

October 11, 2019

Docket No. 52-048

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
One White Flint North
11555 Rockville Pike
Rockville, MD 20852-2738

SUBJECT: NuScale Power, LLC Submittal of Changes to Final Safety Analysis Report, Section 14.3, "Certified Design Material and Inspections, Test, Analyses and Acceptance Criteria"

REFERENCES:

1. Letter from NuScale Power, LLC to Nuclear Regulatory Commission, "NuScale Power, LLC Submittal of the NuScale Standard Plant Design Certification Application, Revision 3," dated August 22, 2019 (ML19241A315)
2. Letter from Nuclear Regulatory Commission, "NuScale Power, LLC, Design Certification Application – Chapter 14, Section 14.3 Initial Test Program, and Inspections, Tests, Analyses and Acceptance Criteria Phase 2 SE with Open Items" dated May 21, 2019 (ML19066A189)

In the Phase 2 SER the staff identified an editorial error in Tier 2, Section 14.3 Table 14.3-1 "Module-Specific Structures, Systems, and Components Based Design Features and Inspections, Test, Analyses, and Acceptance Criteria Cross Reference," as Open Item 14.3.6-1." As a result, NuScale changed the Final Safety Analysis Report (FSAR), Section 14.3, "Certified Design Material and Inspections, Test, Analyses and Acceptance Criteria" to close Open Item 14.3.6-1. The Enclosure to this letter provides a mark-up of the FSAR pages incorporating revisions to Section 14.3, in redline/strikeout format. NuScale will include this change as part of a future revision to the NuScale Design Certification Application.

This letter makes no regulatory commitments or revisions to any existing regulatory commitments.

If you have any questions, please feel free to contact Nadja Joergensen at 541-452-7338 or at njoergensen@nusclepwr.com.

Sincerely,



Michael Melton
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Enclosure: Changes to NuScale Final Safety Analysis Report Section 14.3, "Certified Design Material and Inspections, Test, Analyses and Acceptance Criteria"

Enclosure:

Changes to NuScale Final Safety Analysis Report Section 14.3, "Certified Design Material and Inspections, Test, Analyses and Acceptance Criteria"

**Table 14.3-1: Module-Specific Structures, Systems, and Components Based Design Features
and Inspections, Tests, Analyses, and Acceptance Criteria Cross Reference⁽¹⁾ (Continued)**

ITAAC No.	System	Discussion	DBA	Internal/External Hazard	Radiological	PRA & Severe Accident	FP
02.01.10	NPM	<p>FSAR Section 8.1.4.5.3 “Regulatory Requirements and Guidance” outlines the applicable General Design Criteria, NRC Regulations, RGs, and Branch Technical Positions, NUREG Reports, SECY Papers, and NRC Bulletins, and also discusses that the NPM CNTS containment electrical penetration assemblies are sized to power their design loads as demonstrated by satisfying the guidance of RG 1.63.</p> <p>An analysis determines the required design electrical rating needed to power the design loads of each NPM CNTS containment electrical penetration assembly listed in Tier 1 Table 2.1-3 (Table 14.3-3c).</p> <p>An ITAAC inspection is performed to verify that the electrical rating of each NPM CNTS containment electrical penetration assembly listed in Tier 1 Table 2.1-3 (Table 14.3-3c) is greater than or equal to the required design electrical rating. This ITAAC inspection may be performed any time after manufacture of the CNTS containment electrical penetration assemblies.</p>	X				
02.01.11		Not used.					
02.01.12	NPM	<p>Section 5.3.1.6, Material Surveillance, discusses the use of specimen capsules installed in specimen guide baskets.</p> <p>An ITAAC inspection is performed to verify that the correct number of guide baskets are attached to the outer surface of the core barrel at about the mid height of the core support assembly at locations where the capsules will be exposed to a neutron flux consistent with the objectives of the RPV surveillance program.</p>	X				