

## **NRR-DRMAPEm Resource**

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**From:** Chawla, Mahesh  
**Sent:** Thursday, October 3, 2019 4:52 PM  
**To:** Schultz, Eric  
**Cc:** Catron, Steve (Steve.Catron@fpl.com); Mack, Jarrett; Schneider, Thomas; Hartman, Thomas; Barclay, Kevin; Duncan, Eric; Sheng, Simon; Alley, David; Skokowski, Richard  
**Subject:** FINAL- Request for Additional Information - Relief Request 1-RR-13 and 2-RR-13, Proposed Alternative in Accordance with 10 CFR 50.55a(z)(1), Extension of Inspection Interval for PBNP Units 1&2 RPV Welds from 10 to 20 Years - EPID L-2019-LLR-0060

Dear Mr. Schultz,

By letter dated June 19, 2019 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML19170A285), NextEra Energy Point Beach, LLC (the licensee) requested an alternative to the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, Table IWB-2500-1 for Category B-A and B-D examinations for Point Beach Nuclear Plant (Point Beach), Units 1 and 2 reactor pressure vessel (RPV) welds and nozzle welds.

Specifically, pursuant to Title 10 of the Code of Federal Regulations (10 CFR) 50.55a(z)(1), the licensee requested to use the proposed alternative to extend the fifth inservice inspection (ISI) interval at Point Beach, Units 1 and 2 for Category B-A and B-D examinations so that the fifth ASME Code required examination can be performed in 2029 for both units on the basis that the alternative provides an acceptable level of quality and safety. The U.S. Nuclear Regulatory Commission (NRC) staff reviewed the submittal and needs the following additional information to complete the review.

### RAI-1

Table 3 in Title 10 of Code of Federal Regulations Part 50.61a, "Alternative fracture toughness requirements for protection against pressurized thermal shock events" lists maximum number of flaws depthwise for plates and forgings of a generic RPV. However, Table 2 of Alternative Request 1-RR-13 shows different maximum number of plate flaws for the Point Beach, Unit 1 RPV beltline, considering the plant-specific RPV geometry. Please explain how 3628 in<sup>2</sup> of inside surface area was calculated to support the differences between the values in Table 3 of 50.61a and Table 2 of 1-RR-13.

The draft version of the RAI was sent to you on October 01, 2019 to ensure that the request is understandable and the regulatory basis for the request is clear. The follow up clarification call was held today (October 3, 2019) with your representatives. As per the discussion on the teleconference this afternoon, the staff has revised the subject RAI. Your representative, Jarrett Mack has agreed to provide your response on the docket within 30 days of the receipt of this email. This email does not convey or represent an NRC staff position regarding NextEra's request. Thanking you,

Sincerely,

Mahesh Chawla  
Division of Operating Reactor Licensing  
Licensing Branch LPL-3

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