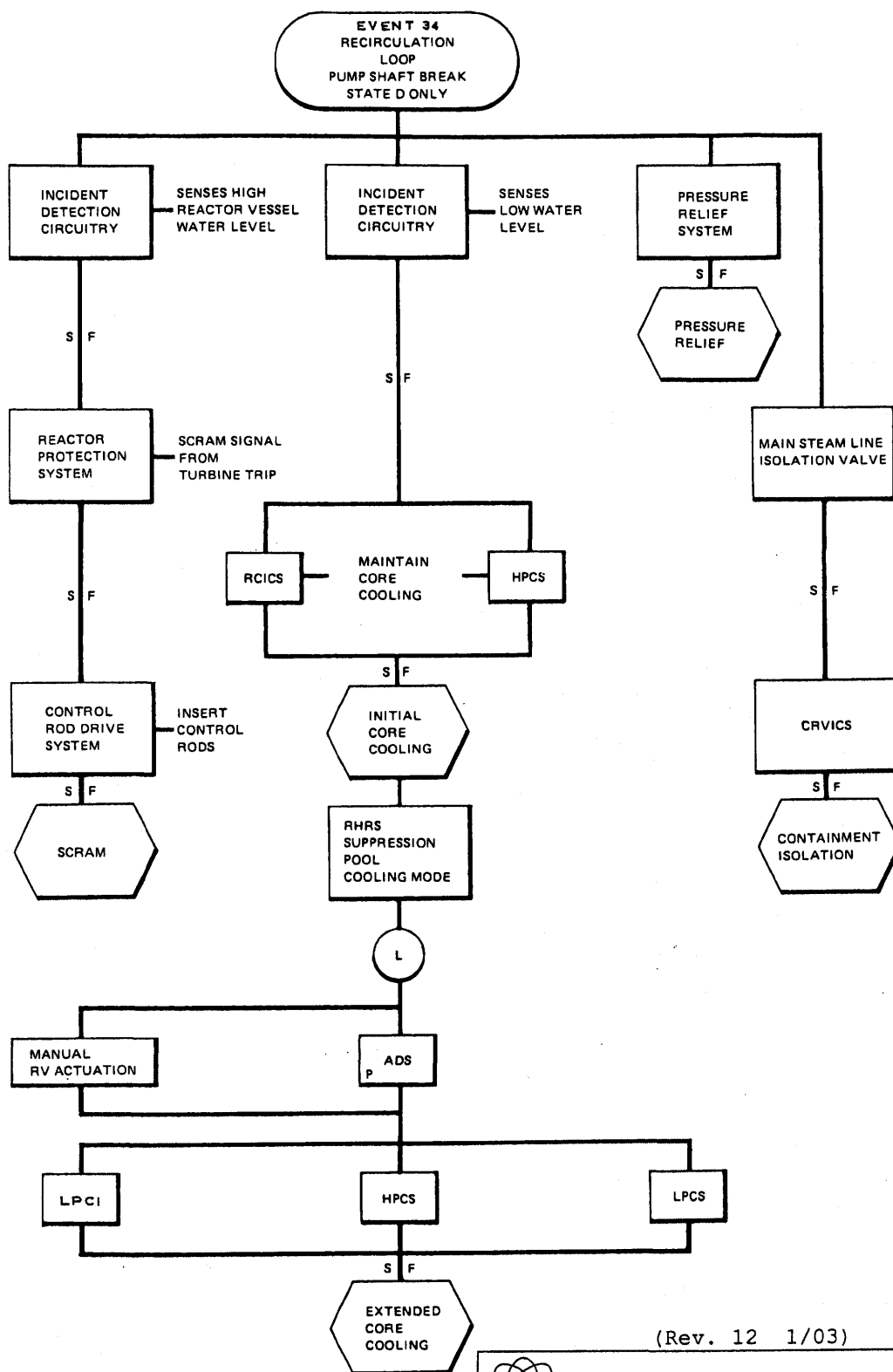
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PERRY NUCLEAR POWER PLANT

Protective Sequence for
Recirculation Loop Pump Seizure

Figure 15A.6-33



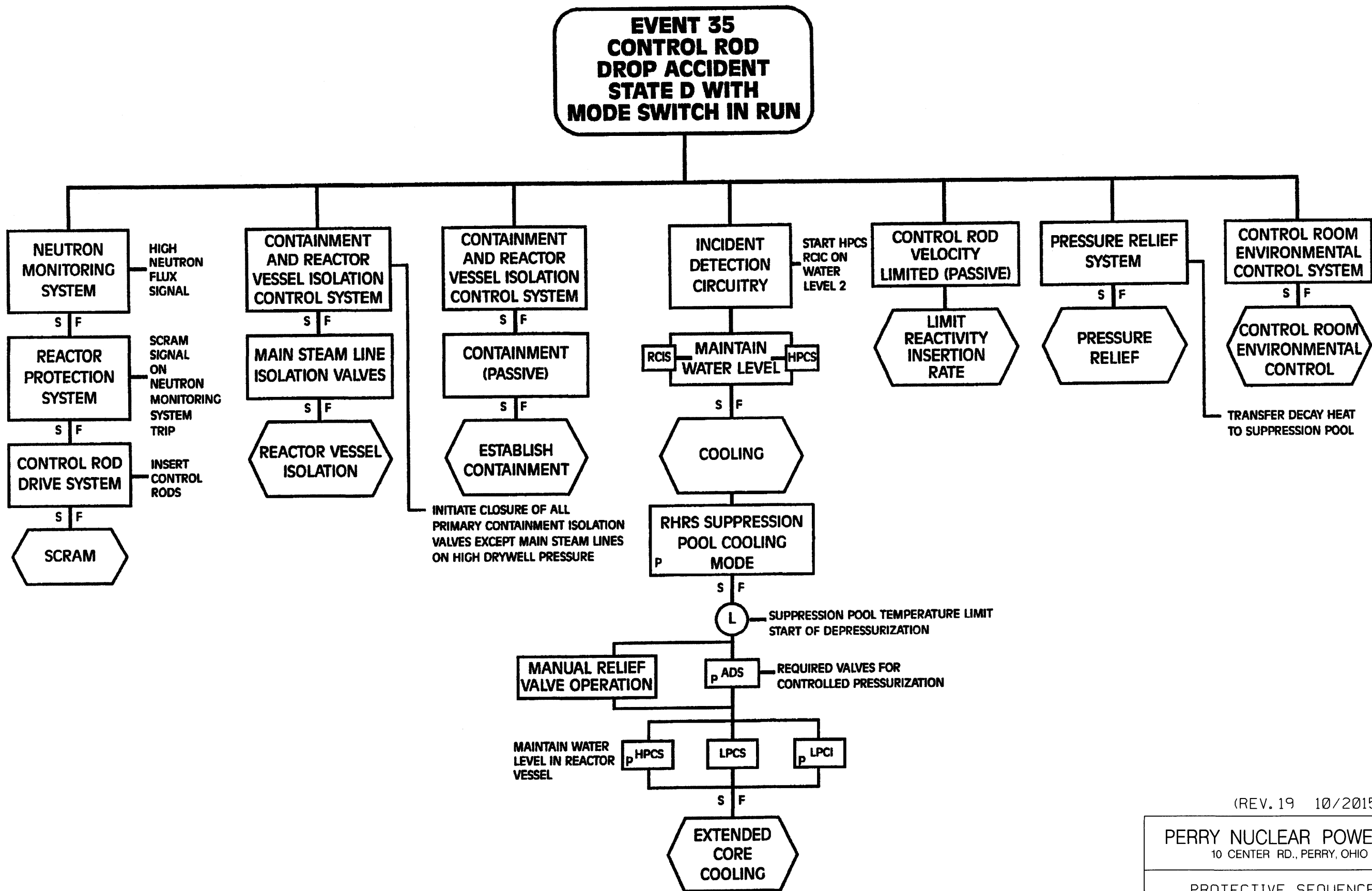
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Protective Sequence for
Recirculation Loop Pump
Shaft Break

Figure 15A.6-34

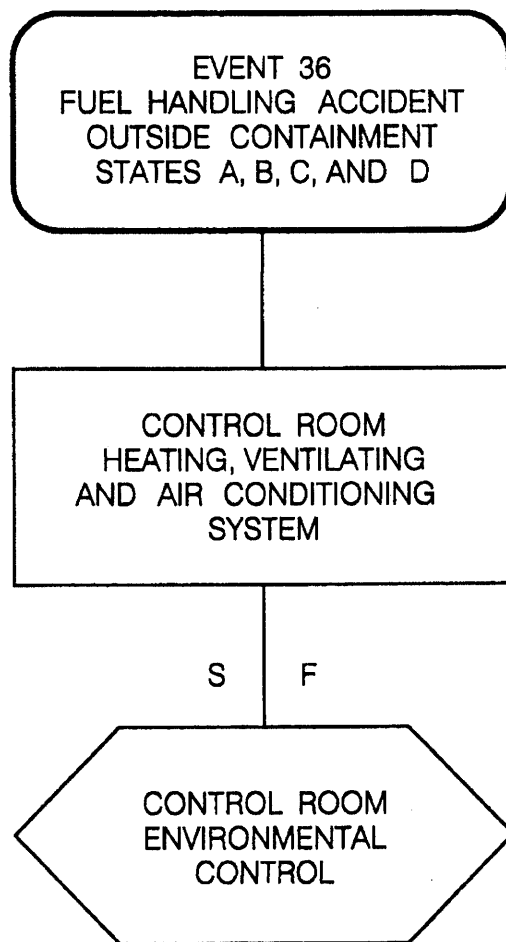


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PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

PROTECTIVE SEQUENCES FOR
CONTROL ROD DROP ACCIDENT

FIGURE 15A.6-35



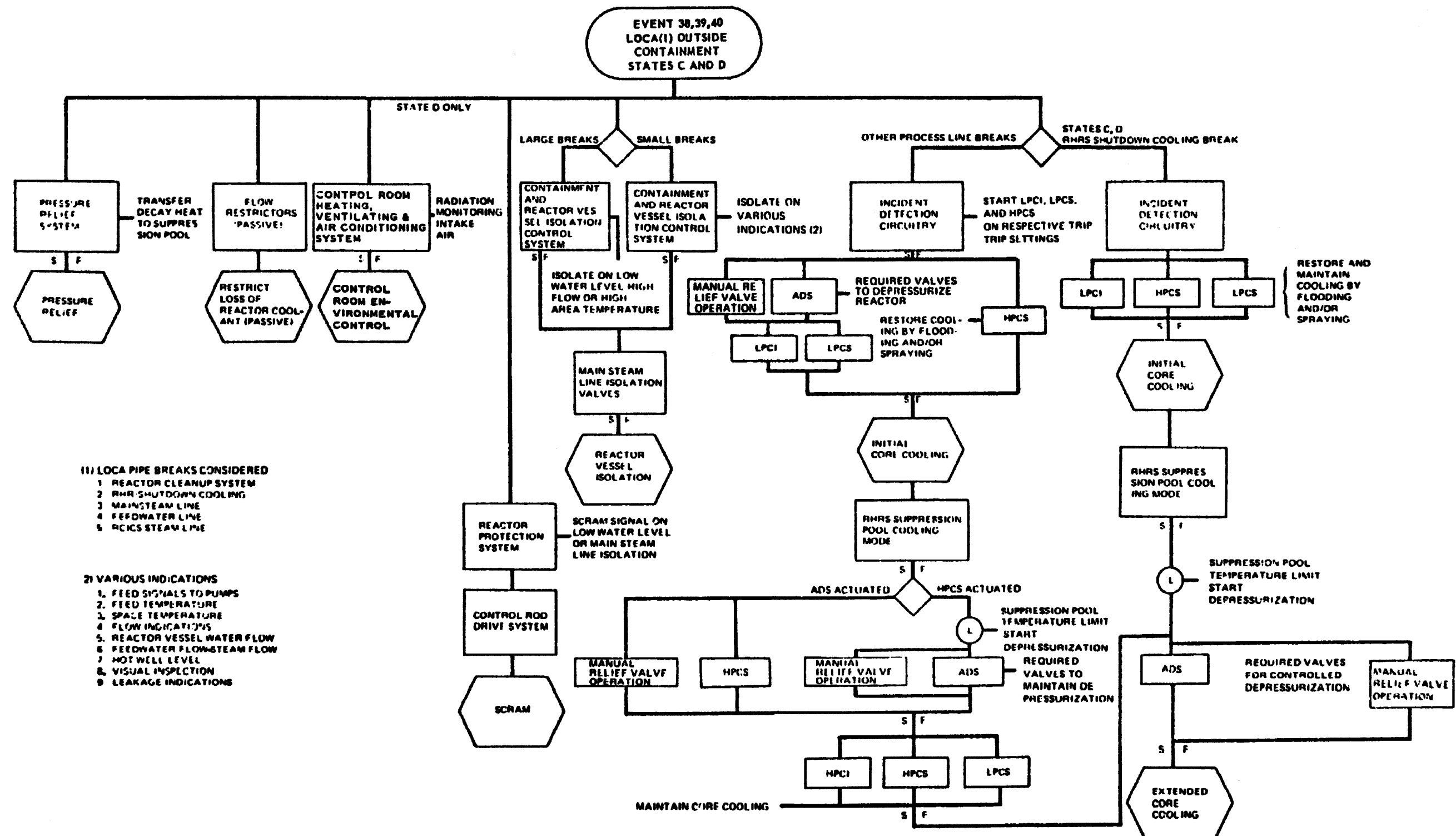
(Rev. 13 12/03)



PERRY NUCLEAR POWER PLANT

Protective Sequences for Fuel
Handling Accident
Outside Containment

Figure 15A.6-36

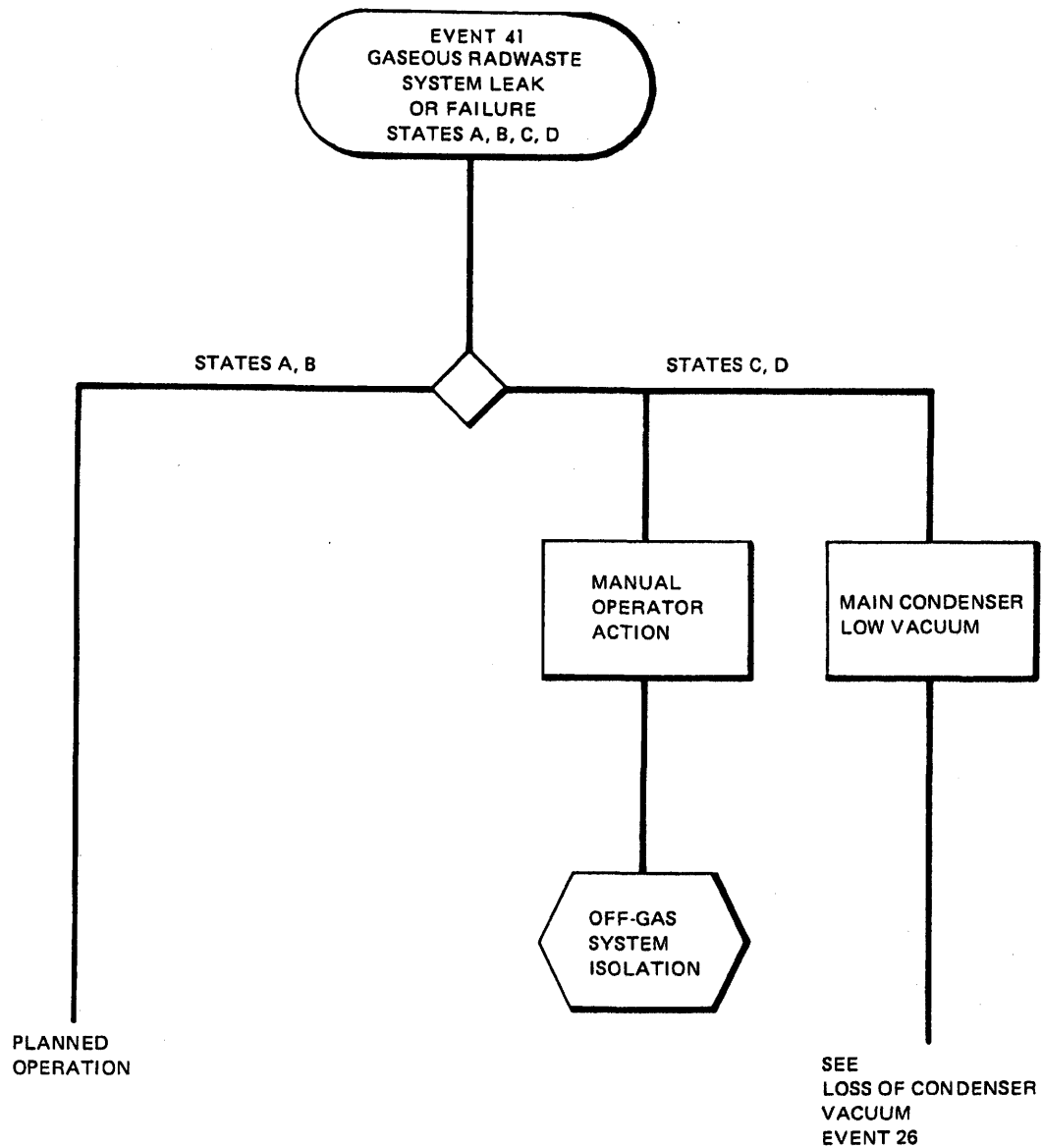


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PERRY NUCLEAR POWER PLANT
10 CENTER RD., PERRY, OHIO 44081

PROTECTIVE SEQUENCES FOR LIQUID,
STEAM, LARGE, SMALL PIPING
BREAKS OUTSIDE CONTAINMENT

FIGURE 15A.6-38



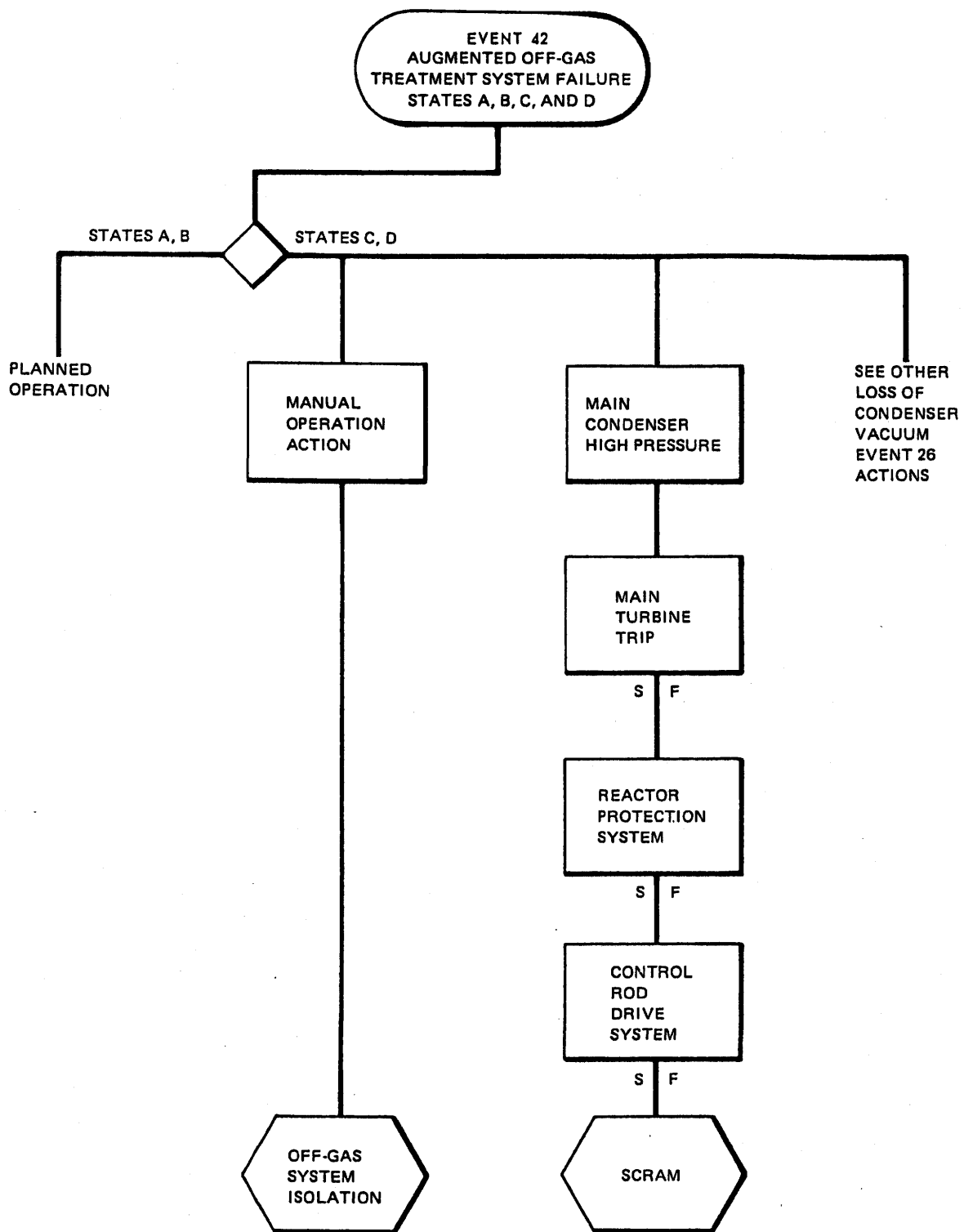
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PERRY NUCLEAR POWER PLANT

Protective Sequences for Gaseous
Radwaste System Leak or Failure

Figure 15A.6-39



(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Protective Sequences Augmented
Offgas Treatment System Failure

Figure 15A.6-40

EVENT 43
RADIOACTIVE LIQUID WASTE
SYSTEM FAILURES
(RELEASE TO ATMOSPHERE)
STATES A, B, C, D

HIGH RADIATION
PROCESS VENTI-
LATION RADIATION
MONITORING
SUBSYSTEM

VENTILATION
SYSTEM
CONTROL

RESTRICT
BUILDING
ACCESS

(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Protective Sequences Augmented
Liquid Radwaste System Failures

Figure 15A.6-41

EVENT 44
POSTULATED RADIOACTIVE
LIQUID RELEASES DUE TO
LIQUID-CONTAINING TANK FAIL-
URES STATES A, B, C, D

HIGH WATER
FLOOR DRAIN
MONITORING
SYSTEM

SUMP
PUMP
SYSTEM

CONTAIN
LIQUID
EFFLUENT

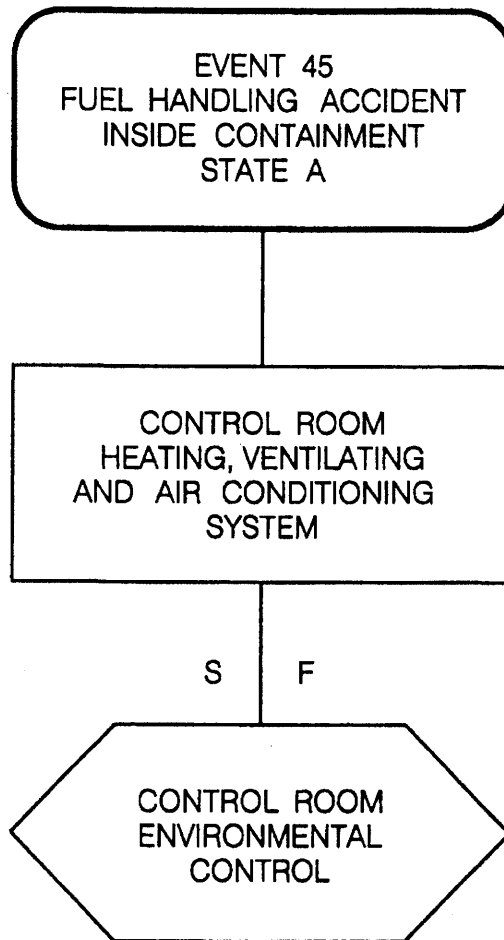
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PERRY NUCLEAR POWER PLANT

Protective Sequence for Liquid
Radwaste System Storage Tank
Failures

Figure 15A.6-42



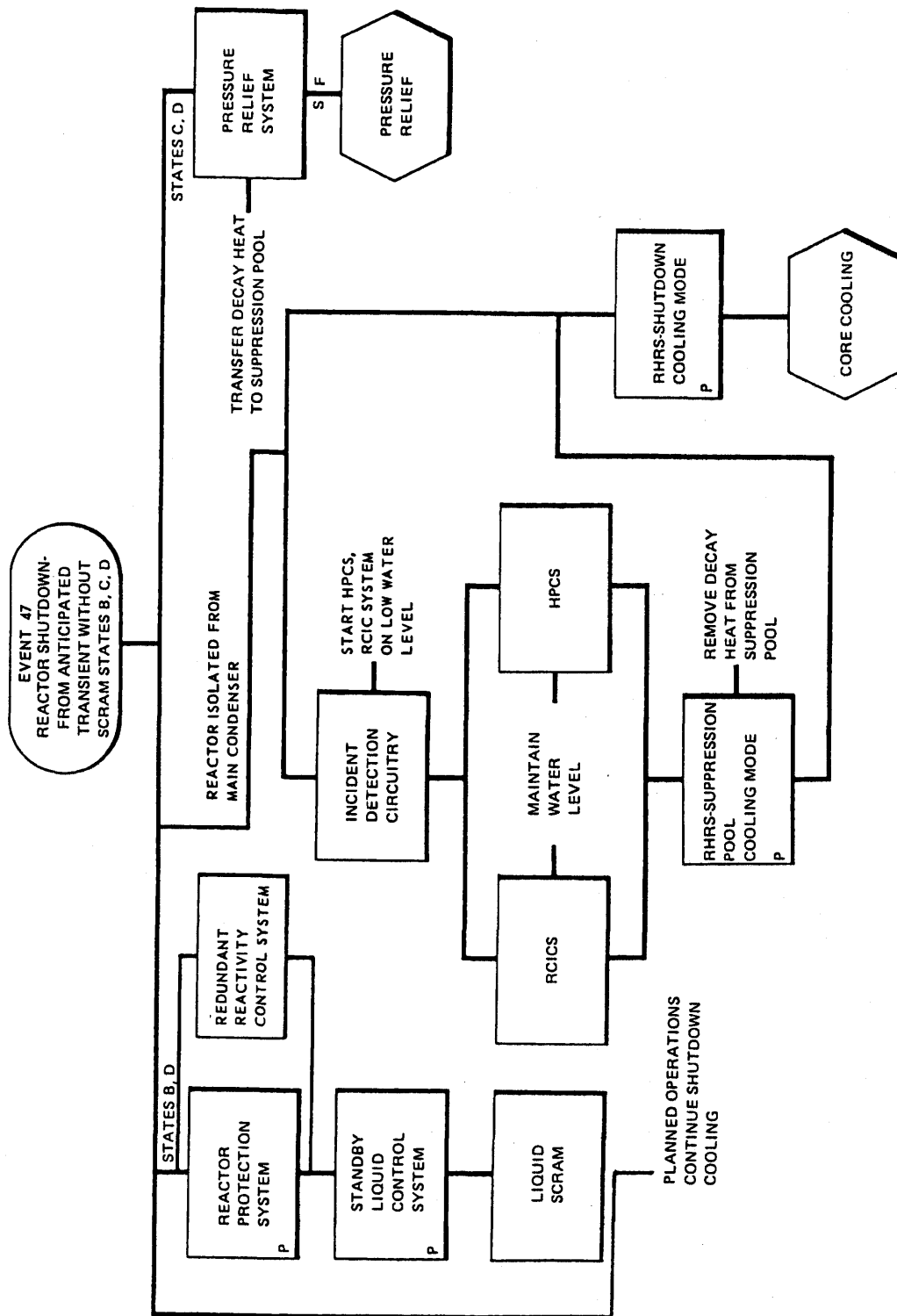
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PERRY NUCLEAR POWER PLANT

Protective Sequences for Fuel
Handling Accident Inside
Containment

Figure 15A.6-43



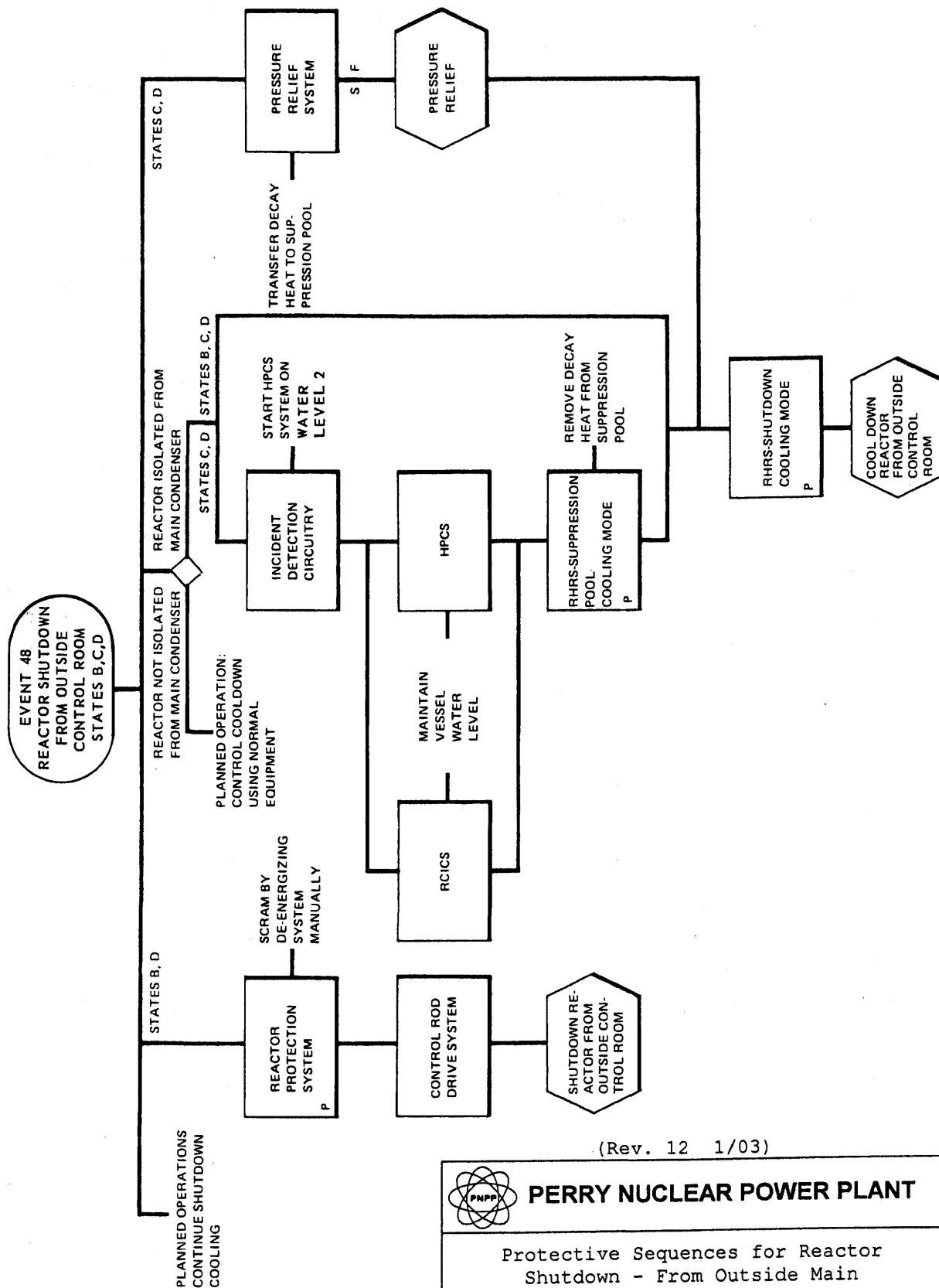
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PERRY NUCLEAR POWER PLANT

Protective Sequence for
Reactor Shutdown From Anticipated
Transient Without Scram

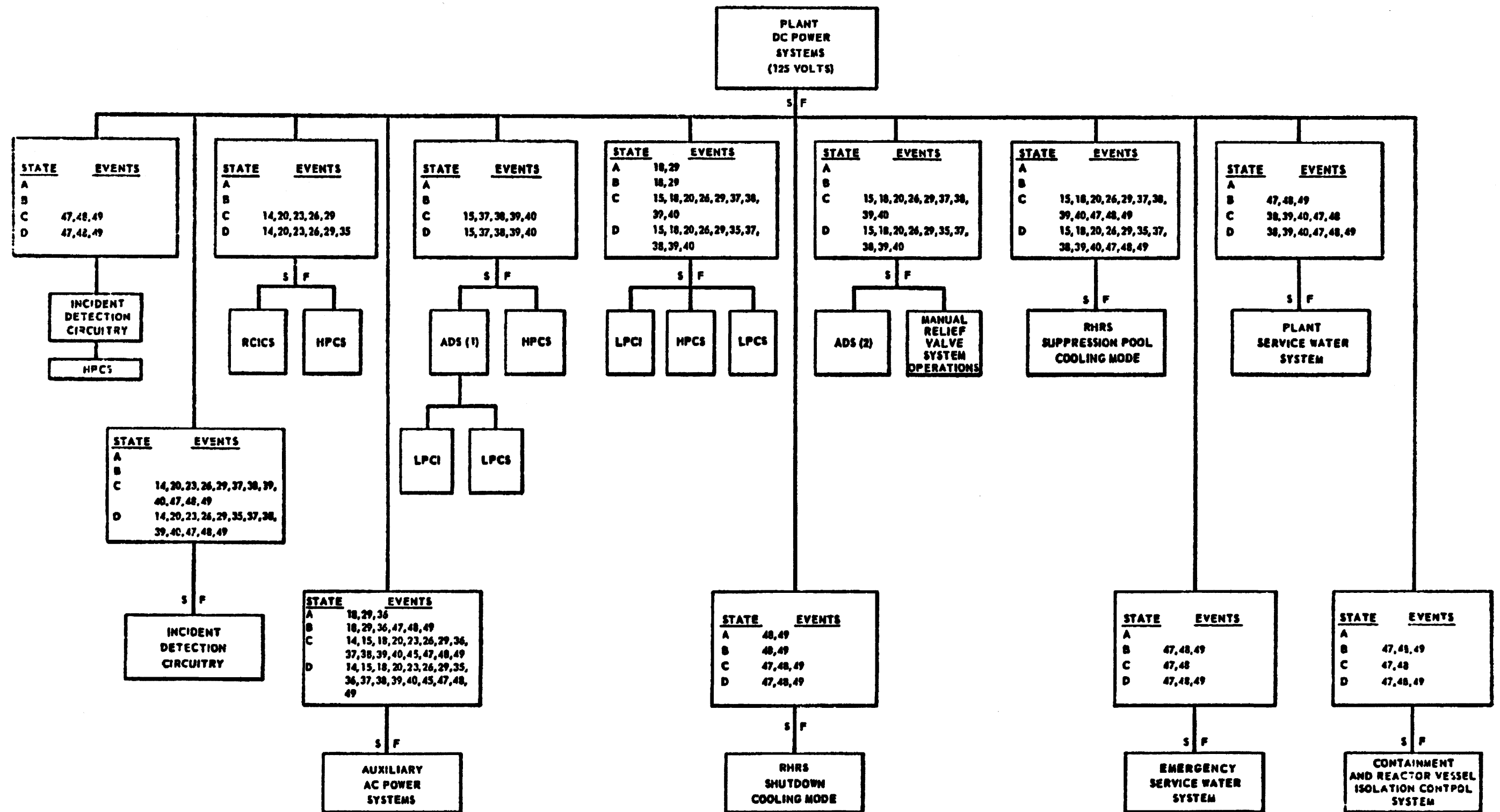
Figure 15A.6-44



PERRY NUCLEAR POWER PLANT

Protective Sequences for Reactor
Shutdown - From Outside Main
Control Room

Figure 15A.6-45



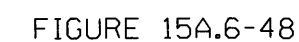
(1) BLOWDOWN.
 (2) CONTROLLED DEPRESSURIZATION.
 (3) SF REQUIREMENT NOT APPLICABLE IN EVENT 51,52,53.

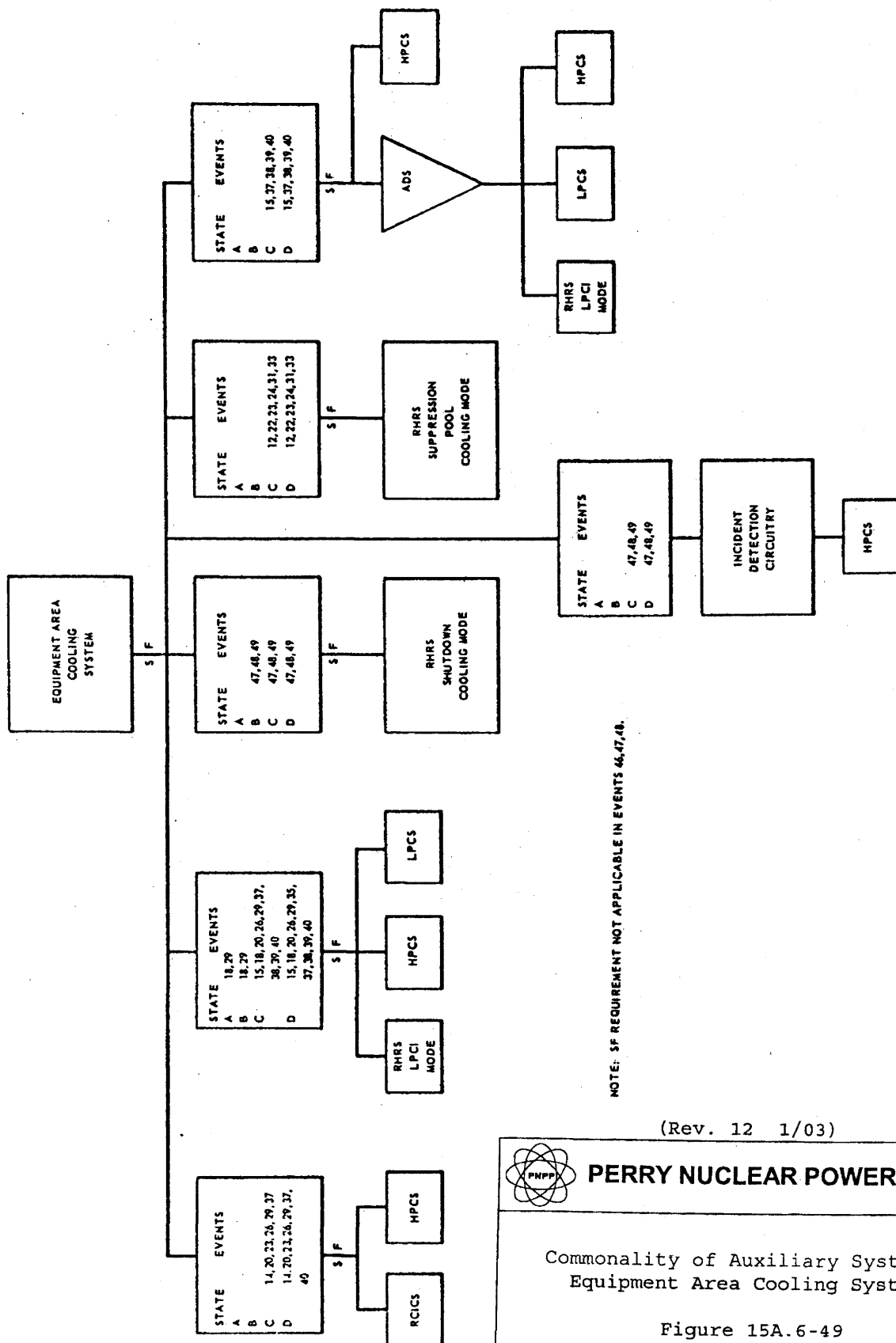
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PERRY NUCLEAR POWER PLANT
 10 CENTER RD., PERRY, OHIO 44081

COMMONALITY OF AUXILIARY SYSTEMS -
 DC POWER SYSTEMS (125 VOLTS)

FIGURE 15A.6-47





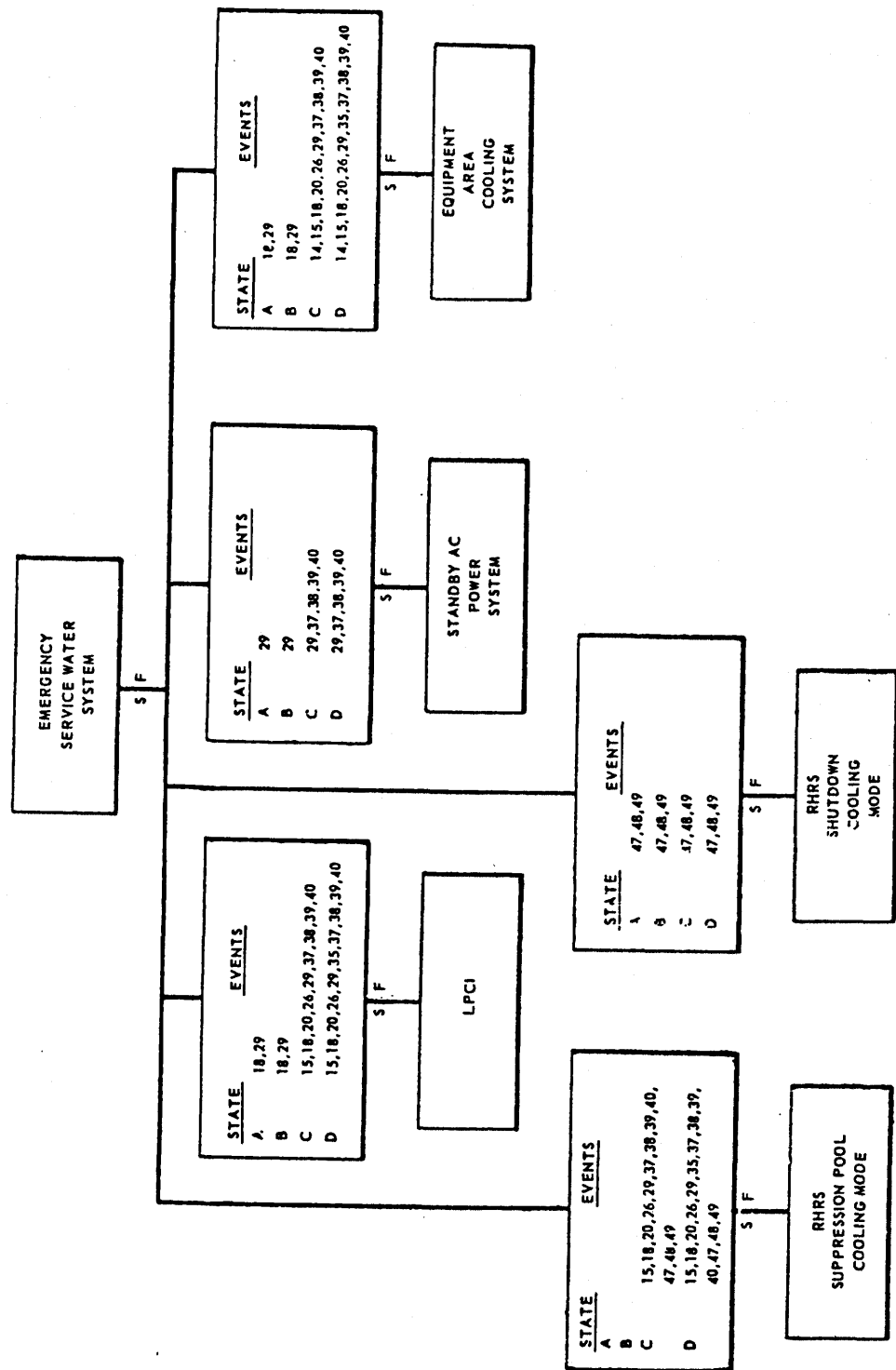
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Commonality of Auxiliary Systems -
Equipment Area Cooling Systems

Figure 15A.6-49



NOTE: SF REQUIREMENT NOT APPLICABLE IN EVENTS 46,47,41.

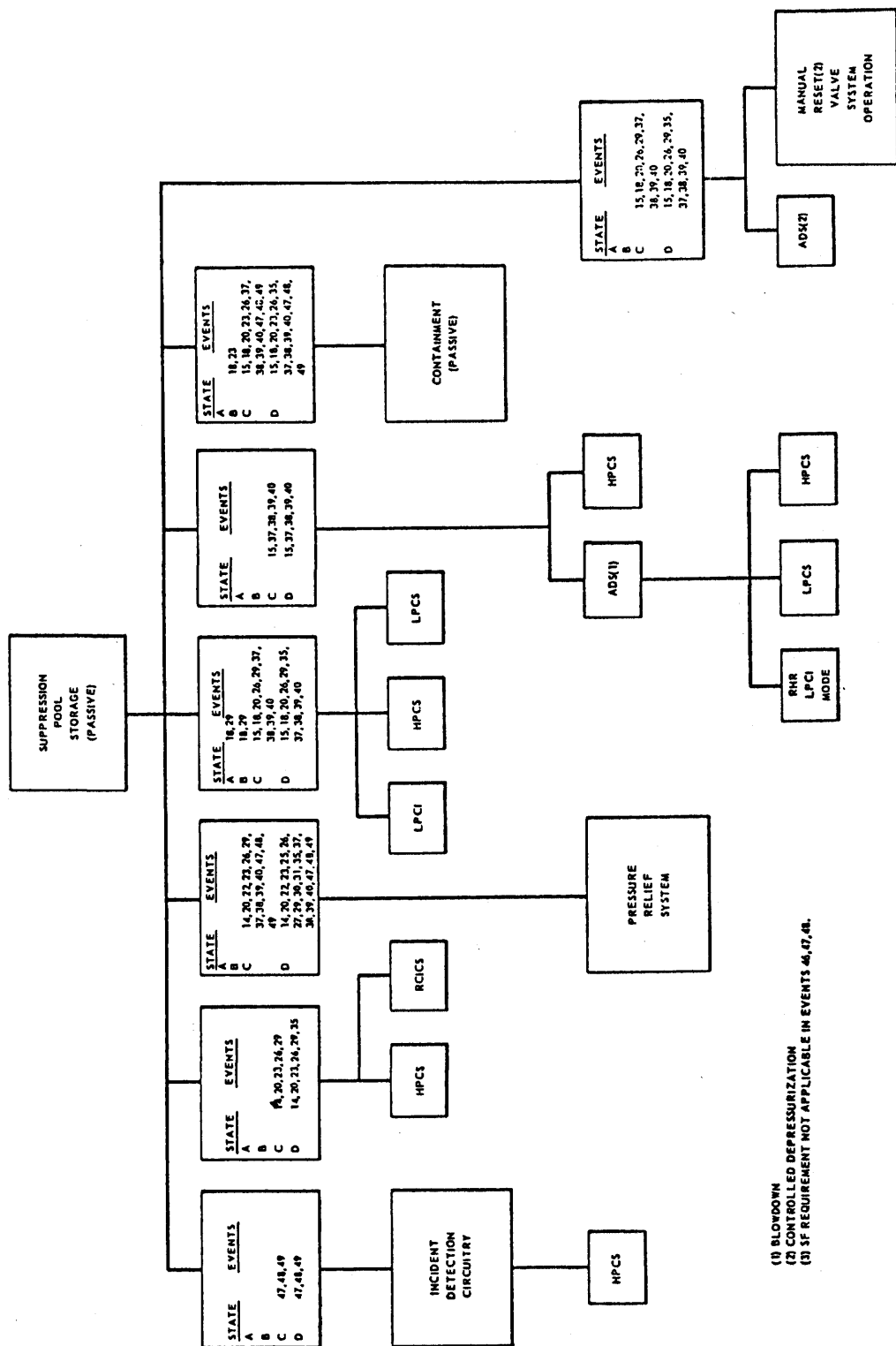
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PERRY NUCLEAR POWER PLANT

Commonality of Auxiliary Systems -
Plant Service Water System

Figure 15A.6-50



(1) BLOWDOWN
(2) CONTROLLED DEPRESSURIZATION
(3) SF REQUIREMENT NOT APPLICABLE IN EVENTS 46, 47, 48.

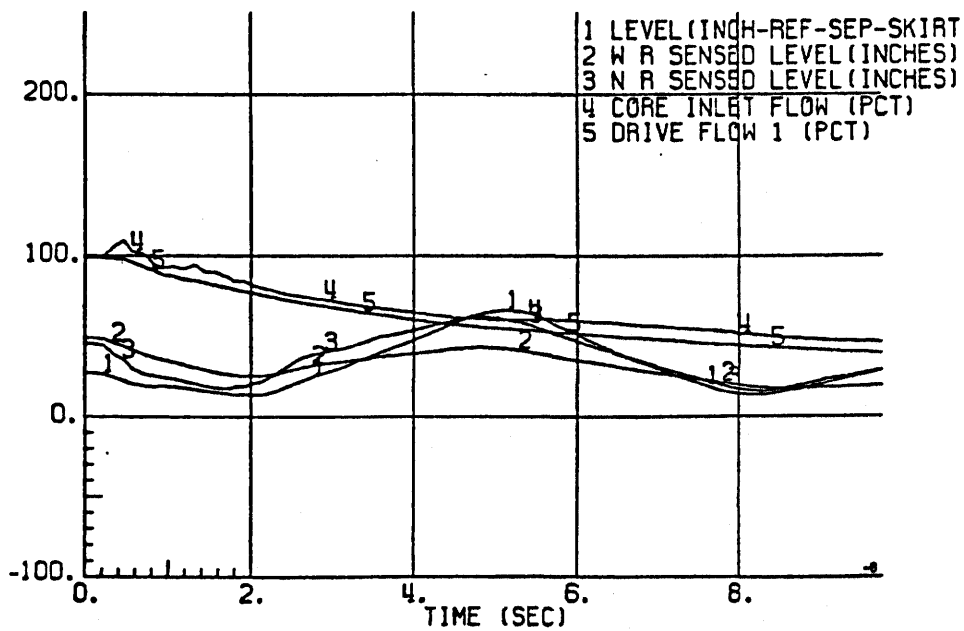
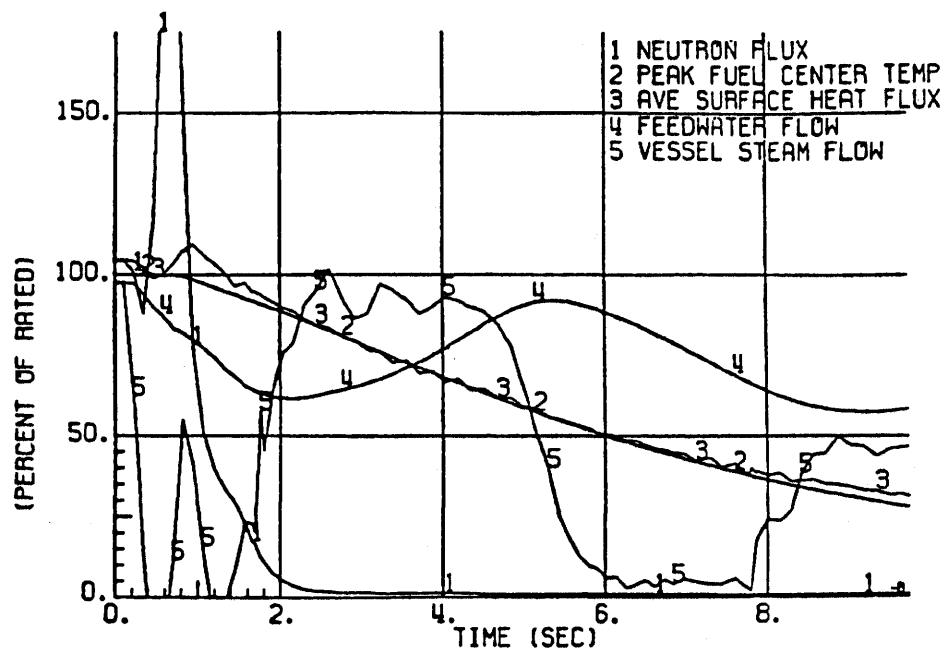
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PERRY NUCLEAR POWER PLANT

Commonality of Auxiliary Systems -
Suppression Pool Storage

Figure 15A.6-51



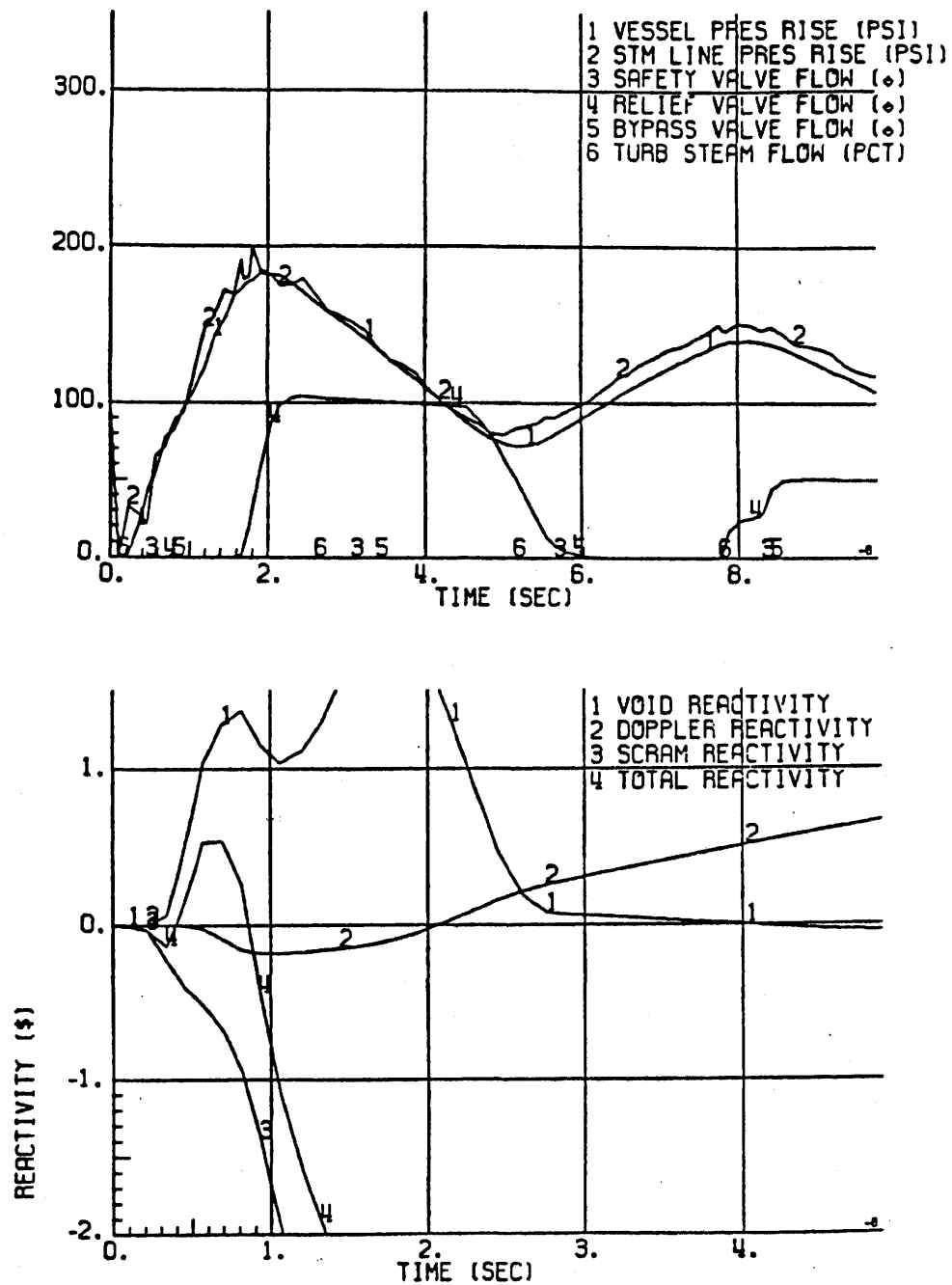
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection with
Bypass Failure 3729 MWt Core Power
100% Flow 370°F Rated FWT

Figure 15D-1 (Sheet 1 of 2)



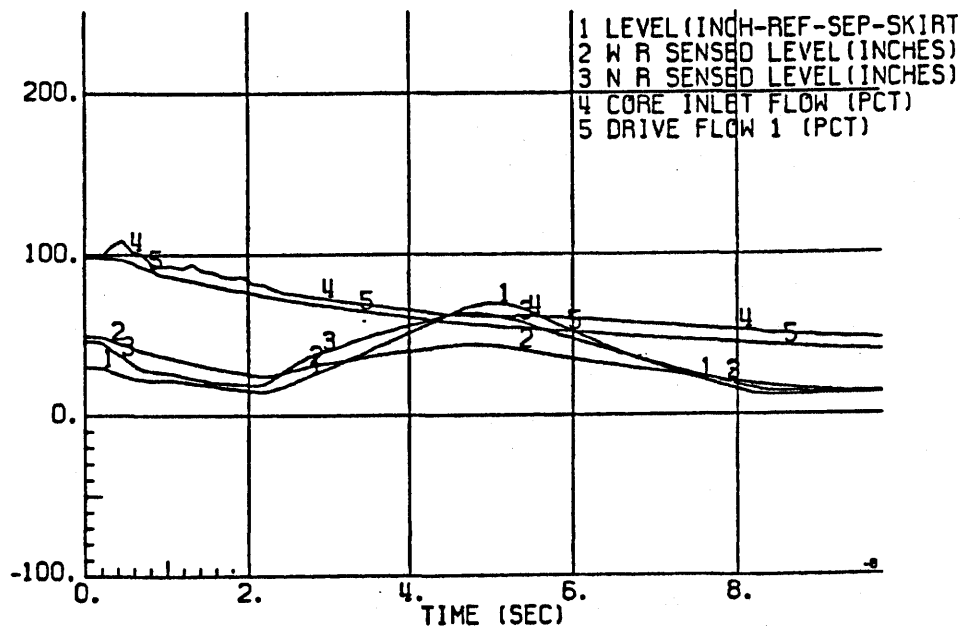
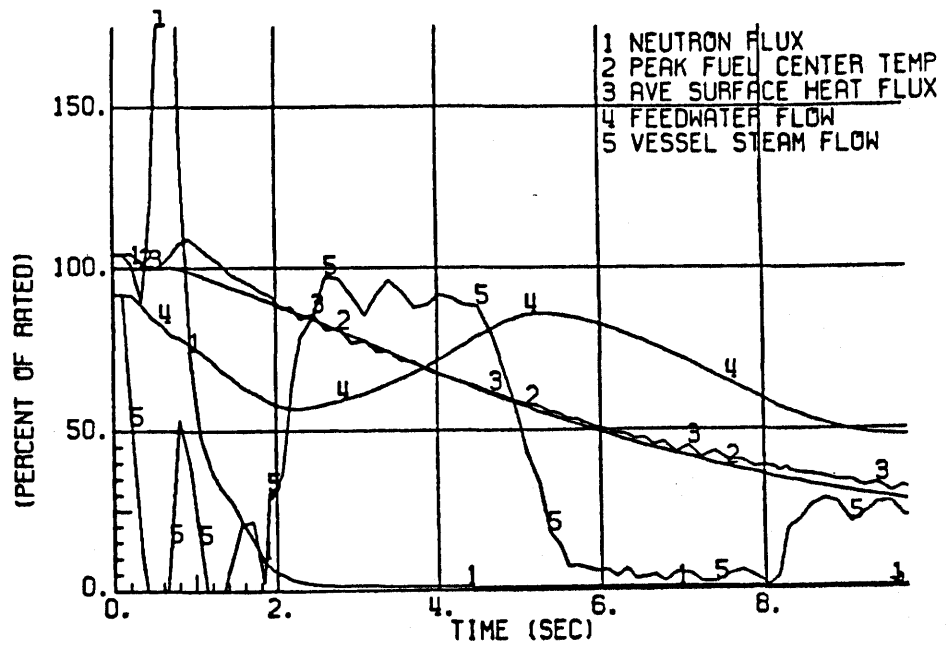
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection with
Bypass Failure 3729 MWt Core Power
100% Flow 370°F Rated FWT

Figure 15D-1 (Sheet 2 of 2)



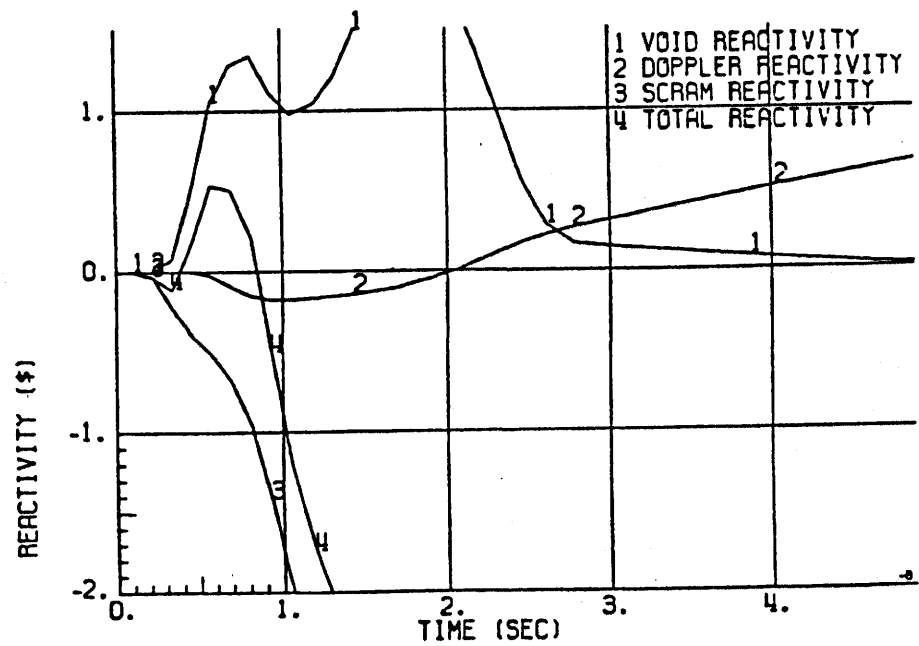
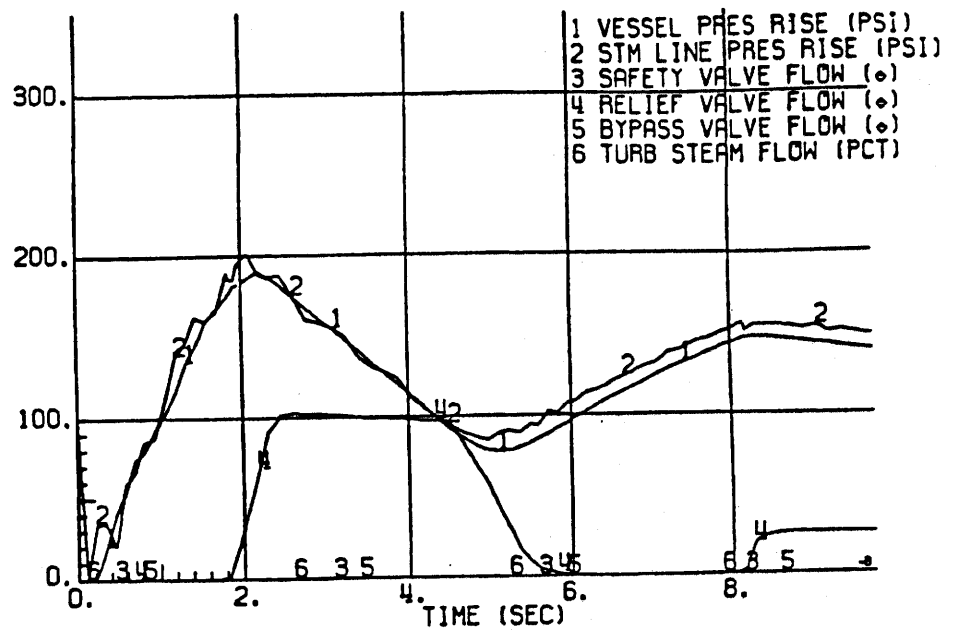
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection with
Bypass Failure 3729 MWt Core Power
100% Flow 320°F Rated FWT

Figure 15D-2 (Sheet 1 of 2)



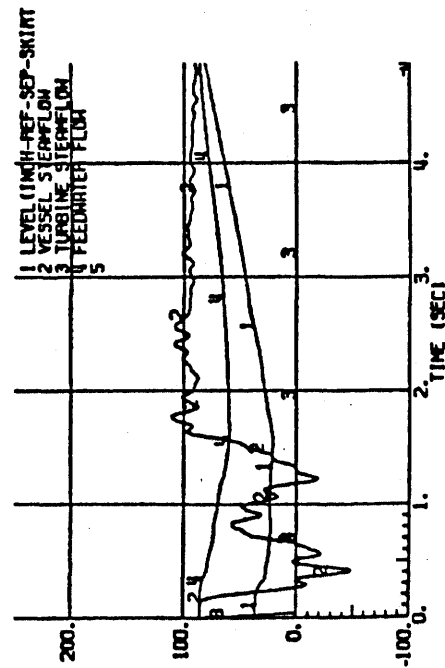
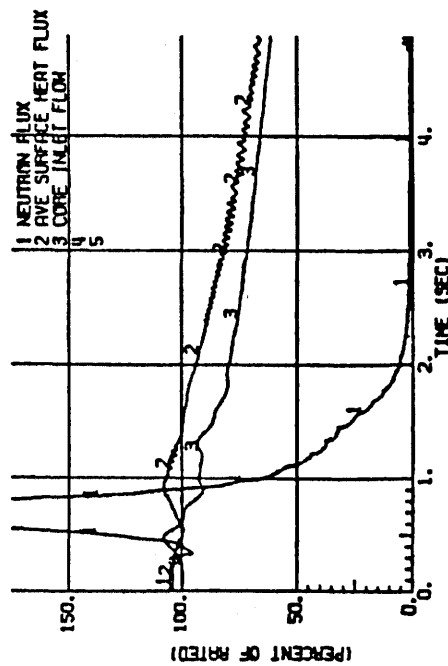
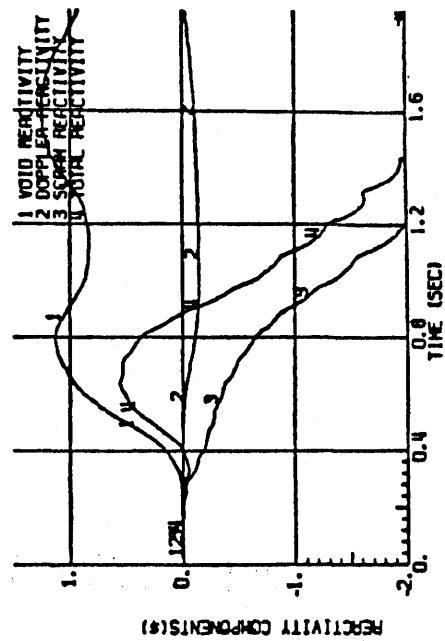
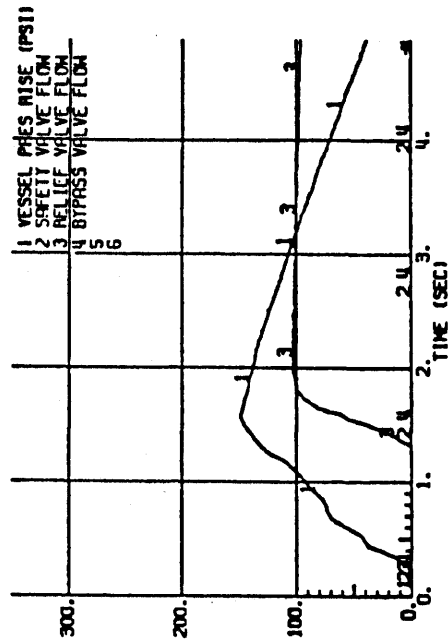
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection with
 Bypass Failure 3729 MWt Core Power
 100% Flow 320°F Rated FWT

Figure 15D-2 (Sheet 2 of 2)



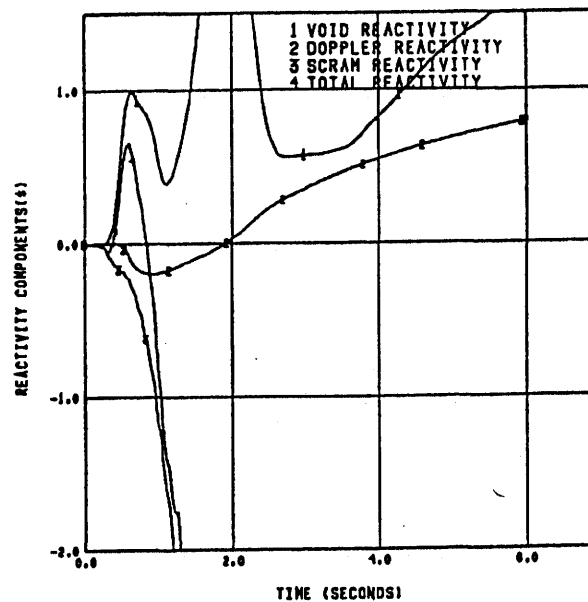
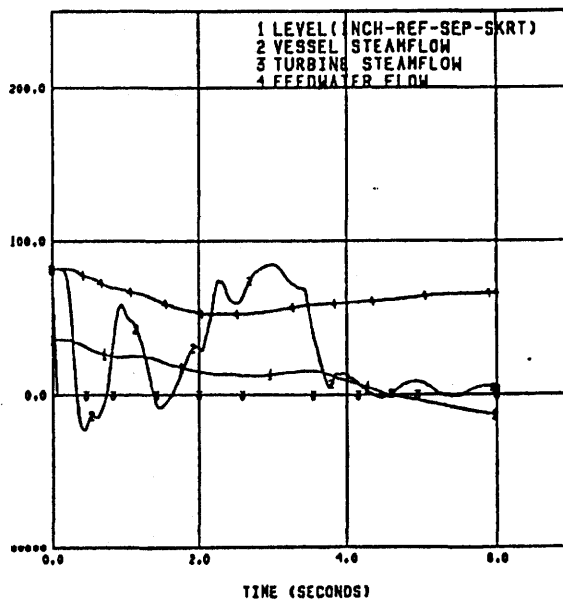
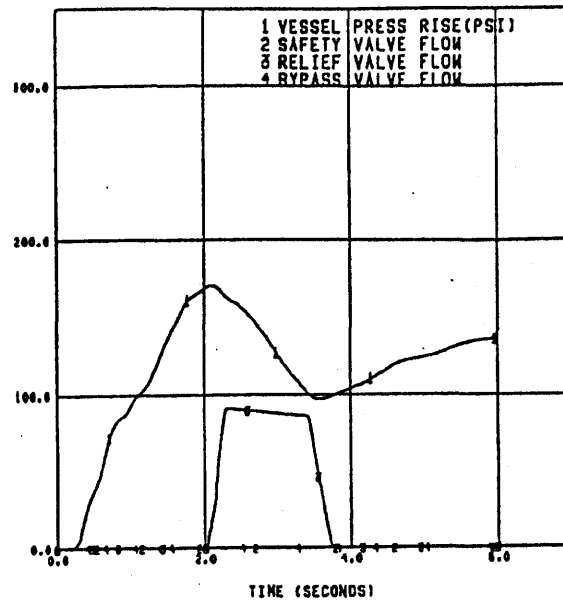
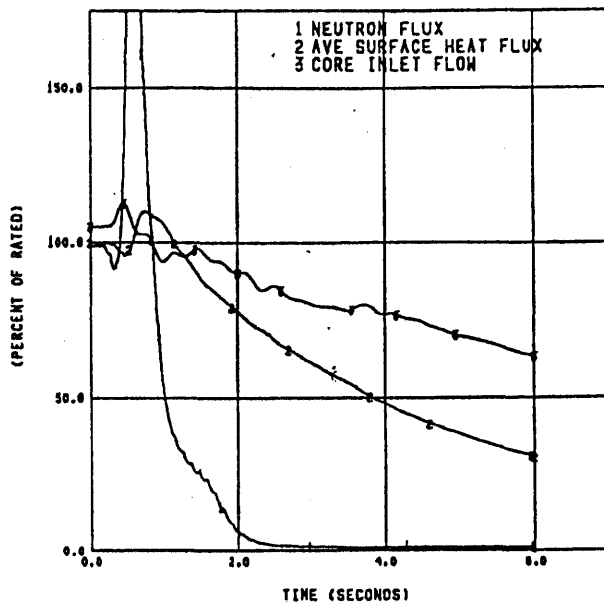
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection with
Bypass Failure 3729 MWt Core Power
100% Flow 250°F Rated FWT

Figure 15D-3a



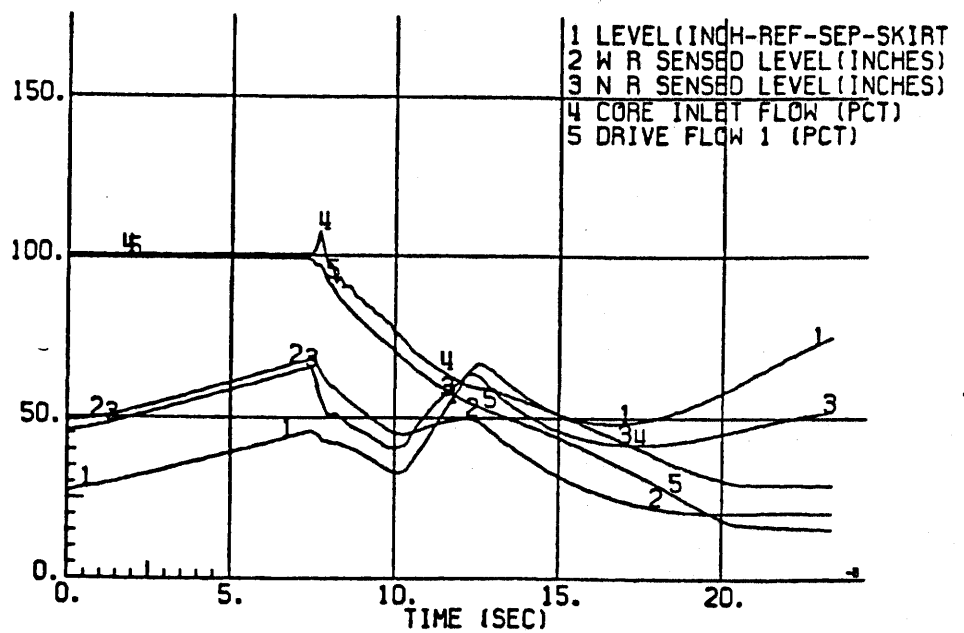
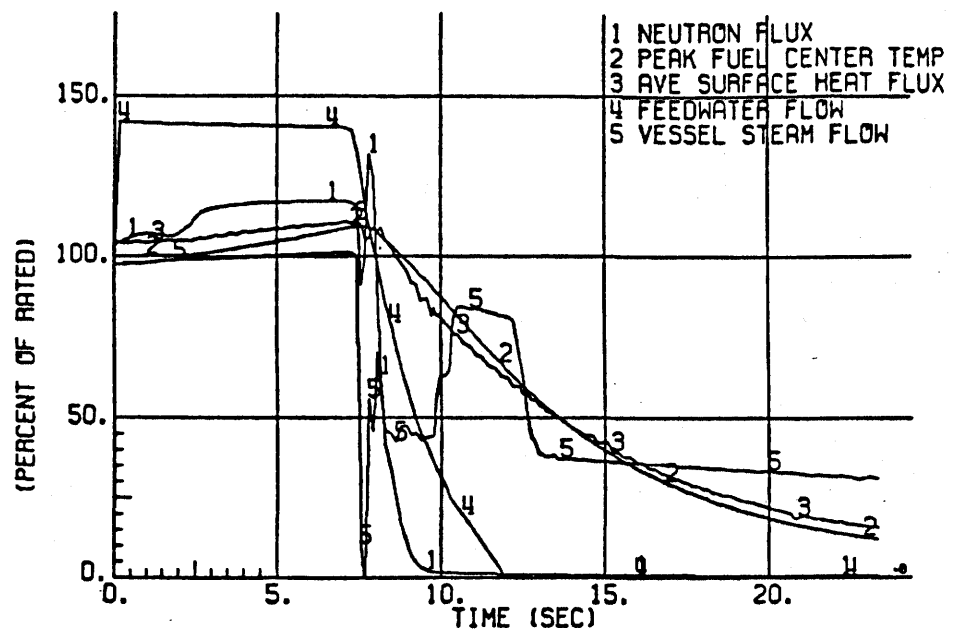
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection with
Bypass Failure for Power Uprate
100% Power, 105% Flow, 255.5°F FWT

Figure 15D-3b



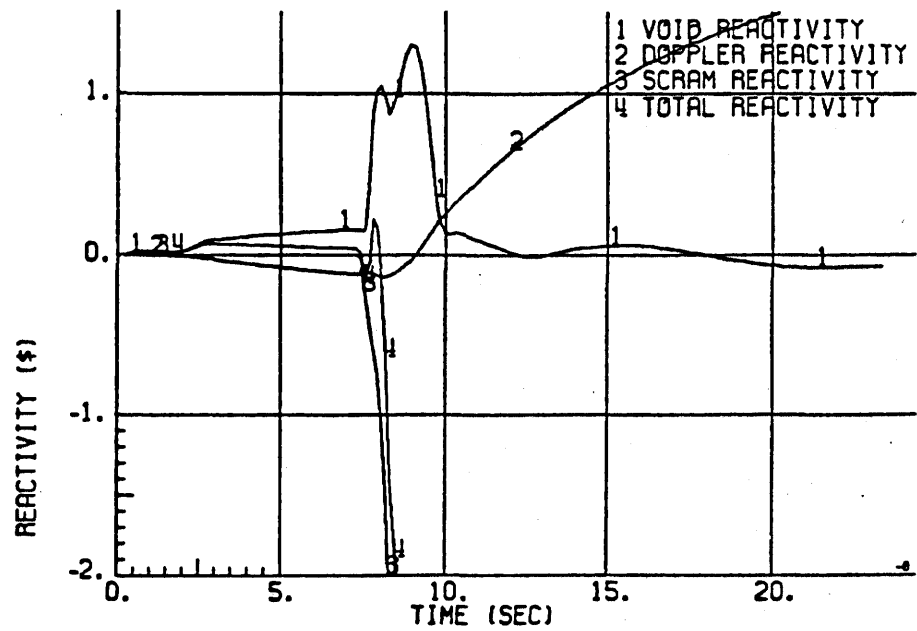
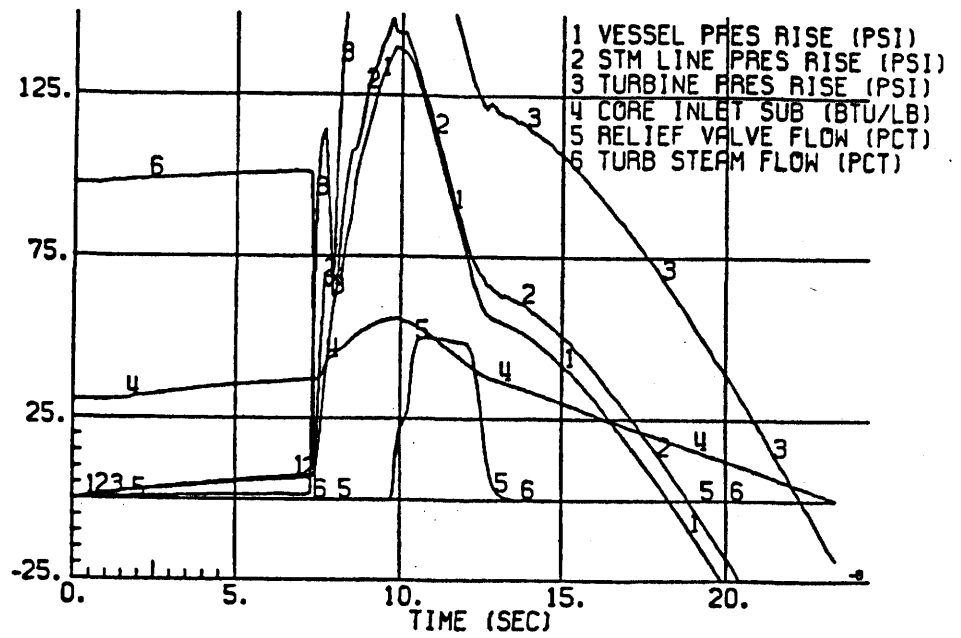
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
3729 MWt Core Power 100% Flow
370°F Rated FWT

Figure 15D-4 (Sheet 1 of 2)



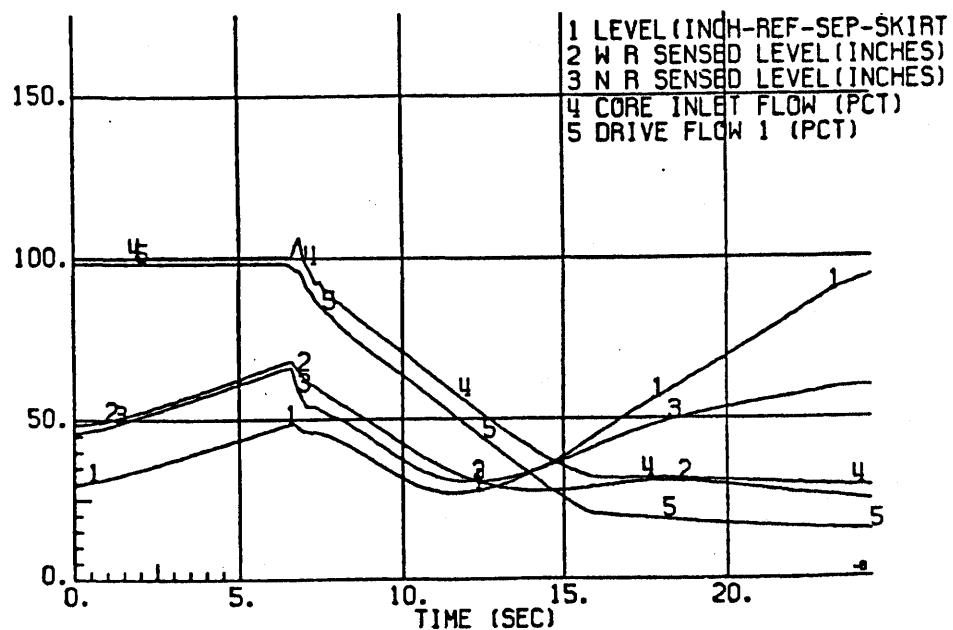
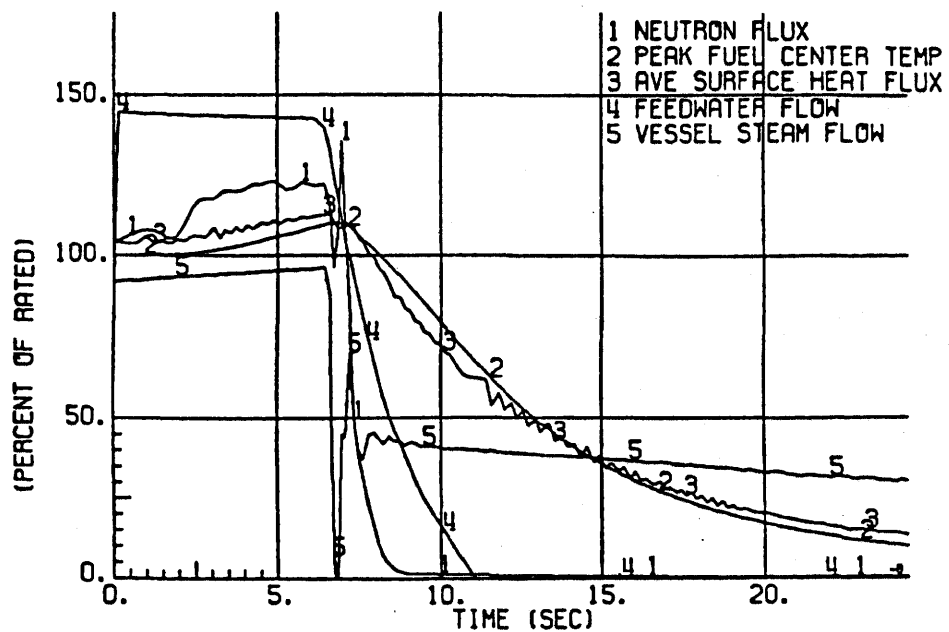
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 3729 MWt Core Power 100% Flow
 370°F Rated FWT

Figure 15D-4 (Sheet 2 of 2)



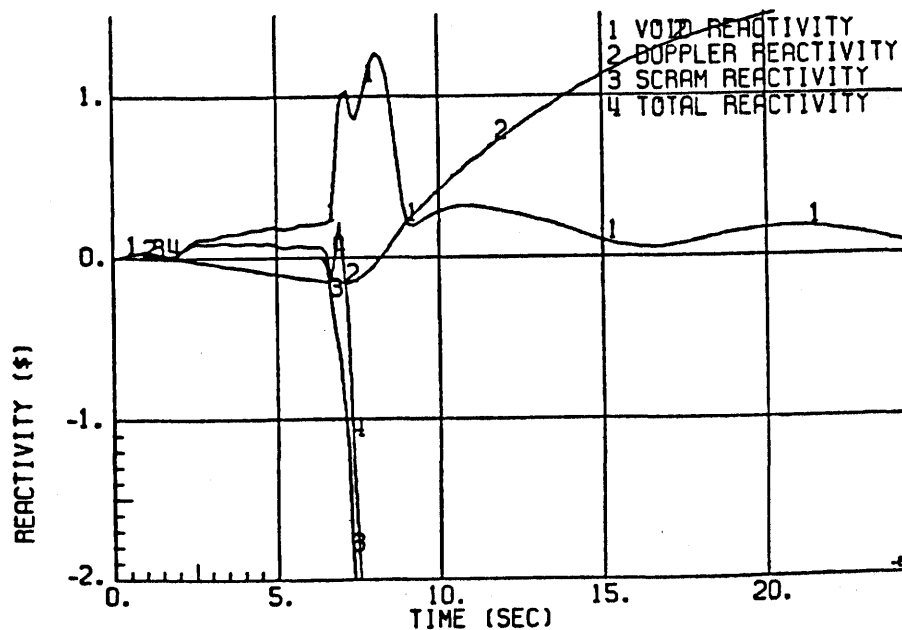
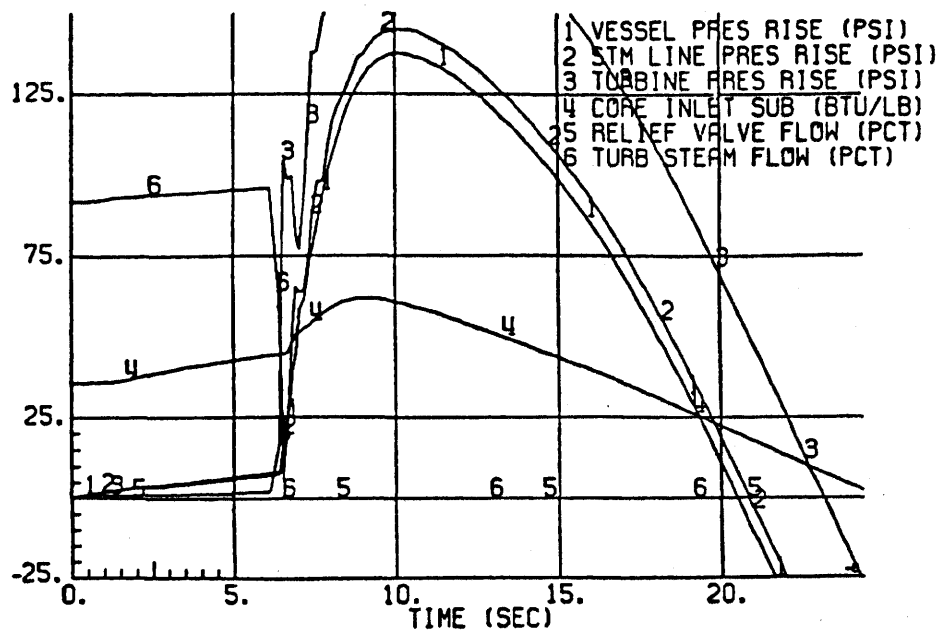
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
3729 MWt Core Power 100% Flow
320°F Rated FWT

Figure 15D-5 (Sheet 1 of 2)



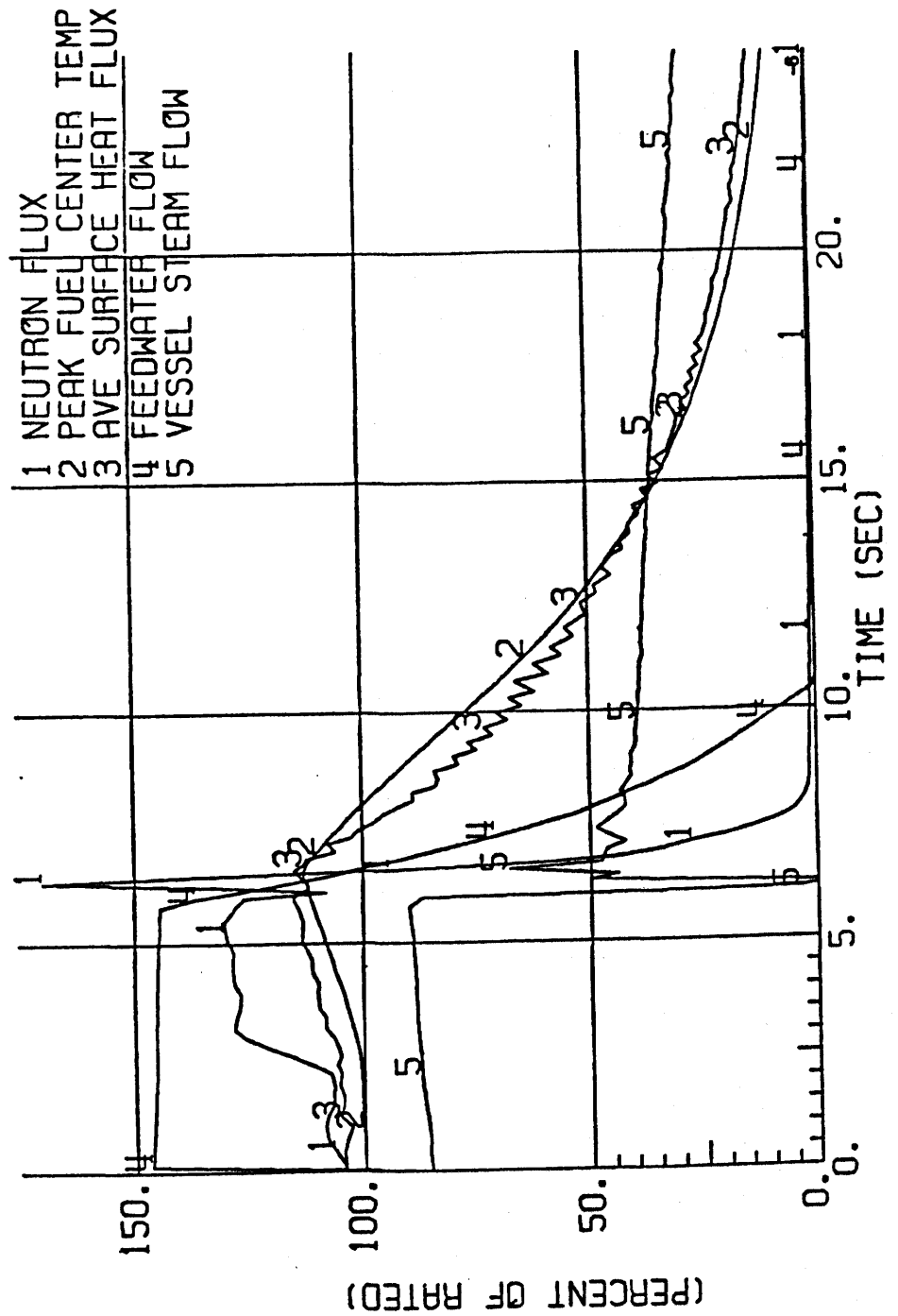
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 3729 MWt Core Power 100% Flow
 320°F Rated FWT

Figure 15D-5 (Sheet 2 of 2)



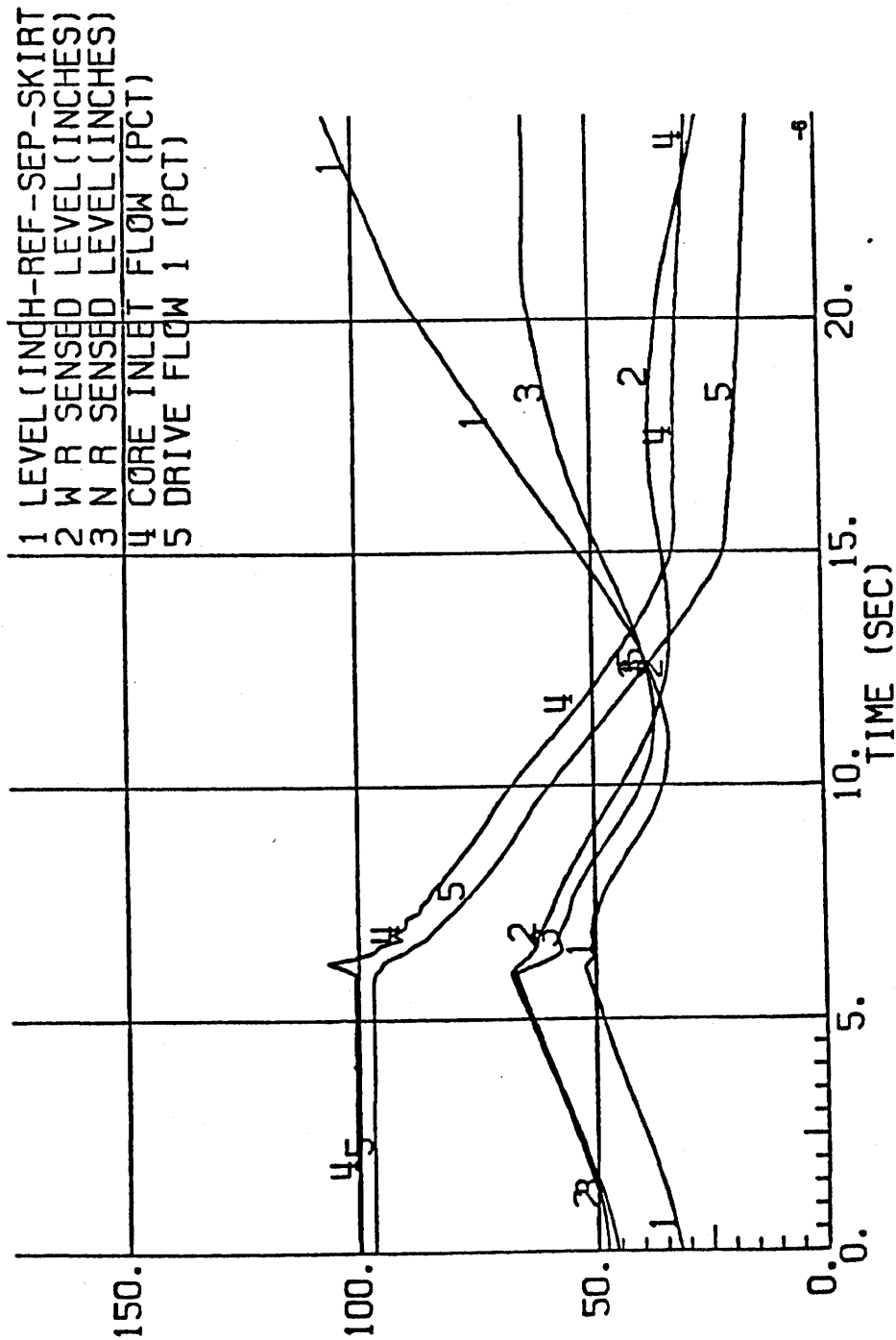
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
3729 MWt Core Power 100% Flow
250°F Rated FWT

Figure 15D-6a (Sheet 1 of 4)



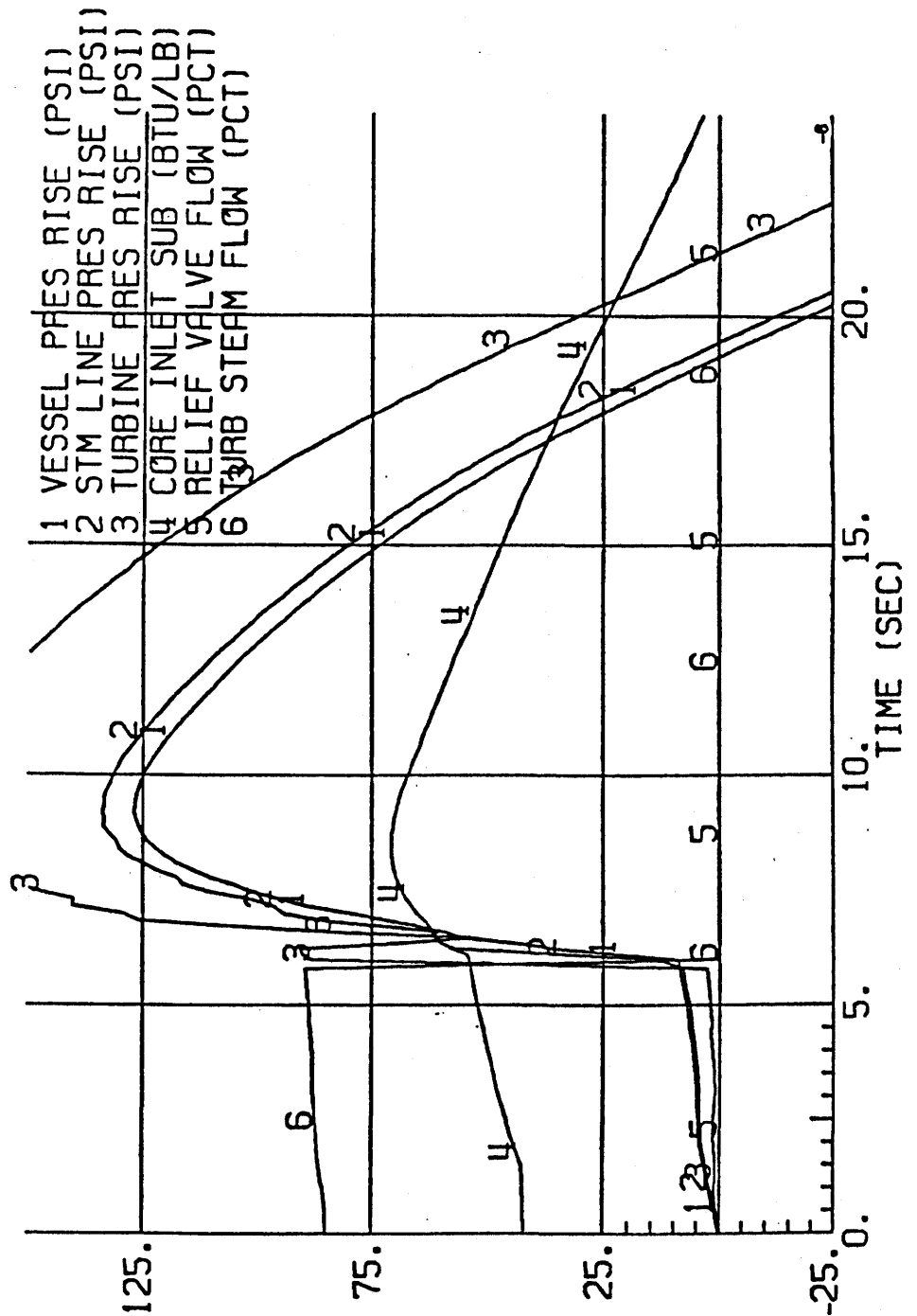
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 3729 MWt Core Power 100% Flow
 250°F Rated FWT

Figure 15D-6a (Sheet 2 of 4)



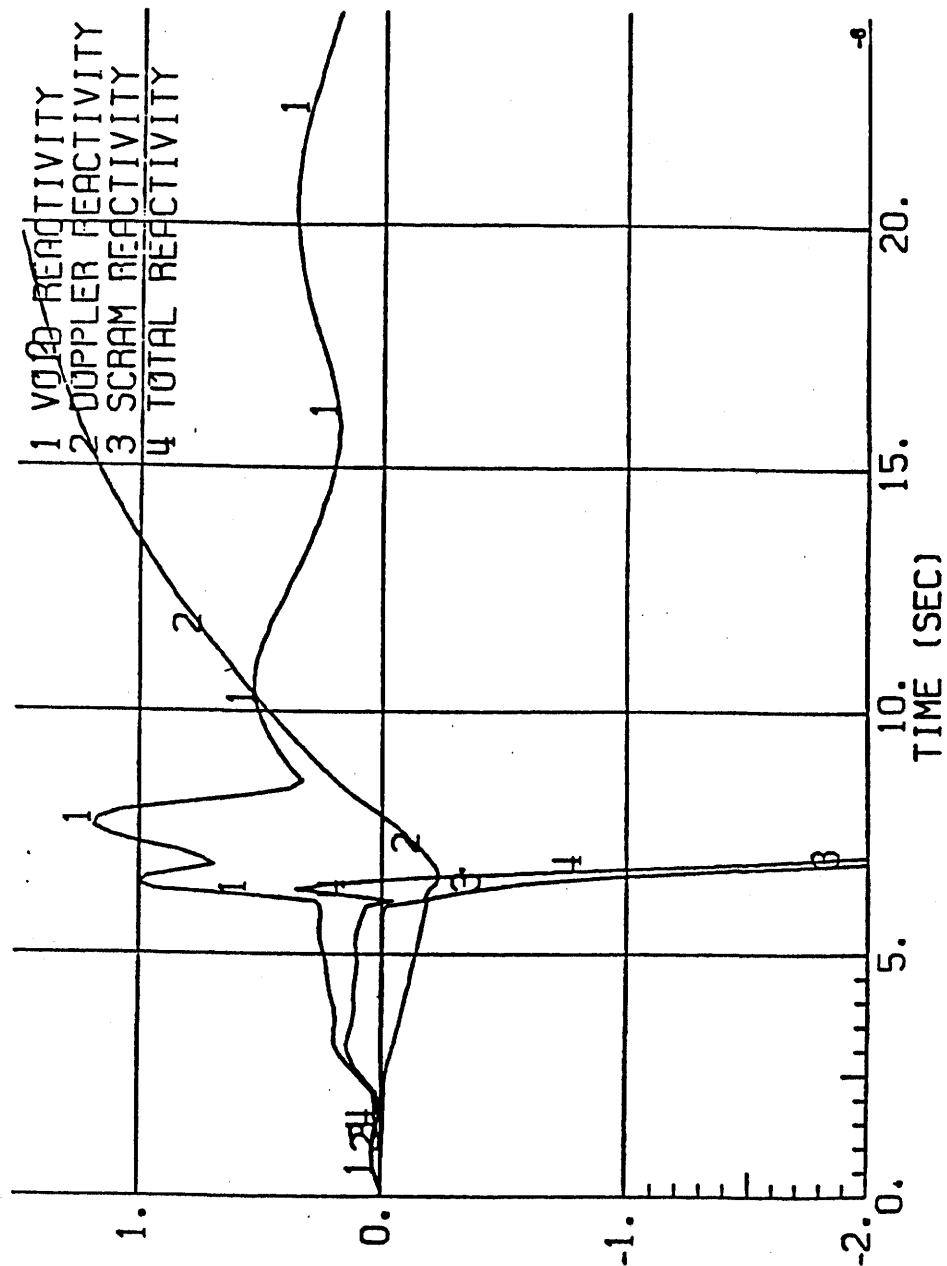
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 3729 MWt Core Power 100% Flow
 250°F Rated FWT

Figure 15D-6a (Sheet 3 of 4)



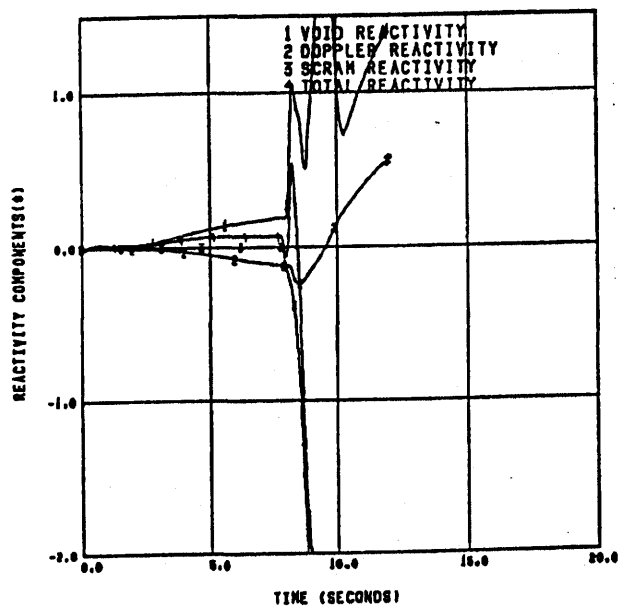
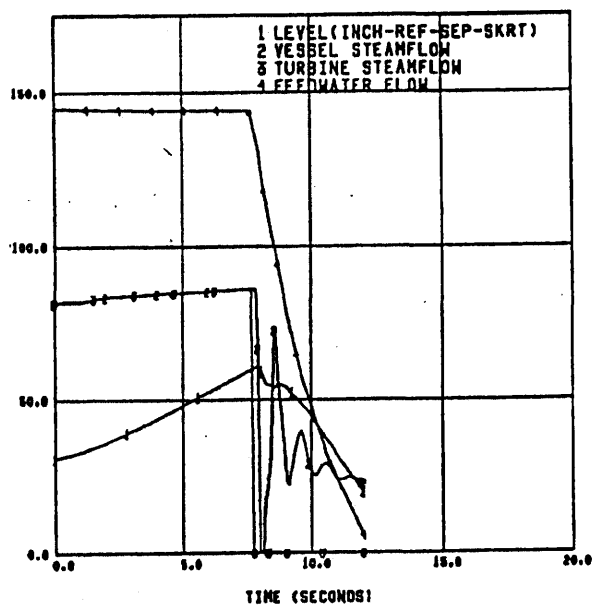
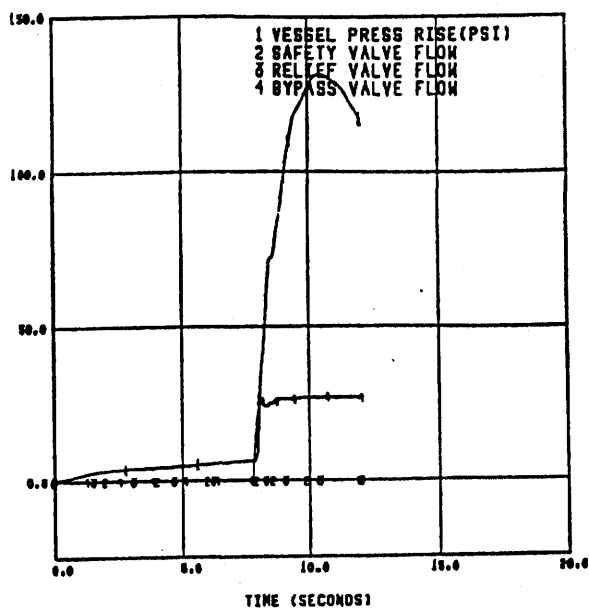
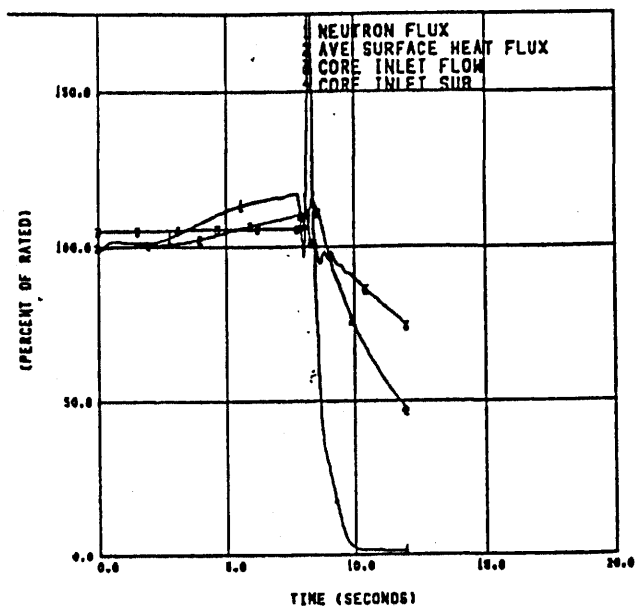
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
3729 MWt Core Power 100% Flow
250°F Rated FWT

Figure 15D-6a (Sheet 4 of 4)



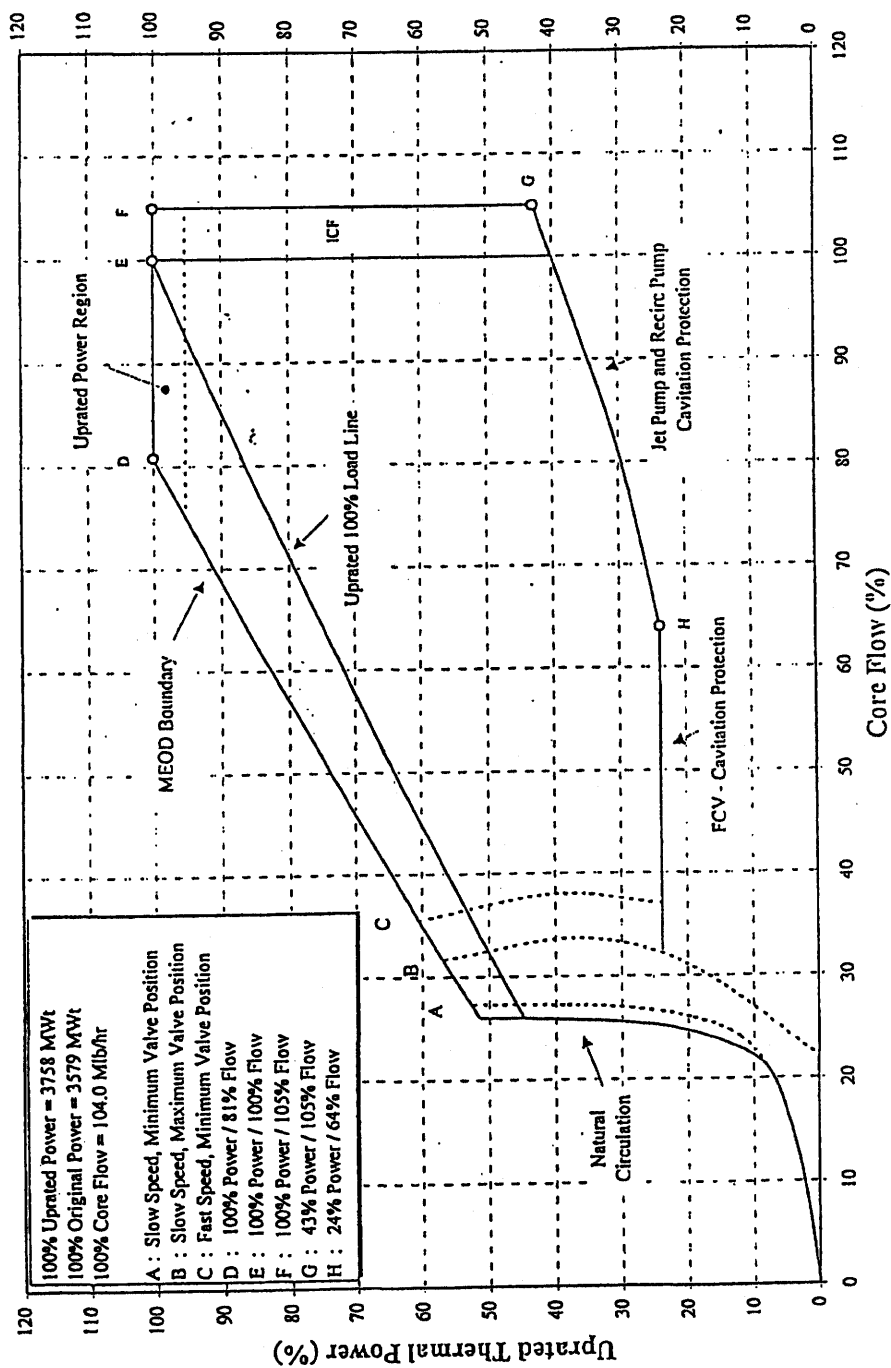
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
for Power Uprate 100% Power,
105% Flow, 255.5°F FWT

Figure 15D-6b



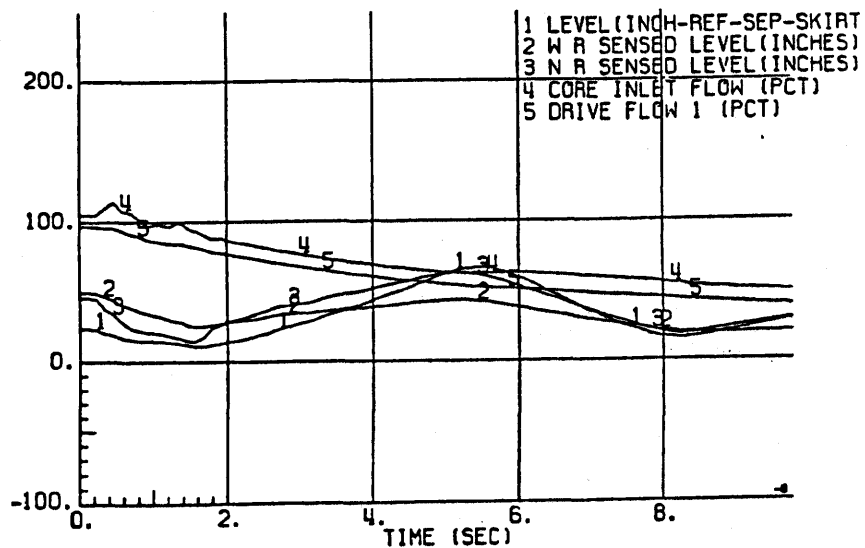
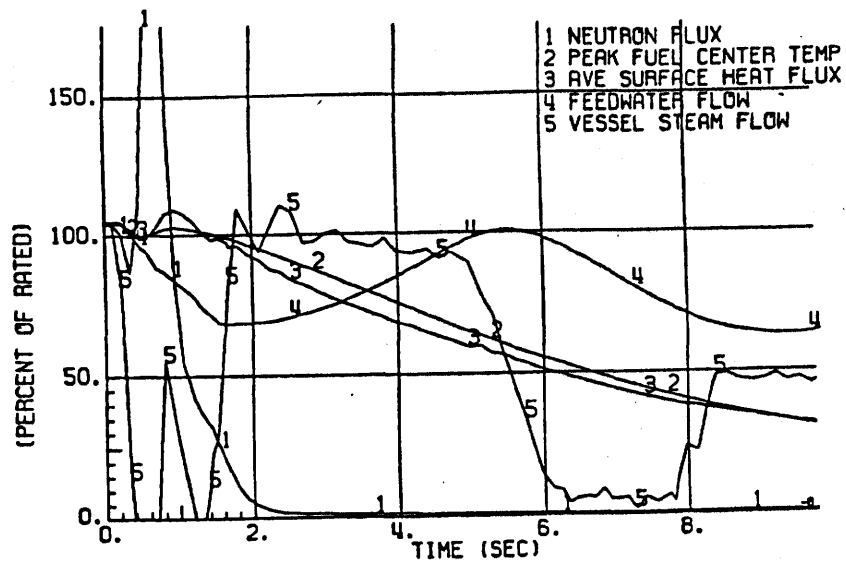
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PERRY NUCLEAR POWER PLANT

Maximum Extended Operating
Domain Power/Flow Map

Figure 15E.2-1



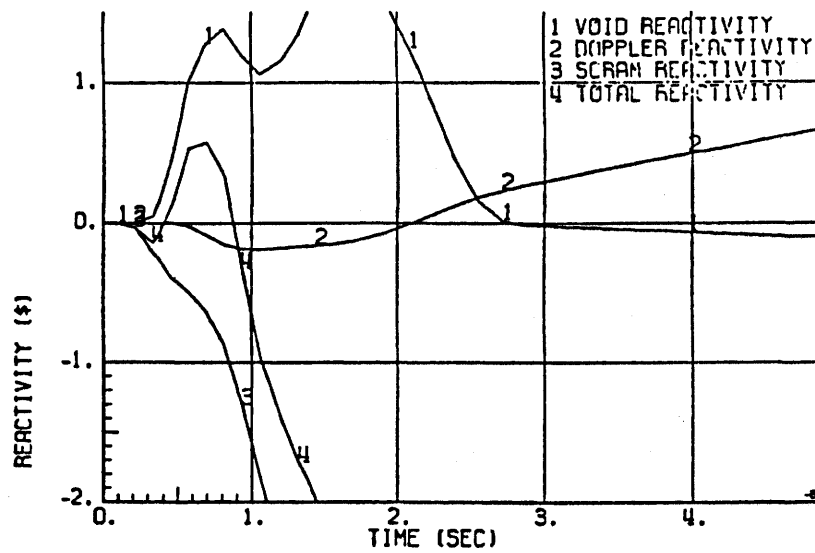
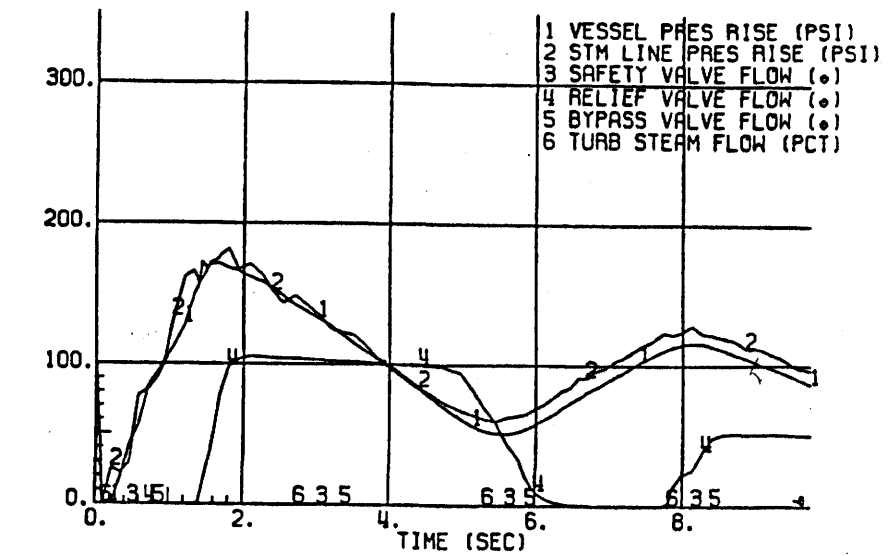
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
W/O Bypass 3729 MWt Core Power
105% Core Flow

Figure 15E.3-1 (Sheet 1 of 2)



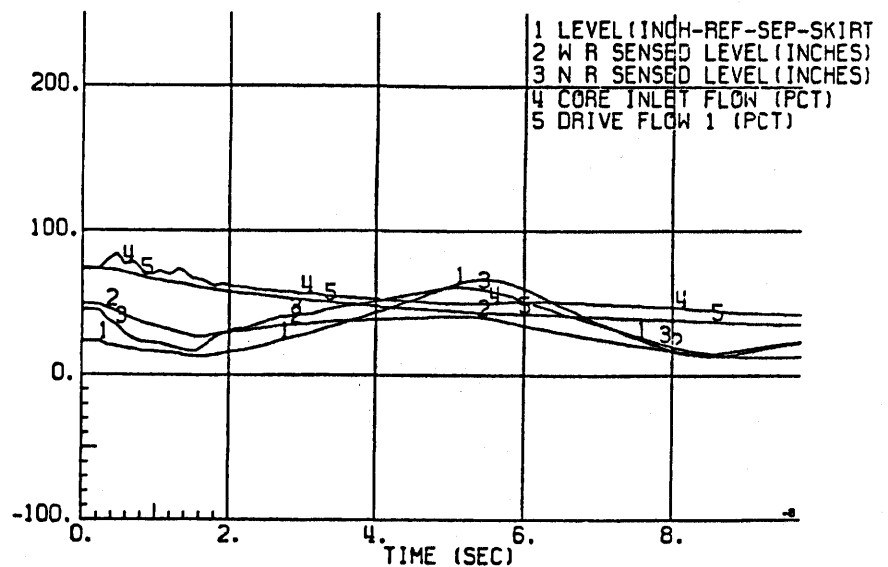
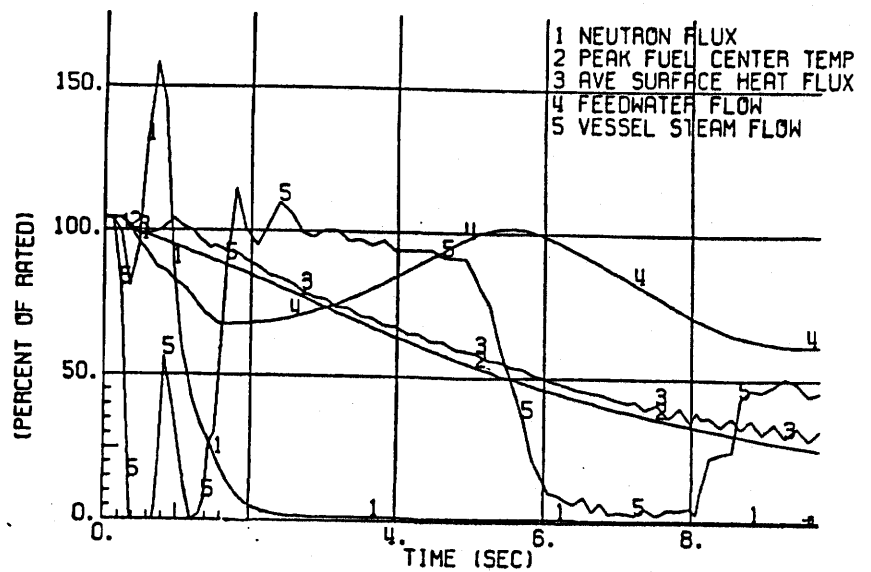
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
 W/O Bypass 3729 MWt Core Power
 105% Core Flow

Figure 15E.3-1 (Sheet 2 of 2)



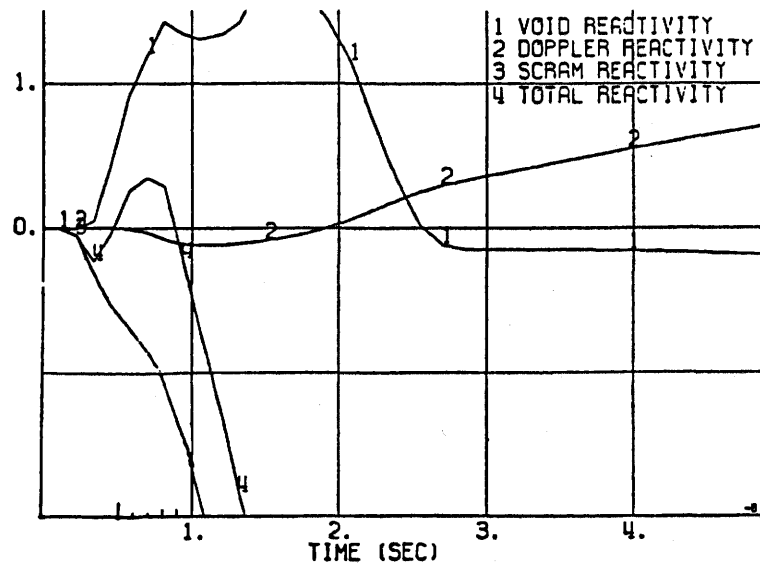
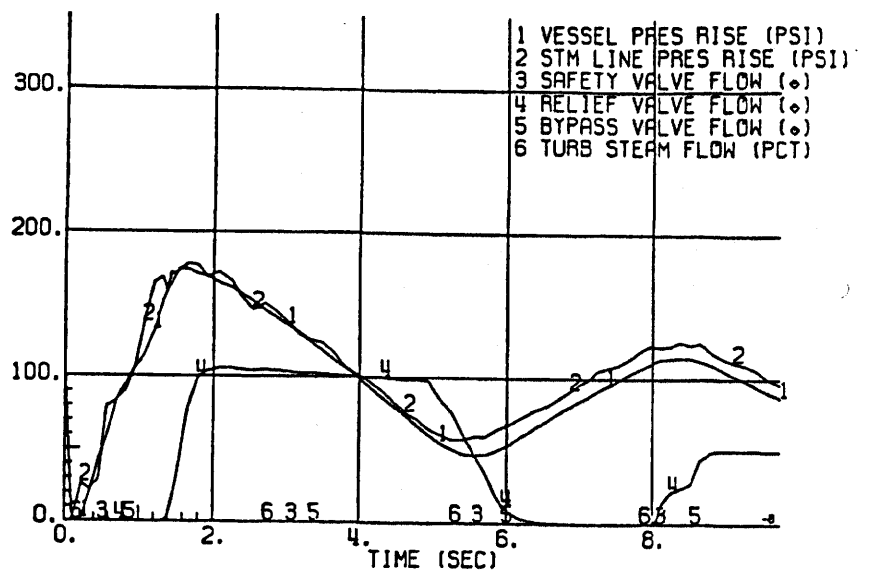
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
W/O Bypass 3729 MWt Core Power
73.6% Flow

Figure 15E.3-2 (Sheet 1 of 2)



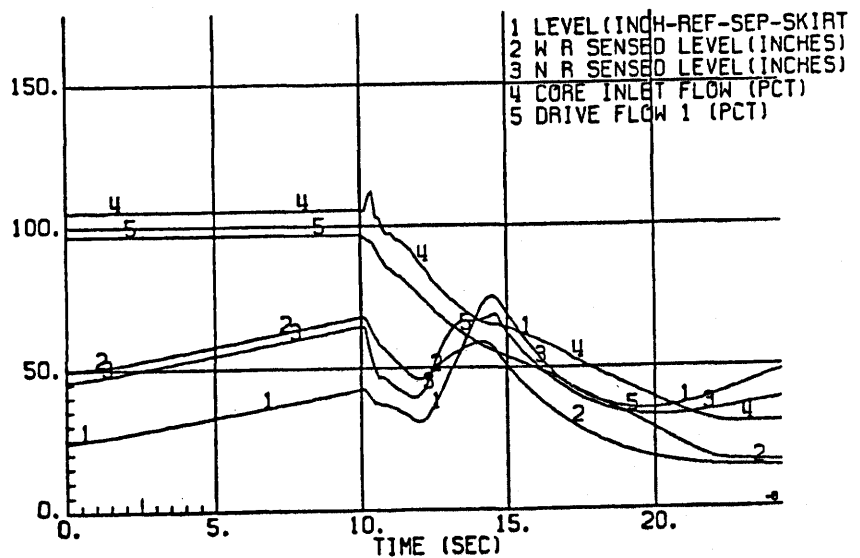
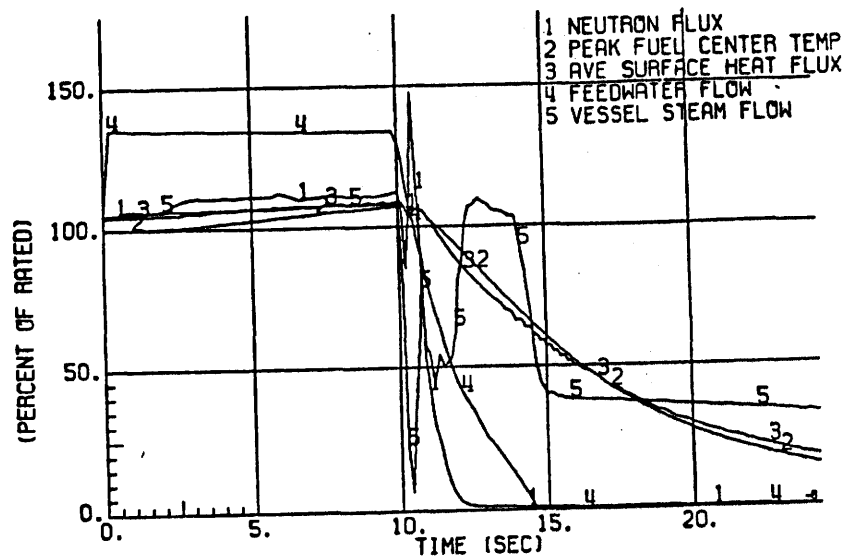
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection
 W/O Bypass 3729 MWt Core Power
 73.6% Flow

Figure 15E.3-2 (Sheet 2 of 2)



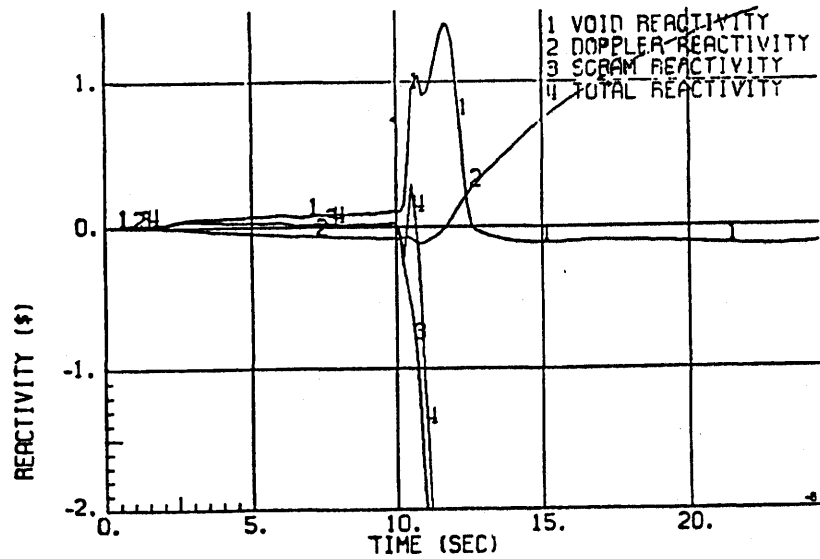
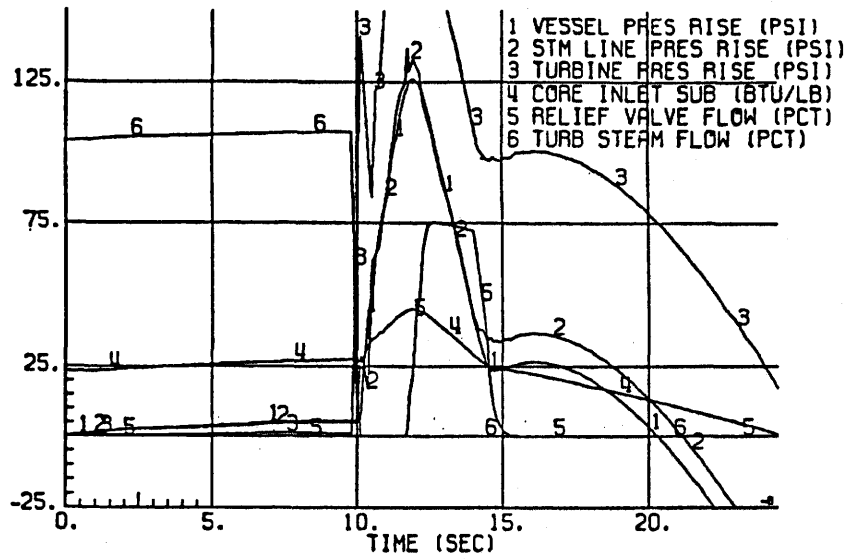
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
3729 MWt Core Power 105% Flow

Figure 15E.3-3 (Sheet 1 of 2)



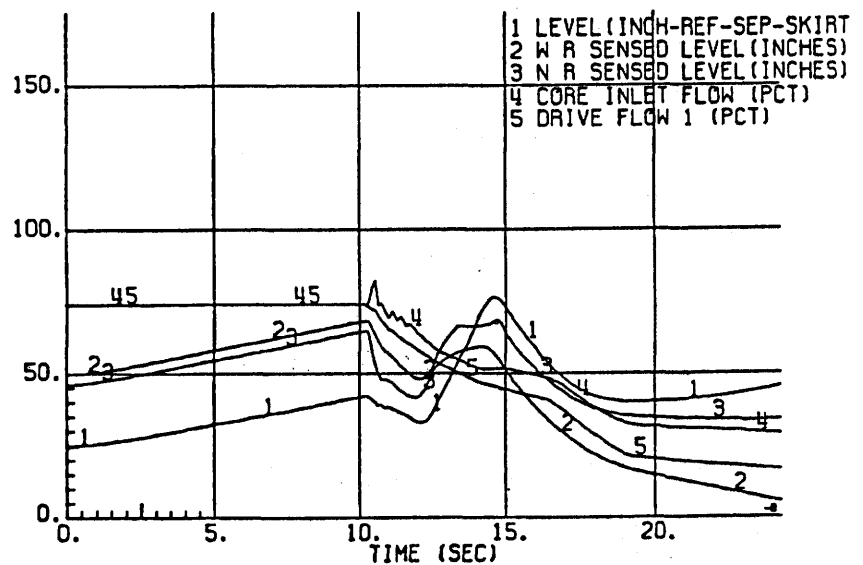
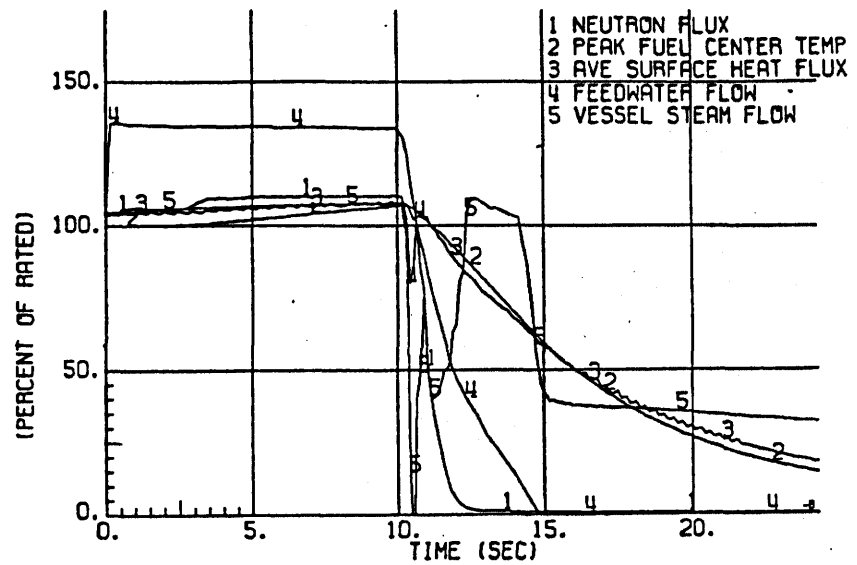
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 3729 MWt Core Power 105% Flow

Figure 15E.3-3 (Sheet 2 of 2)



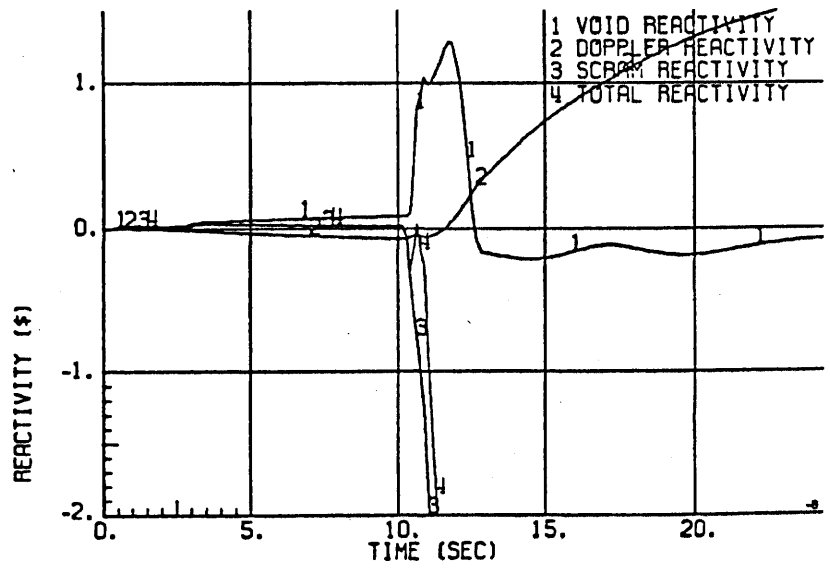
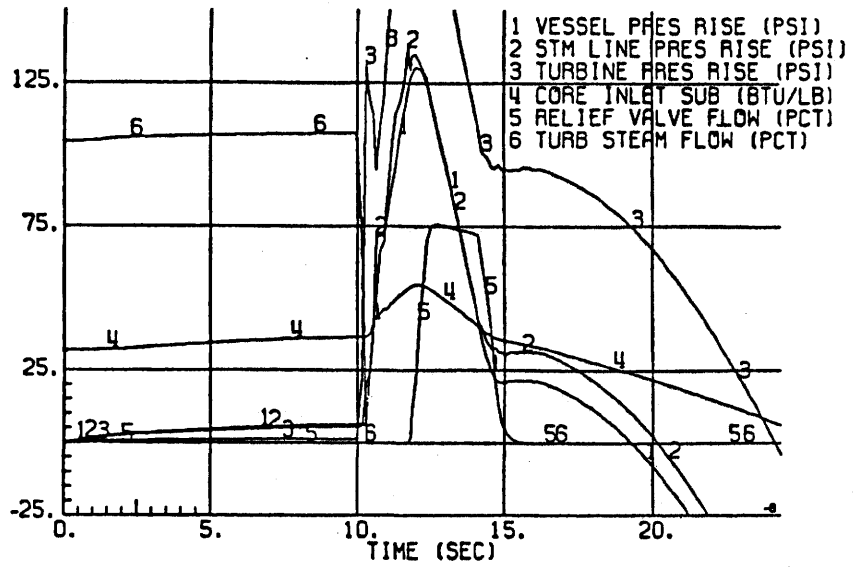
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
3729 MWt Core Power 73.6% Flow

Figure 15E.3-4 (Sheet 1 of 2)



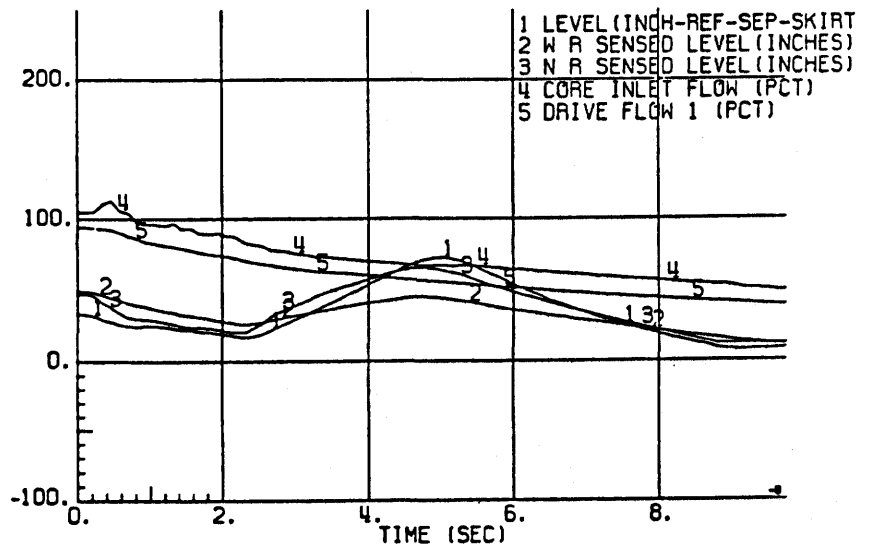
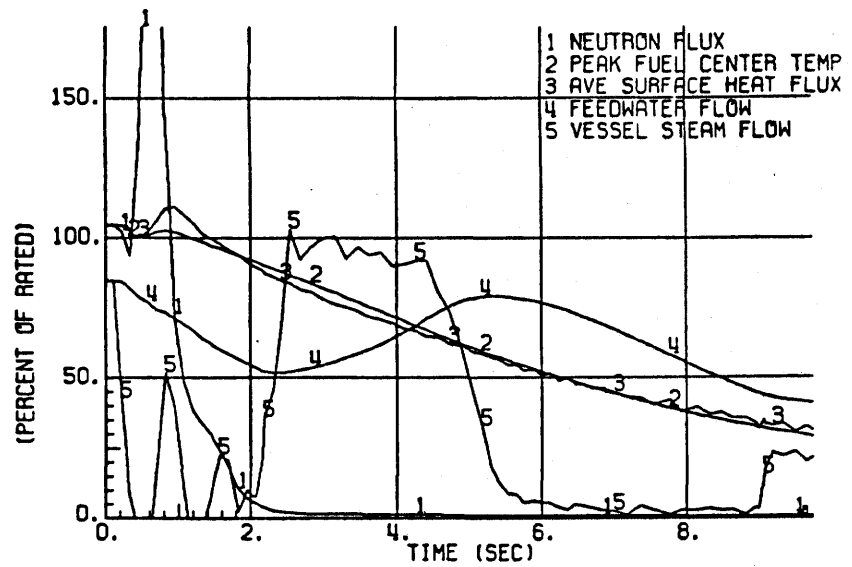
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 3729 MWt Core Power 73.6% Flow

Figure 15E.3-4 (Sheet 2 of 2)



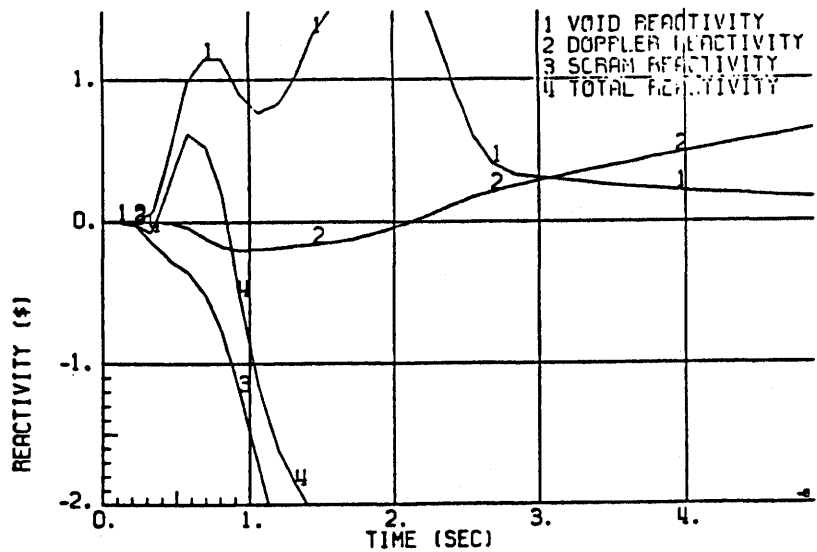
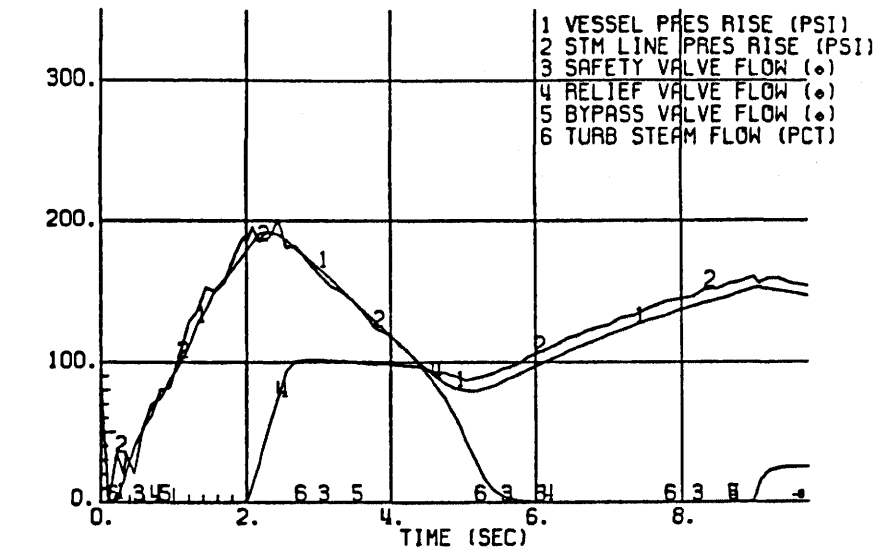
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection
W/O Bypass 104.2% Power
105% Flow 250°F FWT

Figure 15E.11-1 (Sheet 1 of 2)



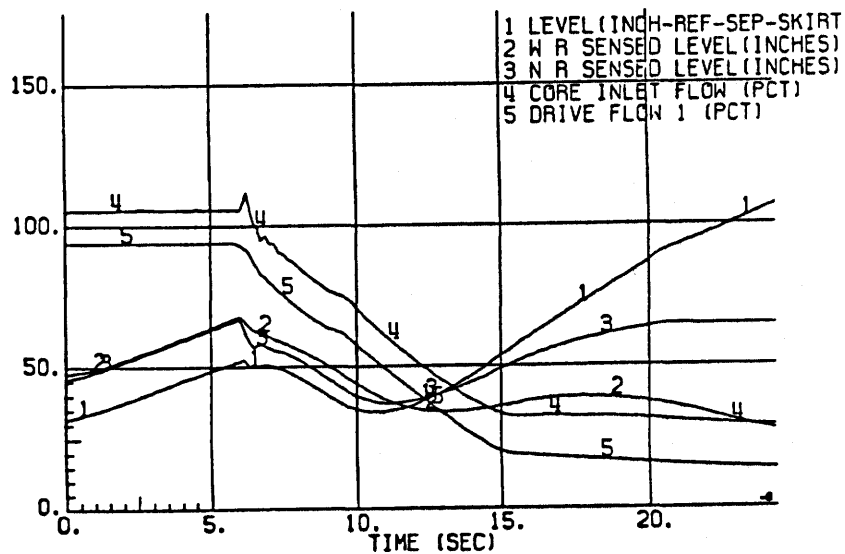
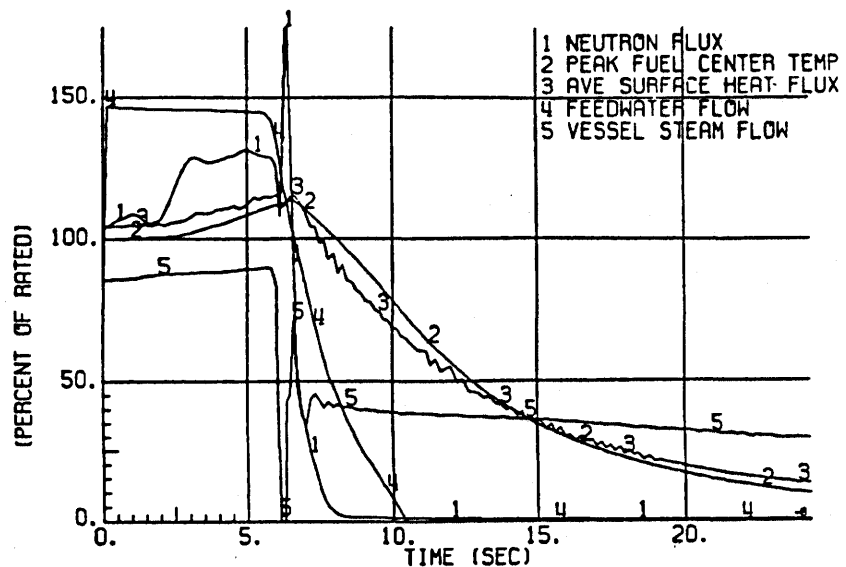
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection
 W/O Bypass 104.2% Power
 105% Flow 250°F FWT

Figure 15E.11-1 (Sheet 2 of 2)



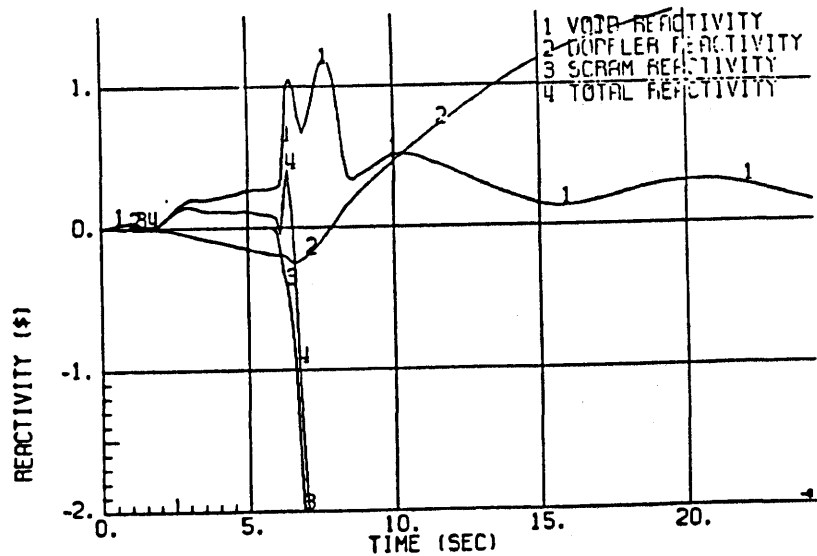
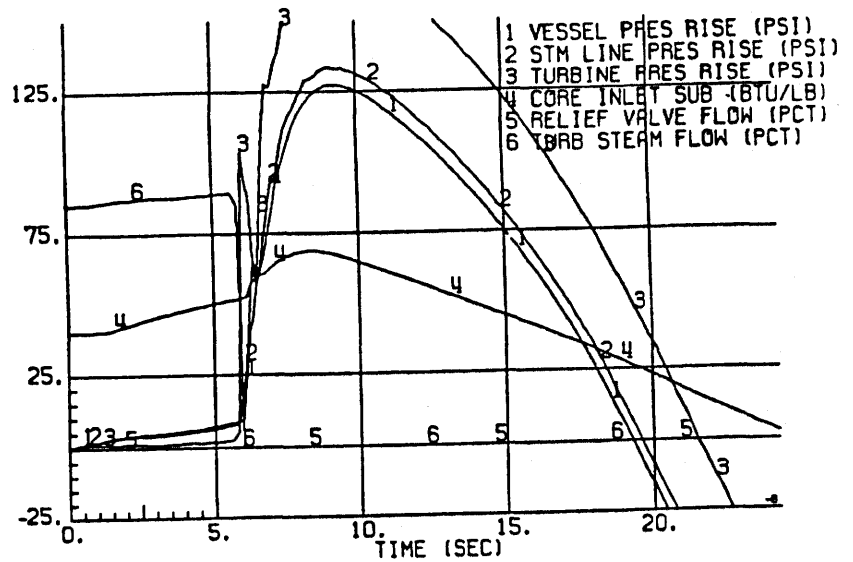
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
104.2% Power 105% Flow
250°F FWT

Figure 15E.11-2 (Sheet 1 of 2)



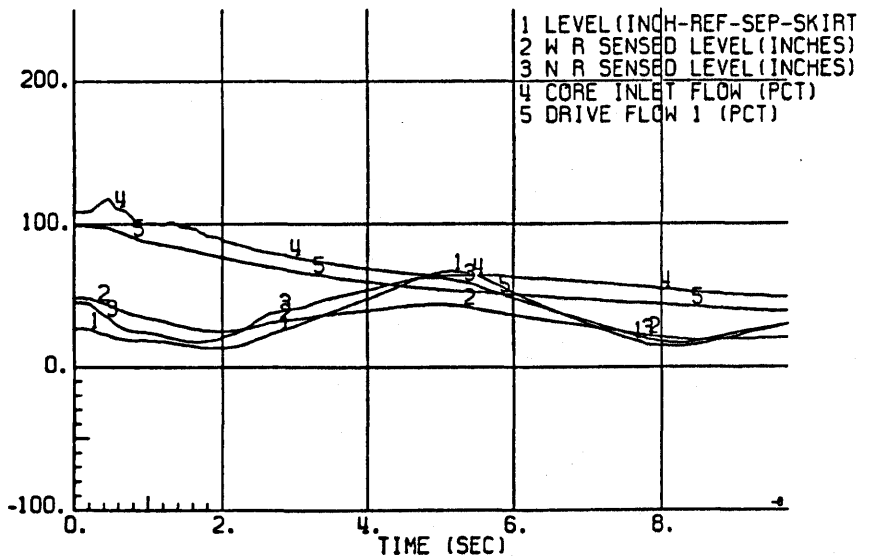
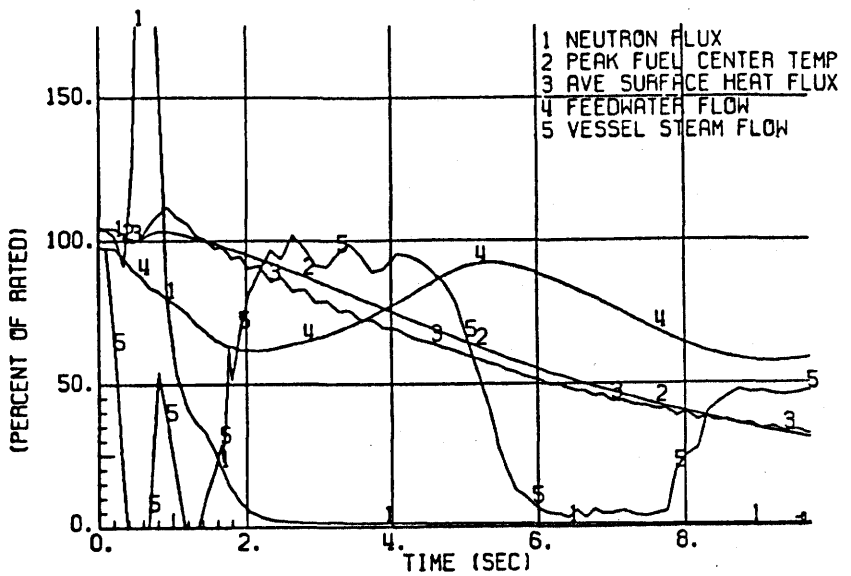
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 104.2% Power 105% Flow
 250°F FWT

Figure 15E.11-2 (Sheet 2 of 2)



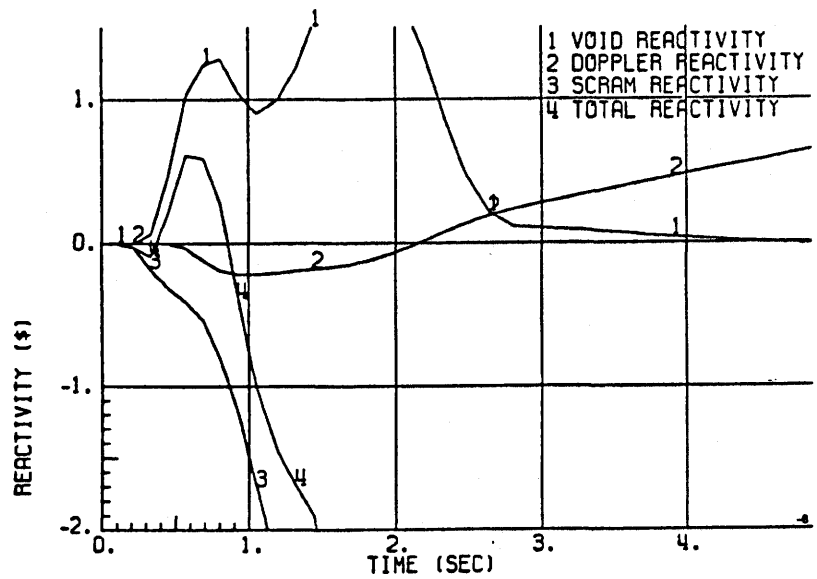
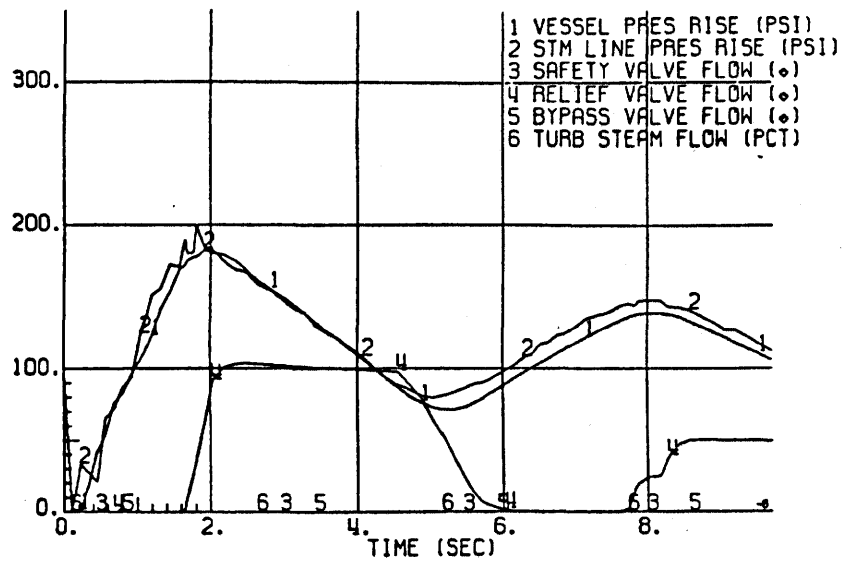
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection
W/O Bypass 104.2% Power 108.7% Flow
370°F FWT

Figure 15E.11-3 (Sheet 1 of 2)



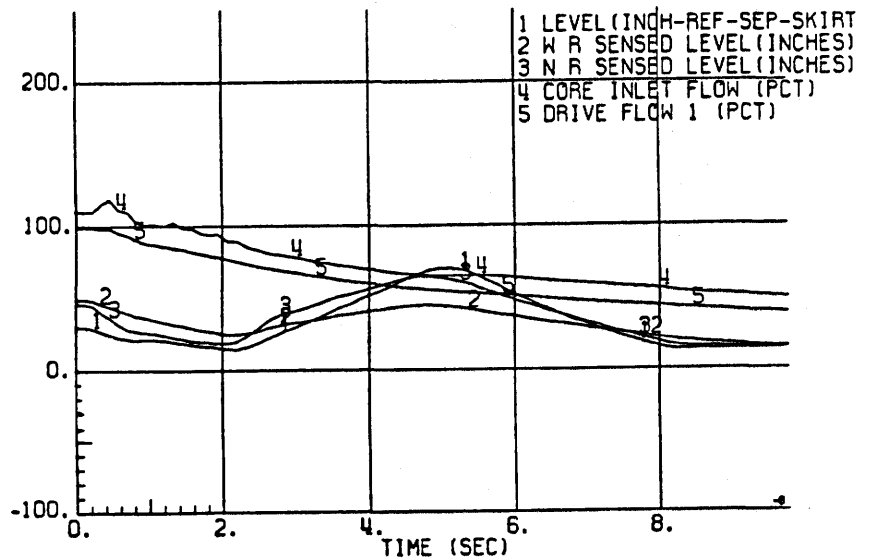
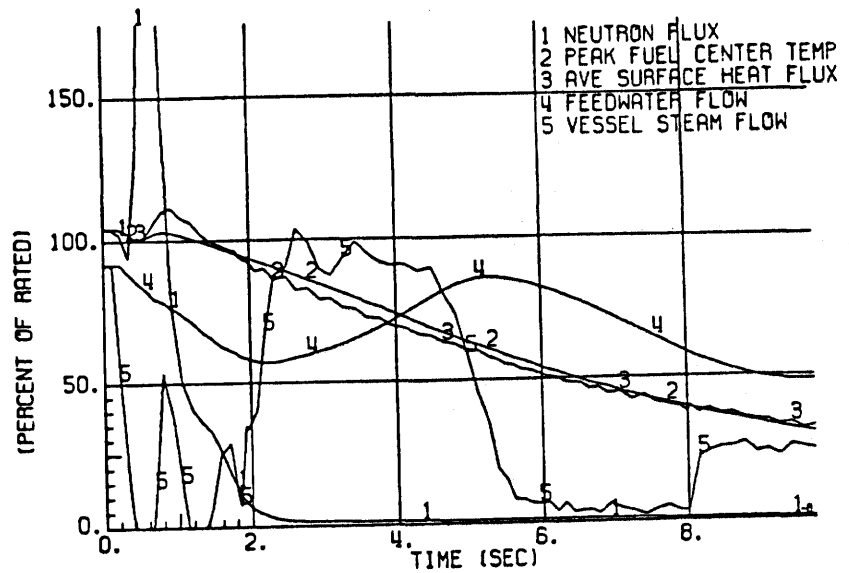
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
 W/O Bypass 104.2% Power
 108.7% Flow 370°F FWT

Figure 15E.11-3 (Sheet 2 of 2)



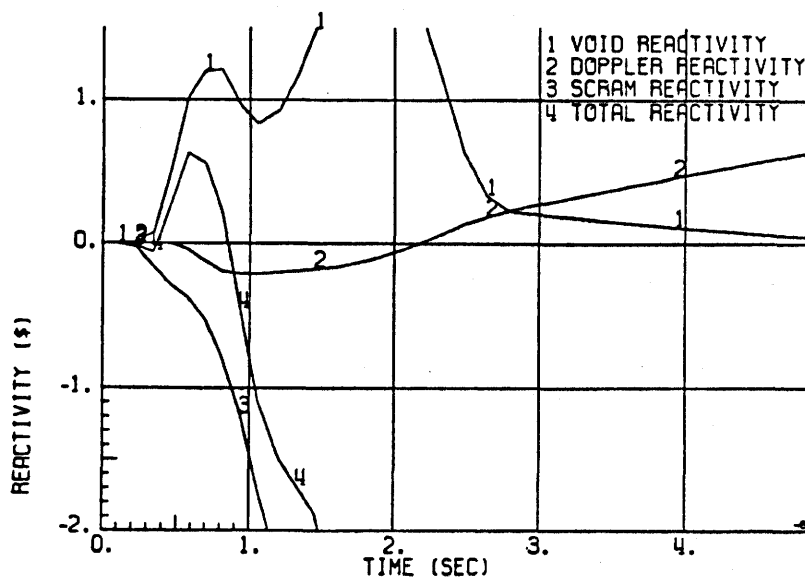
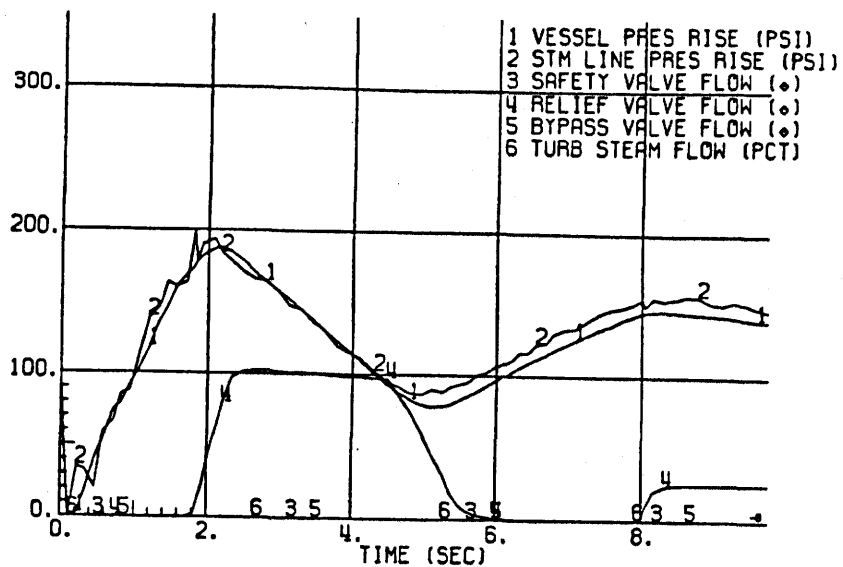
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection
W/O Bypass 104.2% Power
110% Flow 320°F FWT

Figure 15E.11-4 (Sheet 1 of 2)



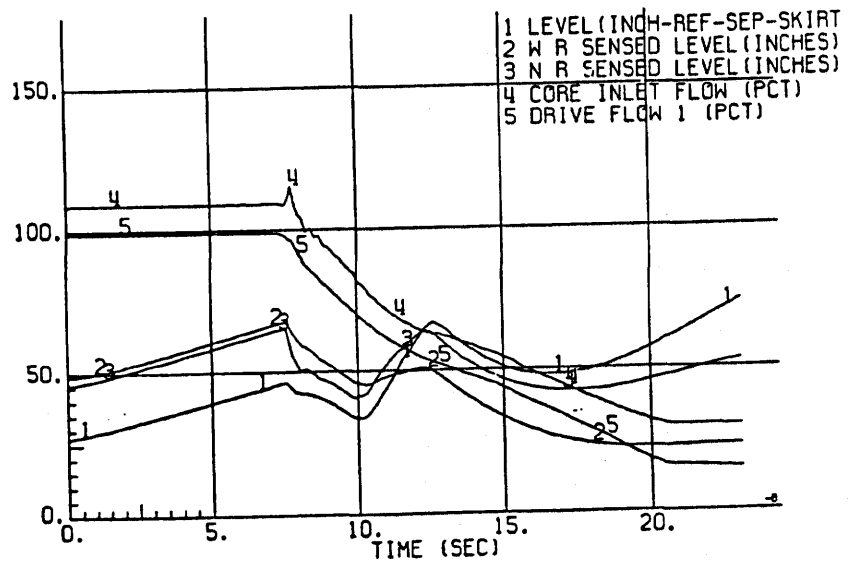
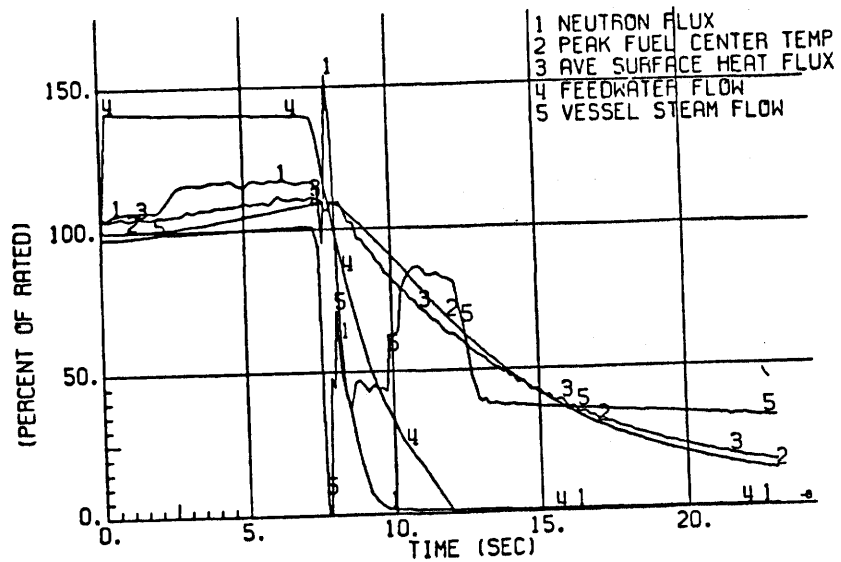
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
 W/O Bypass 104.2% Power
 110% Flow 320°F FWT

Figure 15E.11-4 (Sheet 2 of 2)



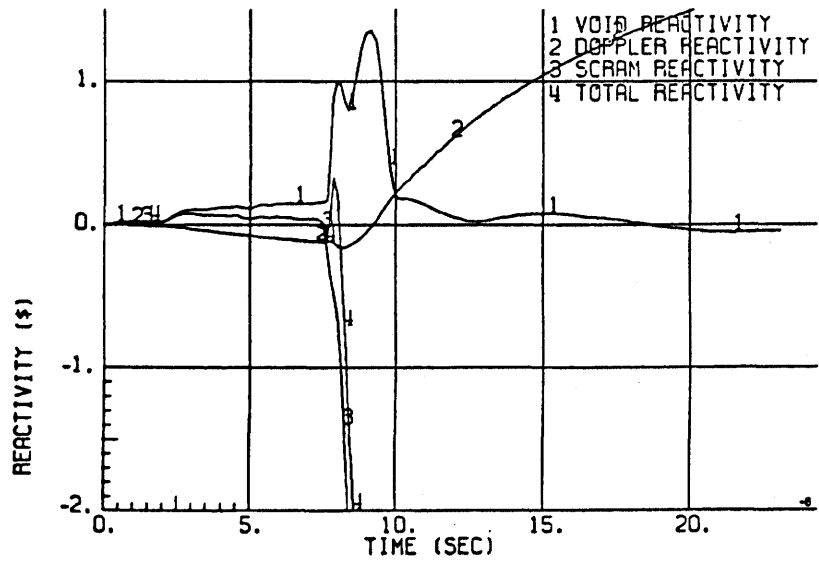
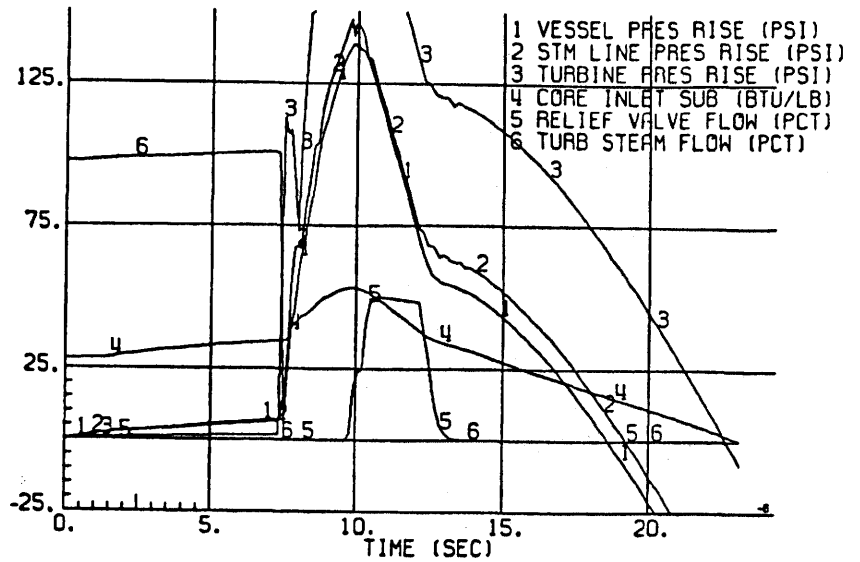
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
104.2% Power 108% Flow
370°F FWT

Figure 15E.11-5 (Sheet 1 of 2)



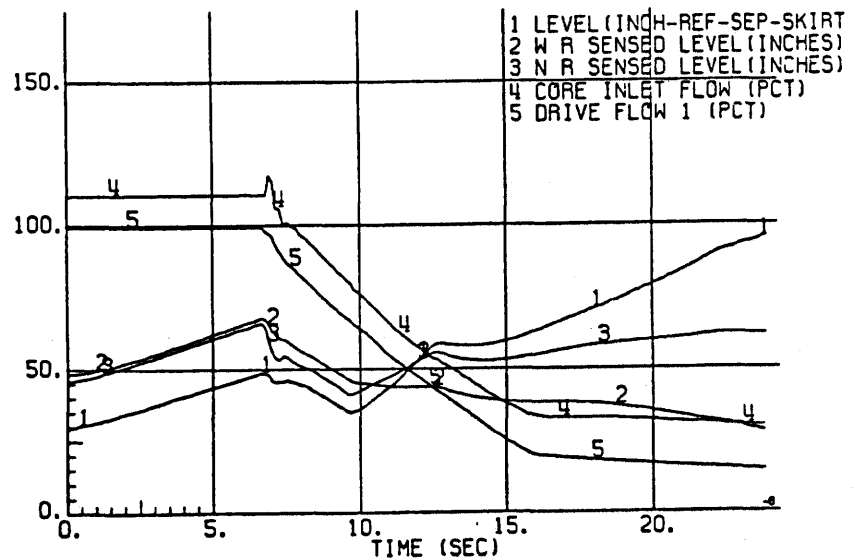
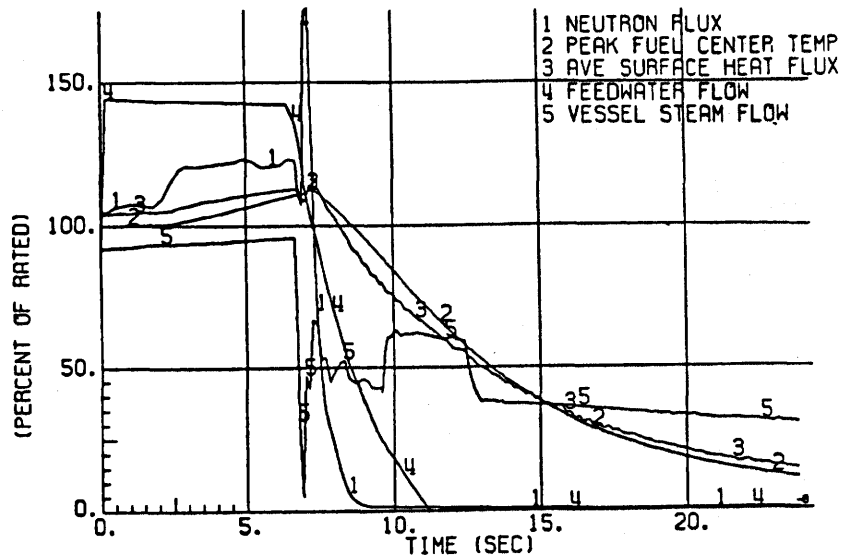
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 104.2% Power 108% Flow
 370°F FWT

Figure 15E.11-5 (Sheet 2 of 2)



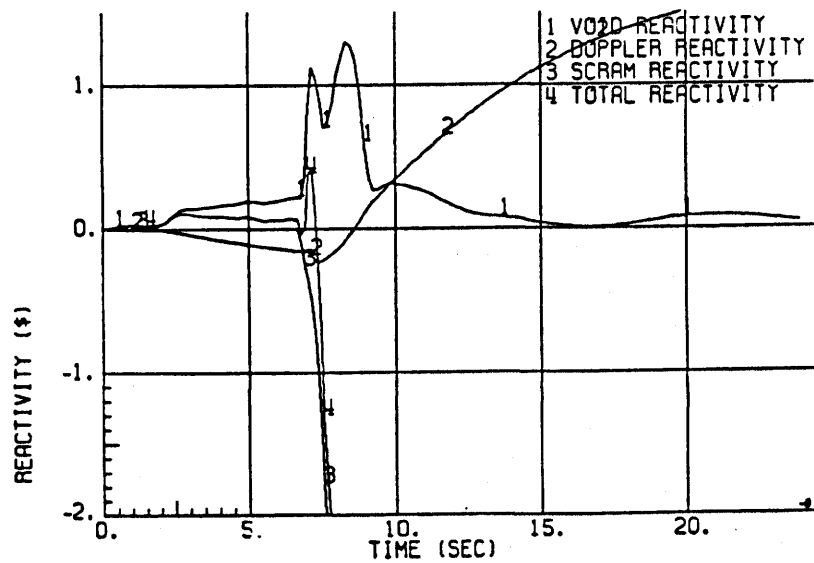
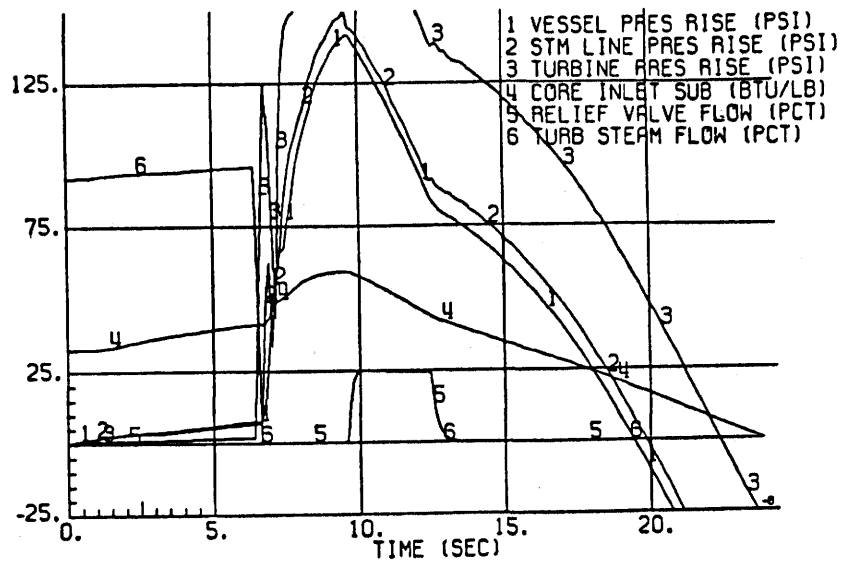
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
104.2% Power 110% Flow
320°F FWT

Figure 15E.11-6 (Sheet 1 of 2)



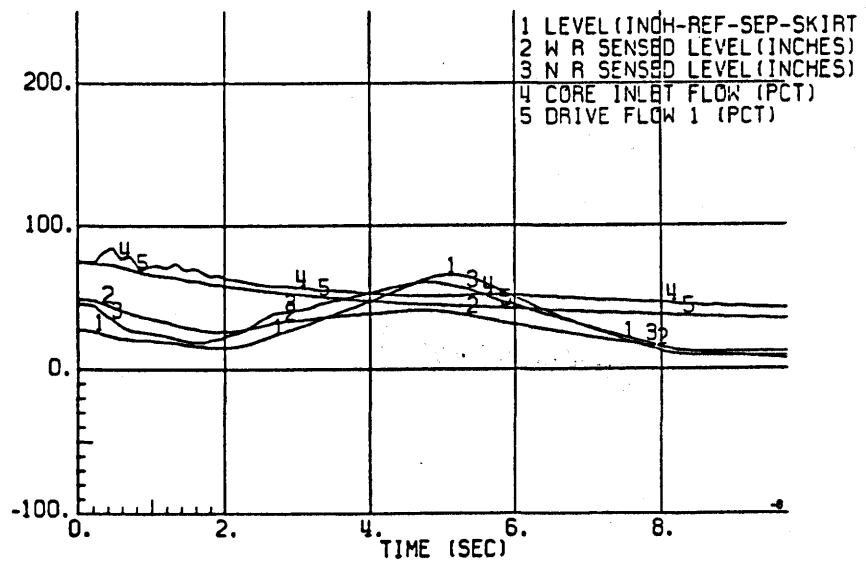
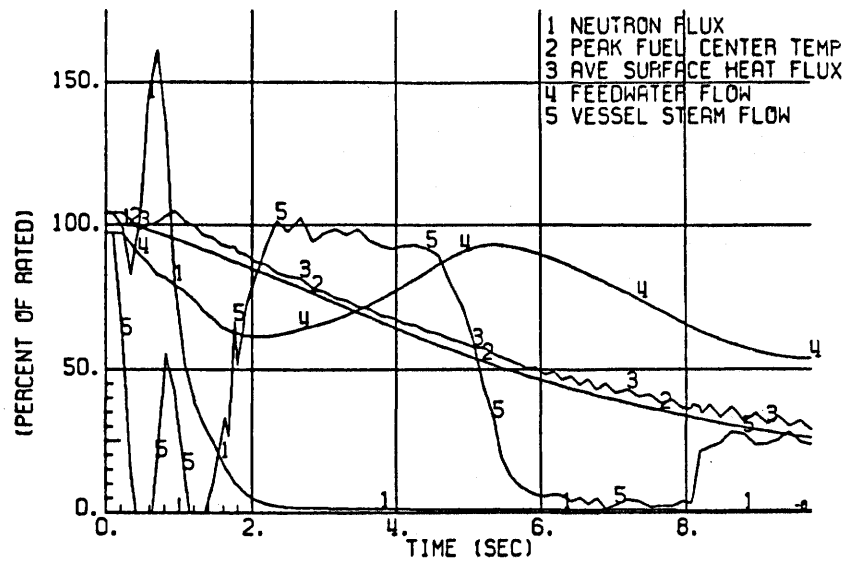
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 104.2% Power 110% Flow
 320°F FWT

Figure 15E.11-6 (Sheet 2 of 2)



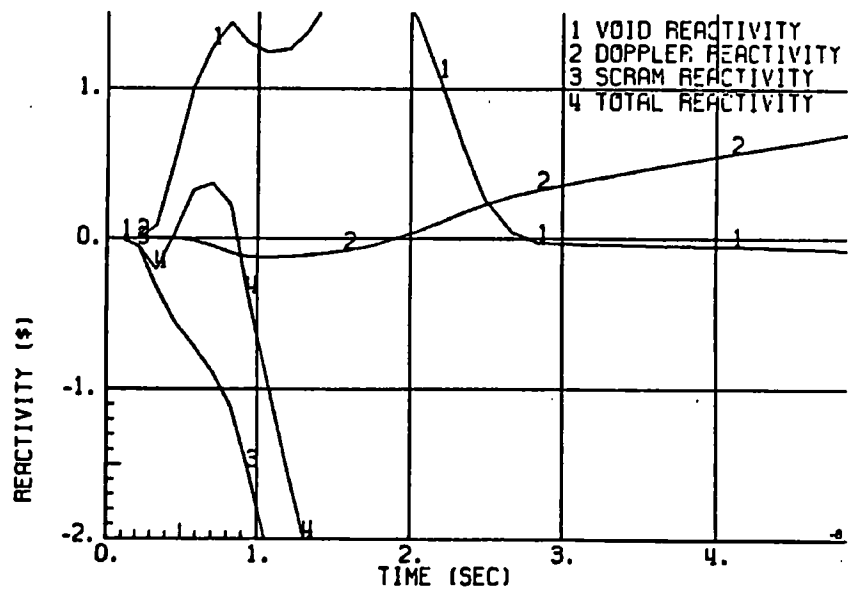
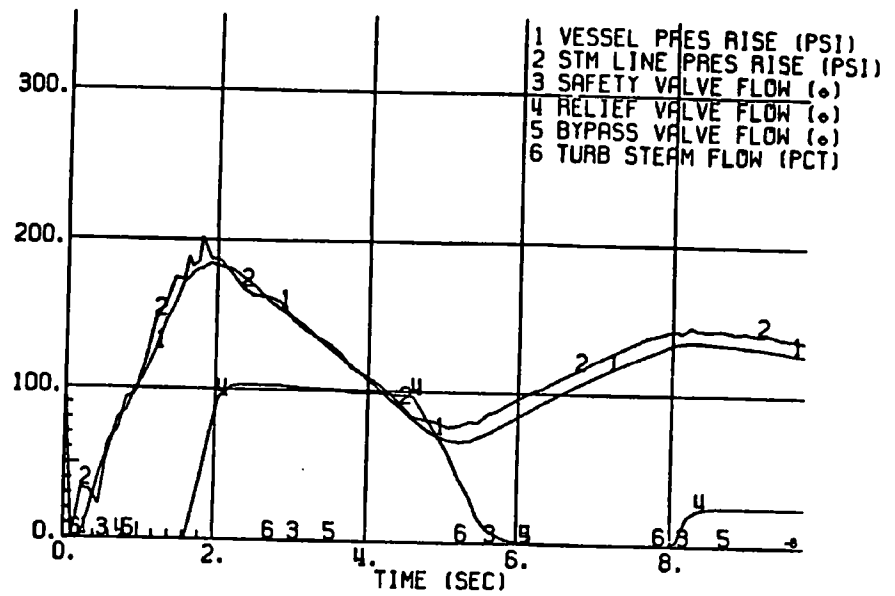
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
W/O Bypass 104.2% Power
74.8% Flow 370°F FWT

Figure 15E.11-7 (Sheet 1 of 2)



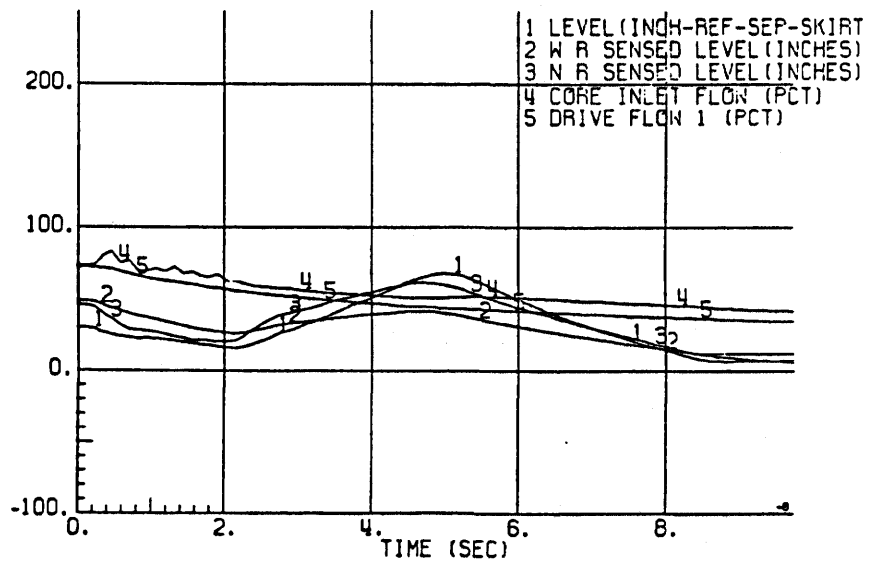
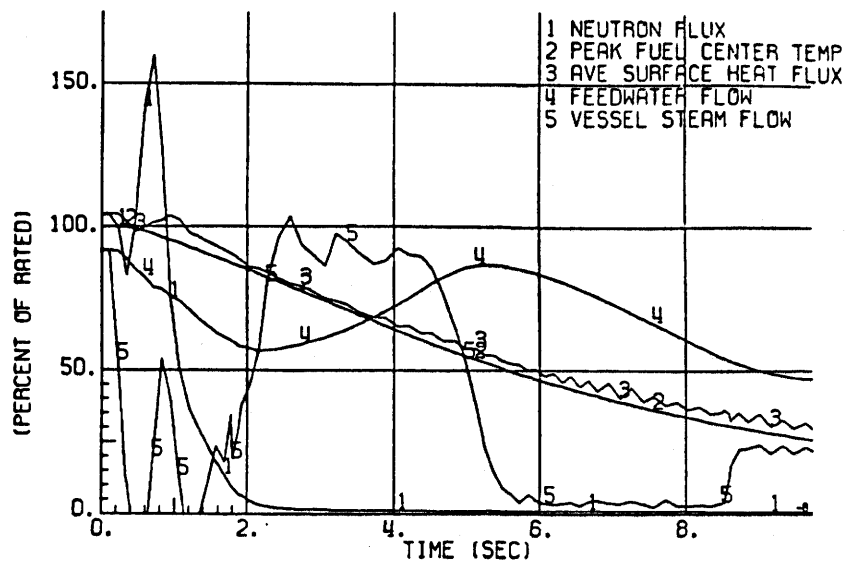
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
 W/O Bypass 104.2% Power
 74.8% Flow 370°F FWT

Figure 15E.11-7 (Sheet 2 of 2)



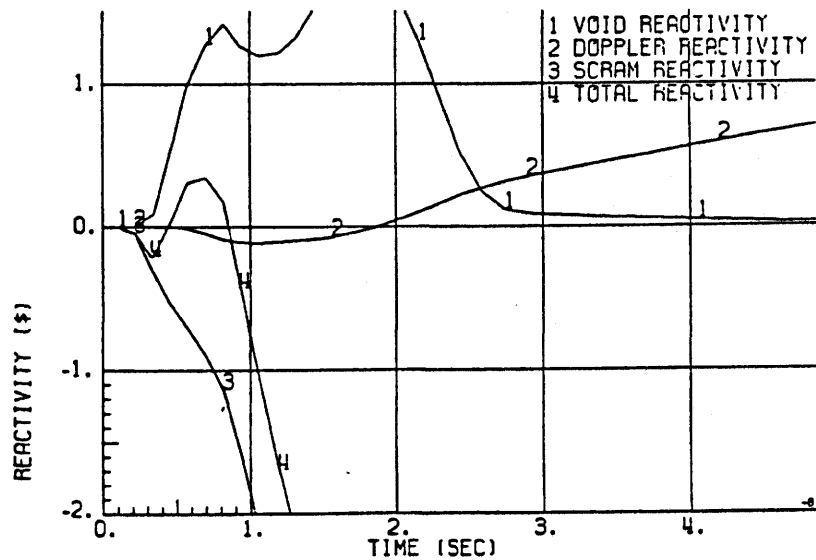
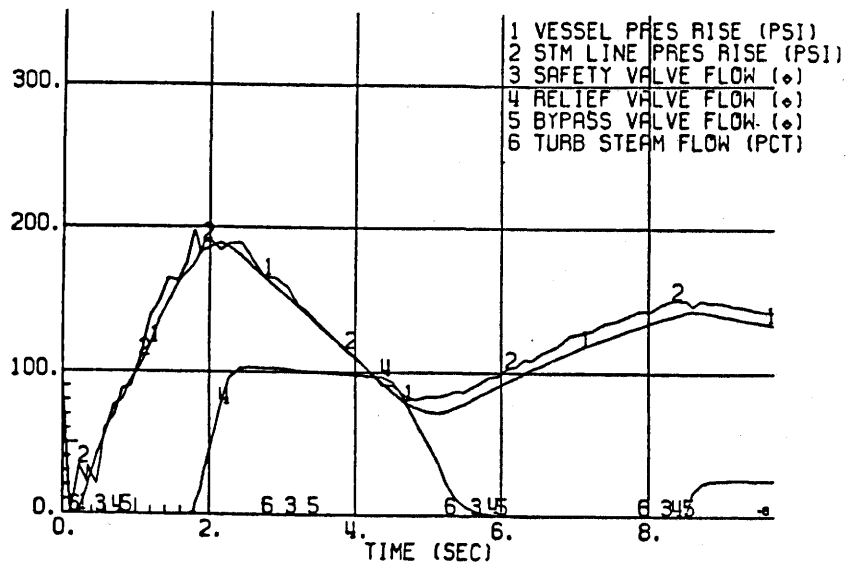
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection
W/O Bypass 104.2% Power
73.7% Flow 320°F FWT

Figure 15E.11-8 (Sheet 1 of 2)



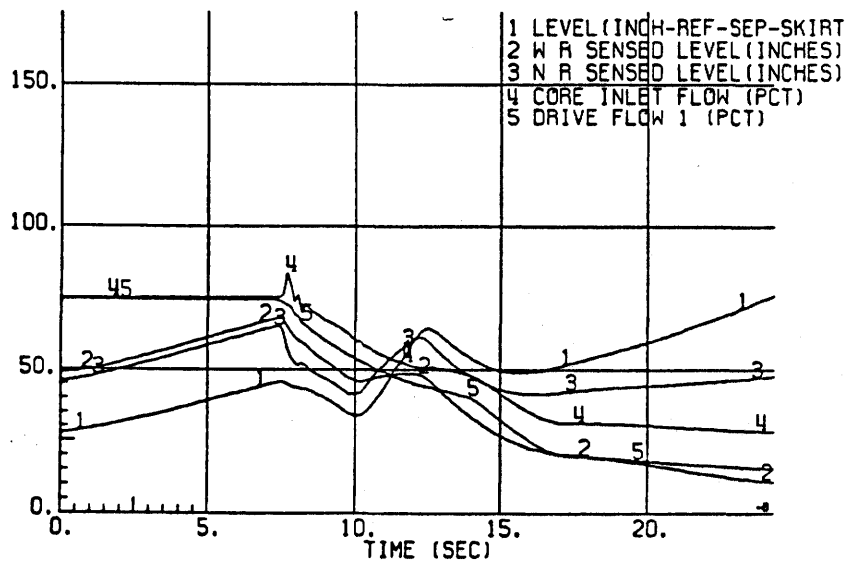
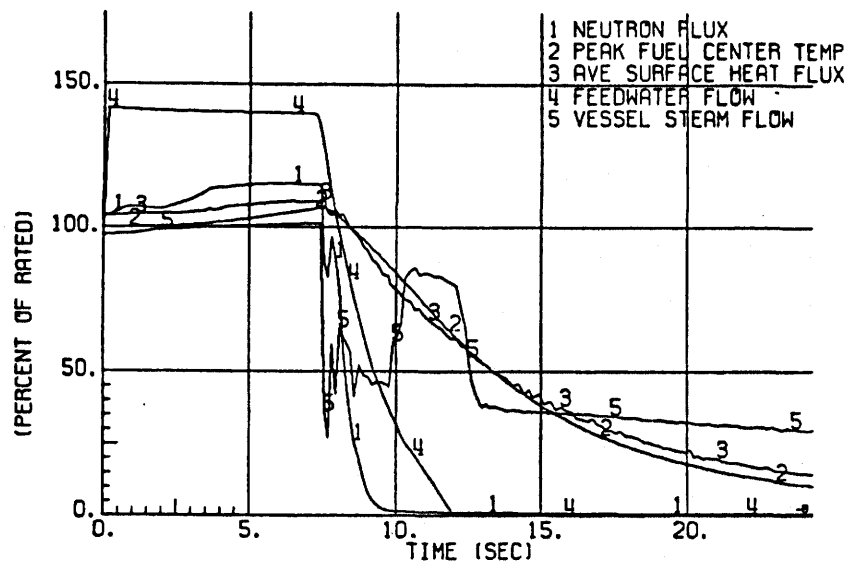
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection
 W/O Bypass 104.2% Power
 73.7% Flow 320°F FWT

Figure 15E.11-8 (Sheet 2 of 2)



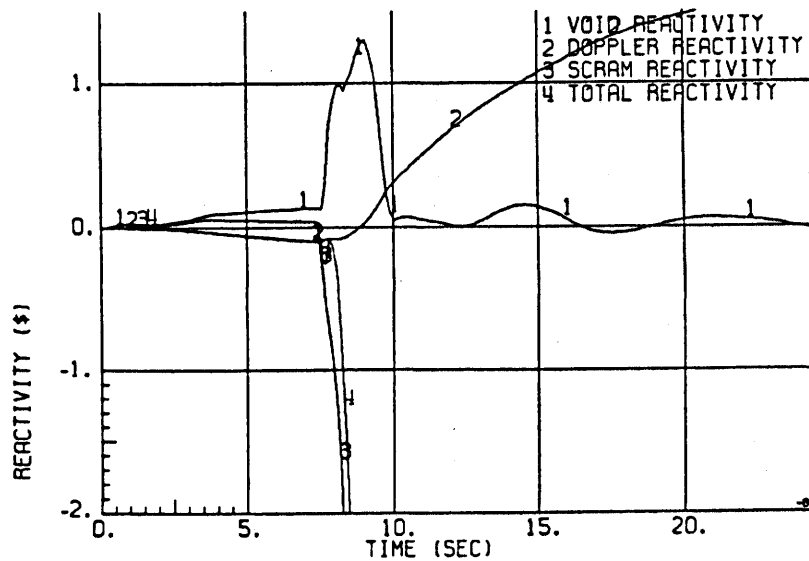
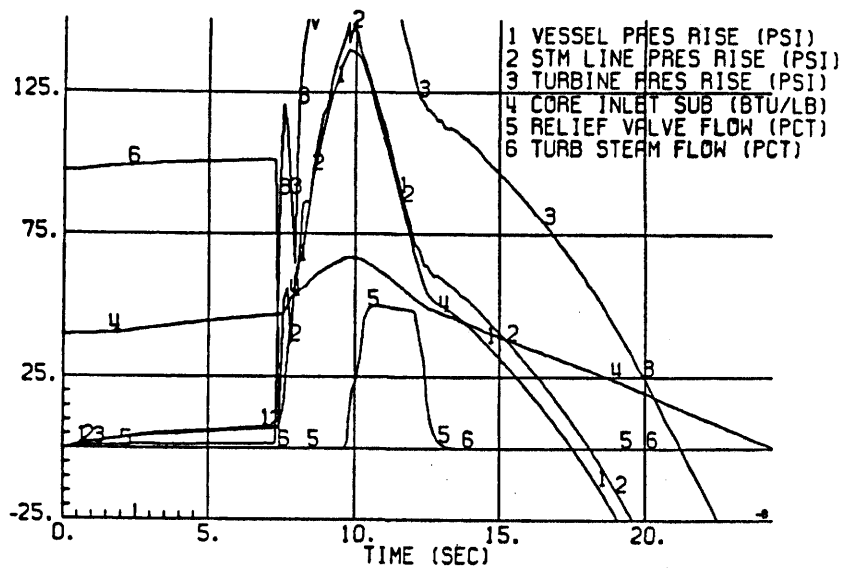
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
104.2% Power 74.8% Flow
370°F FWT

Figure 15E.11-9 (Sheet 1 of 2)



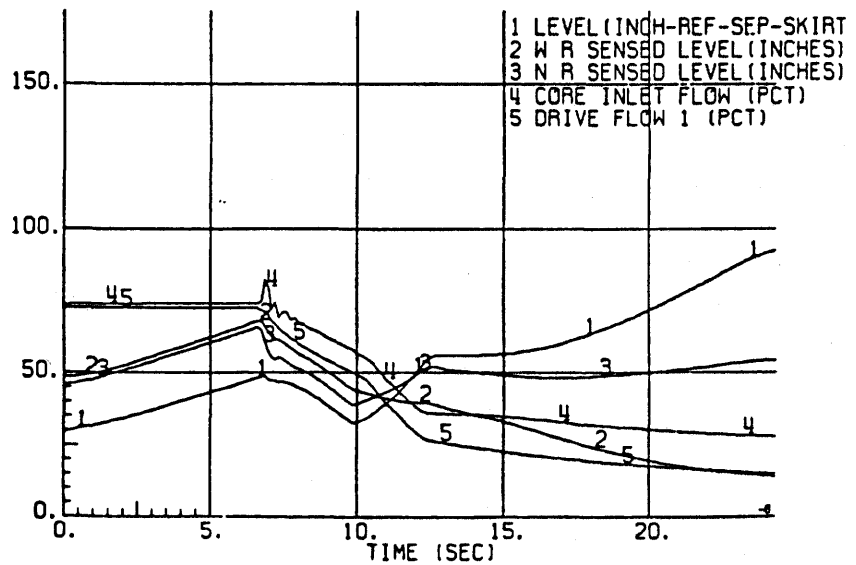
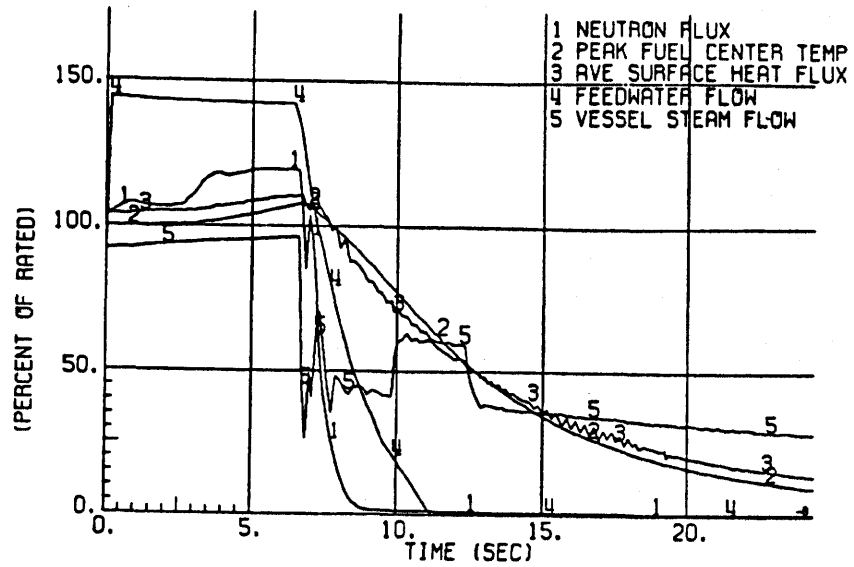
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 104.2% Power 74.8% Flow
 370°F FWT

Figure 15E.11-9 (Sheet 2 of 2)



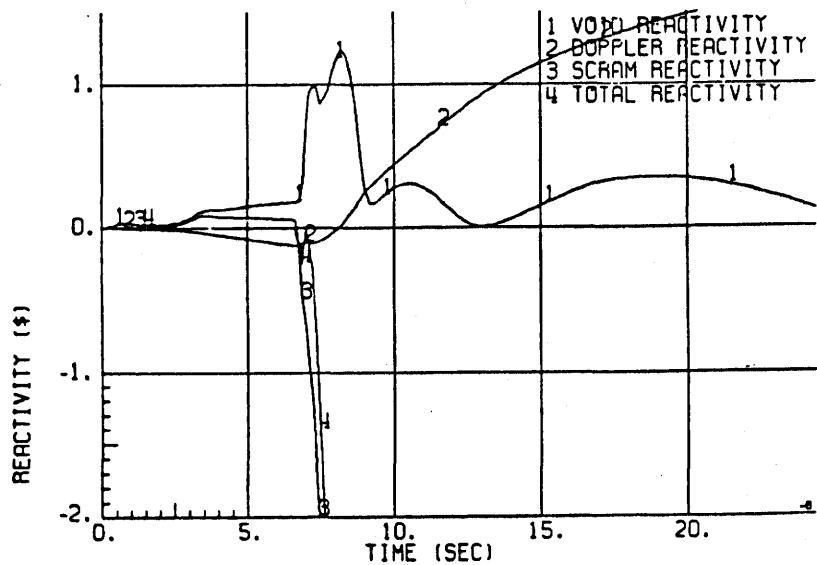
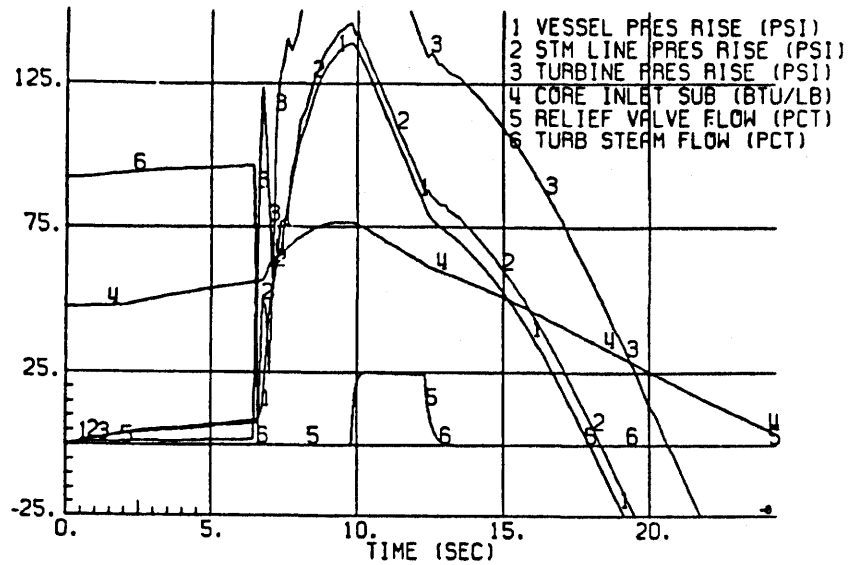
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
104.2% Power 73.7% Flow
320°F FWT

Figure 15E.11-10 (Sheet 1 of 2)



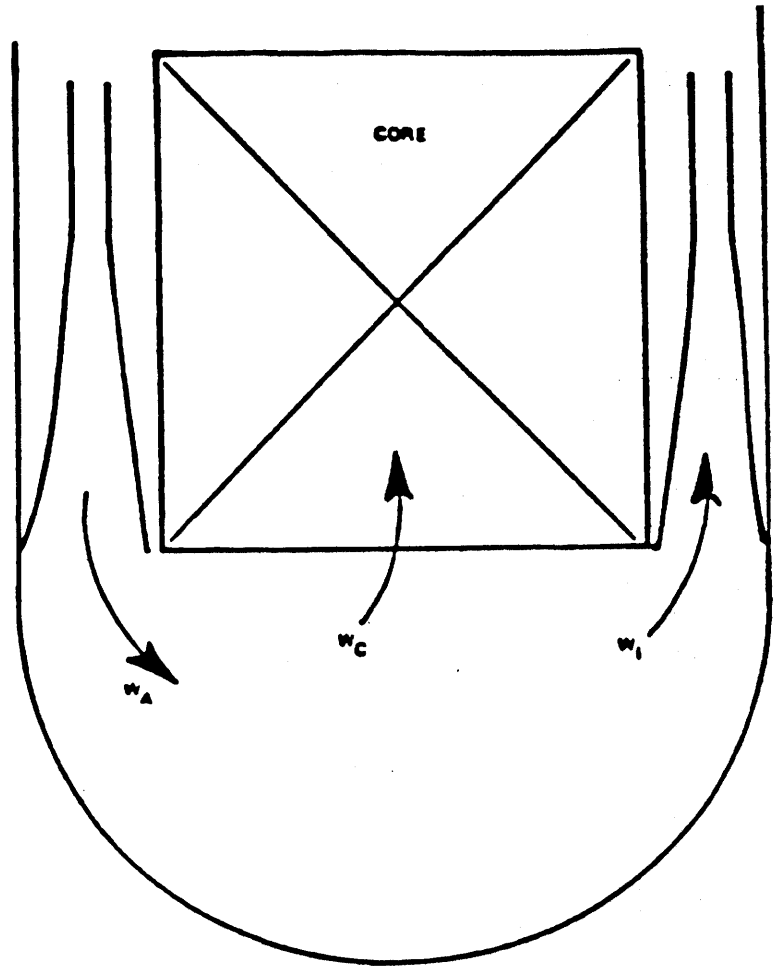
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure
 104.2% Power 73.7% Flow
 320°F FWT

Figure 15E.11-10 (Sheet 2 of 2)



w_C = Total Core Flow
 w_A = Active Loop Flow
 w_I = Inactive Loop Flow

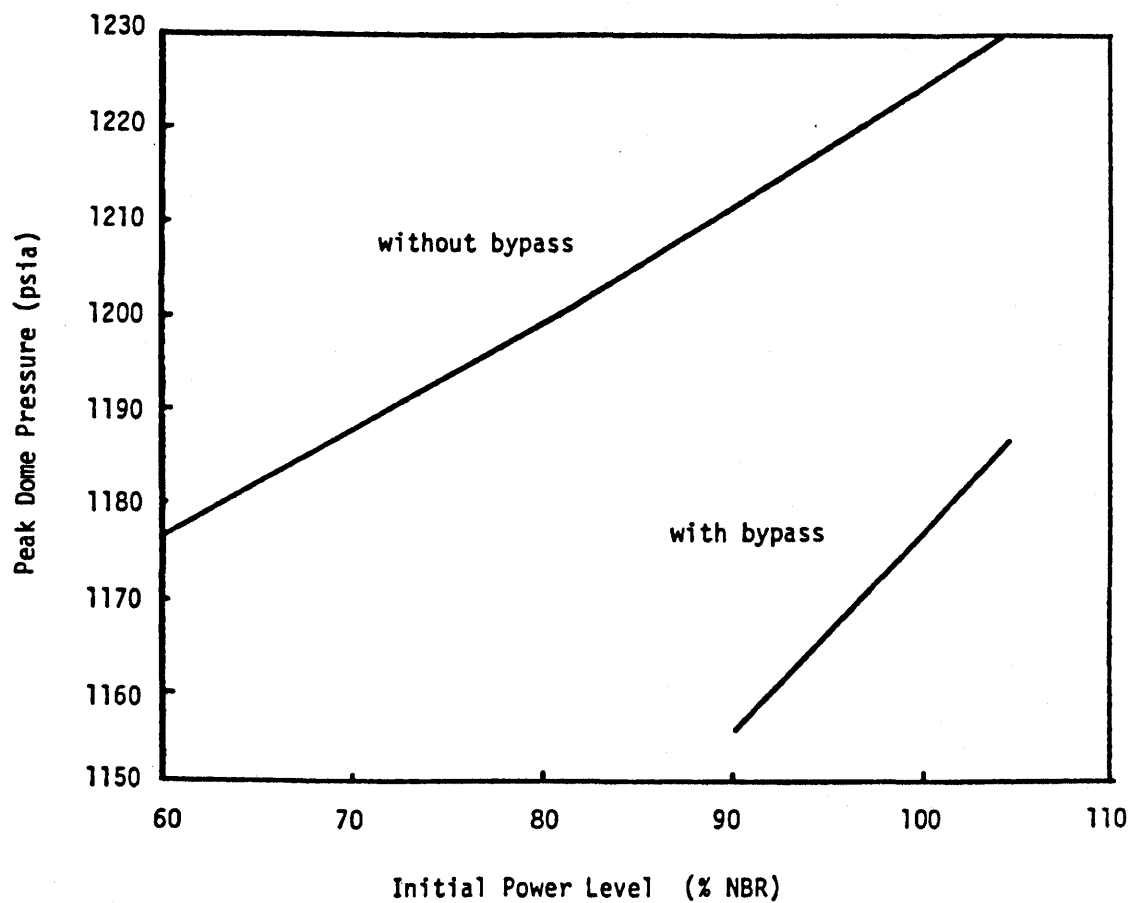
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PERRY NUCLEAR POWER PLANT

Illustration of Single
 Recirculation Loop
 Operation Flows

Figure 15F.2-1



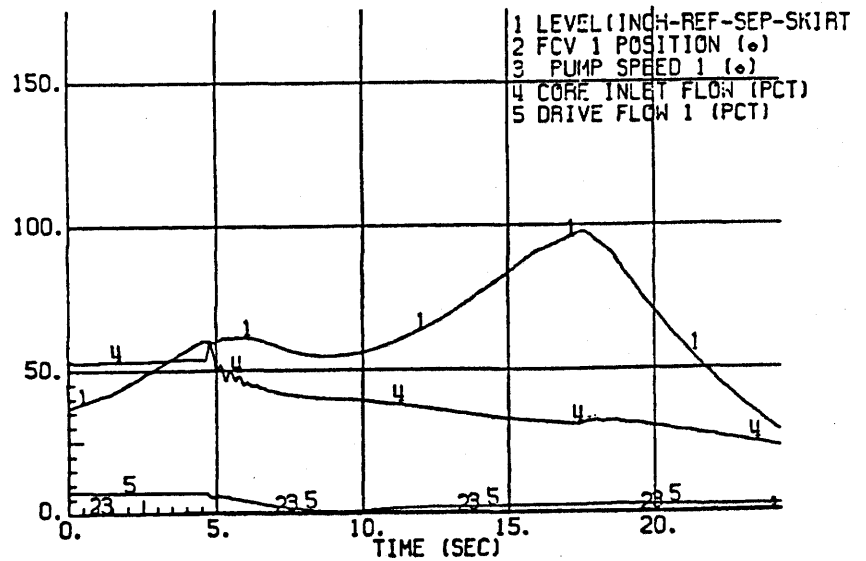
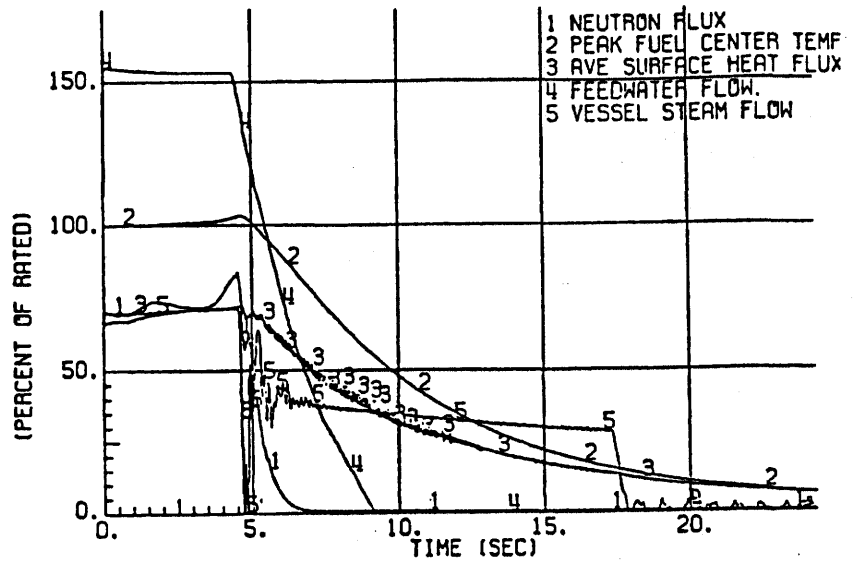
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Peak Dome Pressure Versus
Initial Power Level,
Turbine Trip at EOEC

Figure 15F.3-1



FWCF MAX DEMA W/ HWL TRIPS

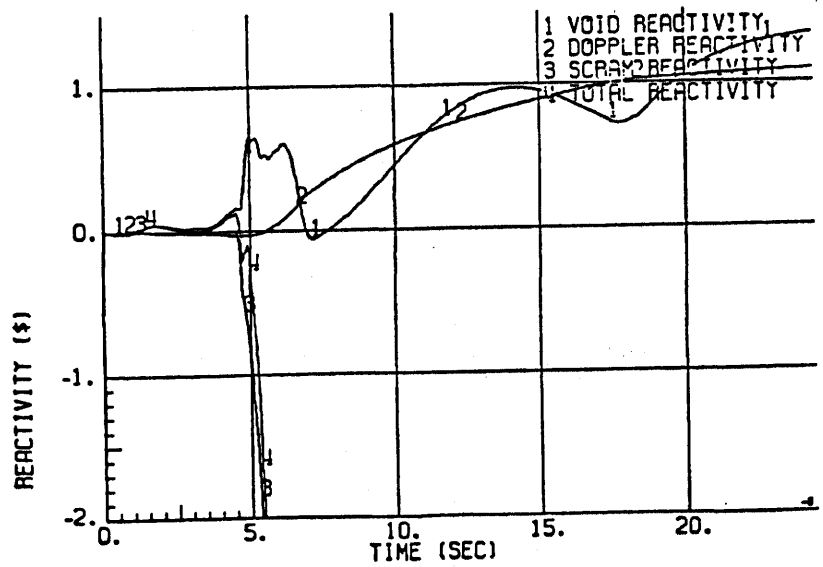
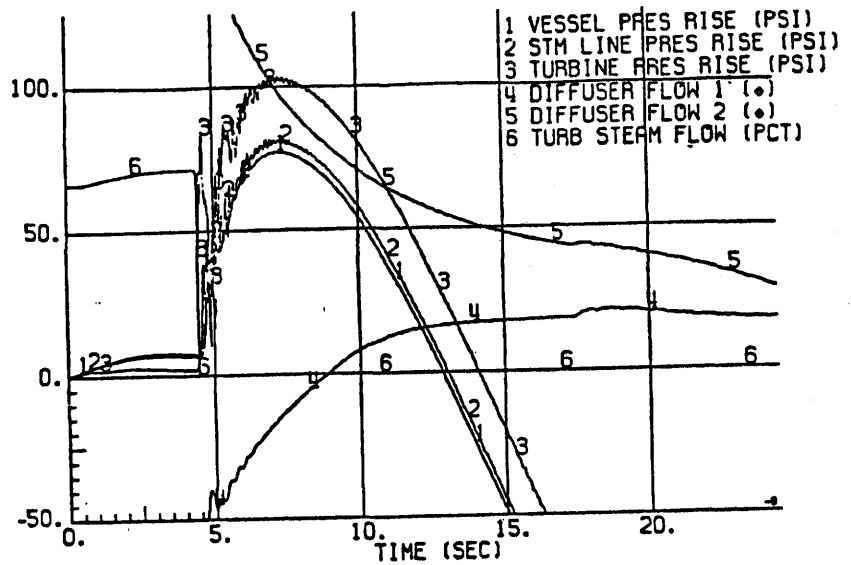
(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure -
Maximum Demand
Single Loop Operations

Figure 15F.3-2 (Sheet 1 of 2)



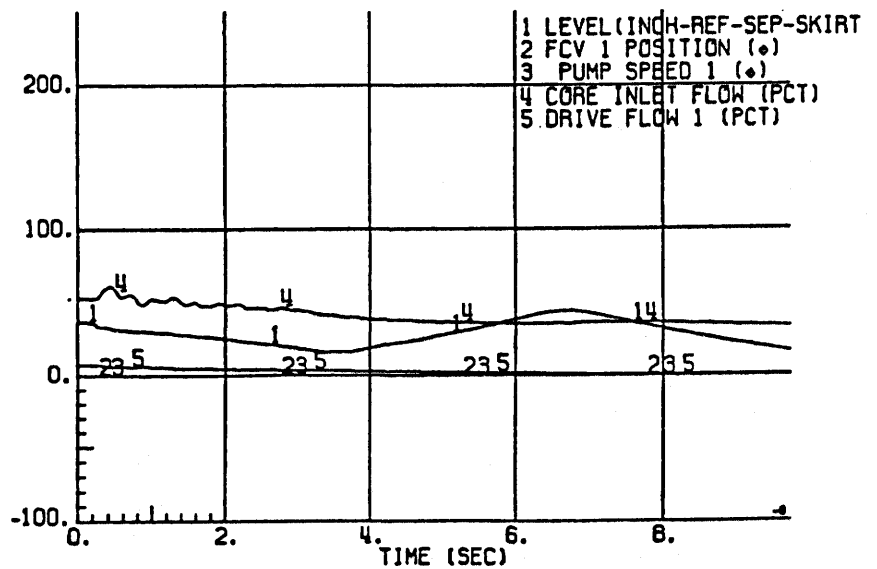
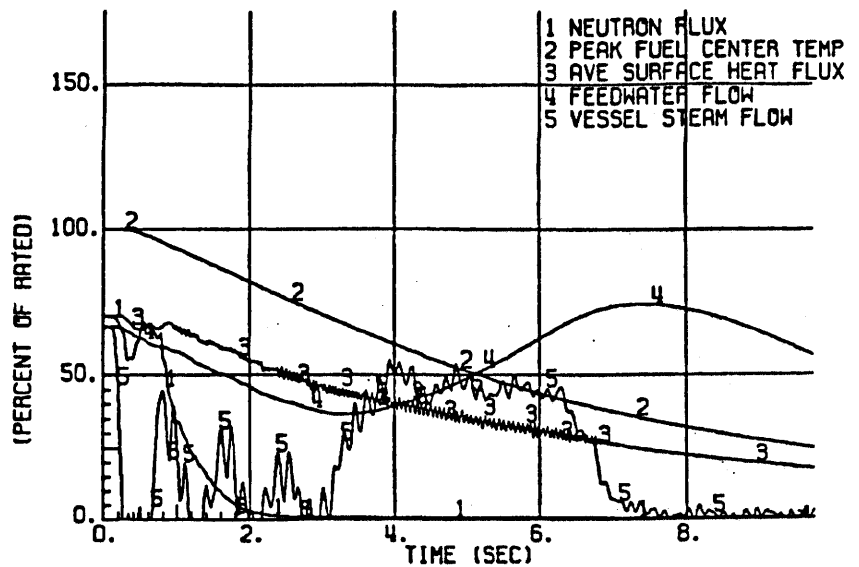
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PERRY NUCLEAR POWER PLANT

Feedwater Controller Failure -
 Maximum Demand
 Single Loop Operations

Figure 15F.3-2 (Sheet 2 of 2)



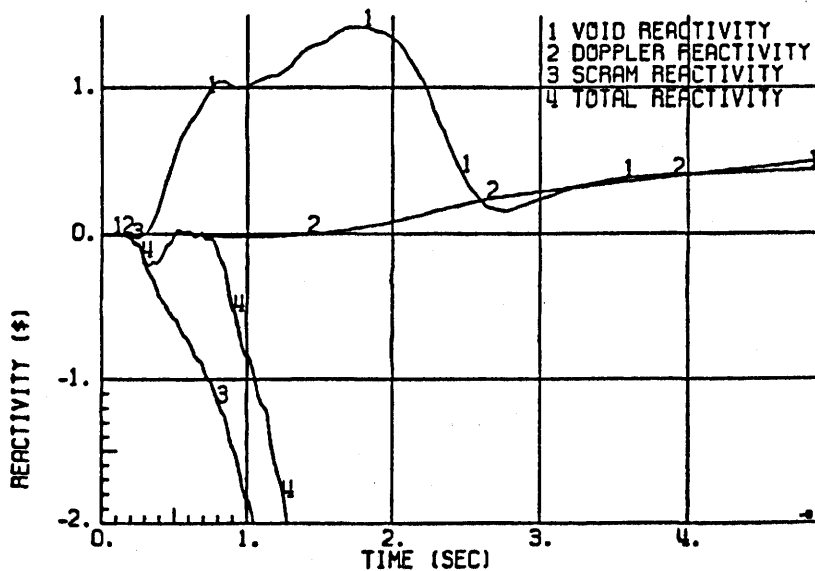
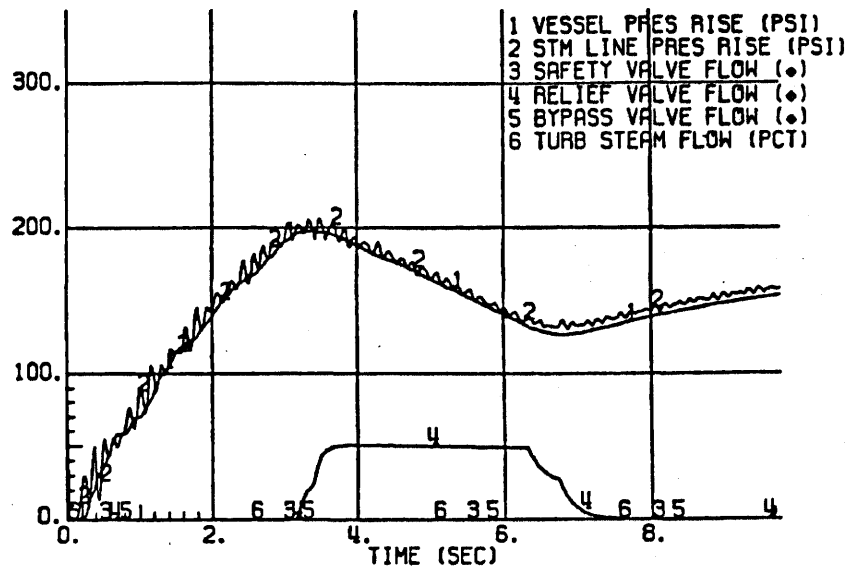
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PERRY NUCLEAR POWER PLANT

Generator Load Rejection,
W/O Bypass Single Loop
Operation

Figure 15F.3-3 (Sheet 1 of 2)



(Rev. 12 1/03)



PERRY NUCLEAR POWER PLANT

Generator Load Rejection,
 W/O Bypass
 Single Loop Operation

Figure 15F.3-3 (Sheet 2 of 2)