

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 F L Q R P 3 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 4 5

LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 37 CAT 52

CONT 01 REPORT SOURCE L 0 5 0 - 0 3 0 2 7 0 7 3 1 7 9 3 0 8 2 1 7 9 9

DOCKET NUMBER 55 58 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 At 1515 on 7/31/79 and again at 1130 on 8/1/79, it was reported that

03 Reactor Building atmospheric monitor RMA-6 was found isolated. This created an

04 event contrary to Technical Specification 3.4.6.1. Redundancy NA. No effect

05 upon the plant or general public. This is the first occurrence of this type

06 reported.

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SYSTEM CODE B B 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE V A L I V E X 14 COMP SUBCODE A 15 VALVE SUBCODE D 16

LER NO 7 9 EVENT YEAR 7 9 SEQUENTIAL REPORT NO. 0 7 0 OCCURRENCE CODE 0 1 3 REPORT TYPE L REVISION NO. 0

ACTION TAKEN B 18 FUTURE ACTION H 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 ATTACHMENT SUBMITTED Y 22 NRC04 ACRN005. PRIME CORP SUPPLIER A 23 COMPONENT MANUFACTURER F 1 6 6

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 The cause of this event is attributed to damaged wires to the control switches

11 for containment monitoring isolation valves WSV-5 and WSV-6. The wires were

12 repaired and RMA-6 was returned to service at 1740 on 31 July 1979 and at 1440

13 on 1 August 1979 respectively.

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FACILITY STATUS C 25 POWER 0 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Technician Observation 32

ACTIVITY CONTENT Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36

PERSONNEL EXPOSURES NUMBER 0 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39

PERSONNEL INJURIES NUMBER 0 0 0 0 40 DESCRIPTION NA 41

LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43

PUBLICITY N 44 DESCRIPTION NA 45

2036 188 NRC USE ONLY

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(SEE ATTACHED SUPPLEMENTARY INFORMATION SHEET)

7908310309

SUPPLEMENTARY INFORMATION

Report No.: 50-302/79-070/03L-0
Facility: Crystal River Unit #3
Report Date: 21 August 1979
Occurrence Dates: 31 July 1979 and 1 August 1979

Identification of Occurrence:

Reactor Coolant leakage detection System (RMA-6) inoperable
contrary to Technical Specification 3.4.6.1 a & c.

Conditions Prior to Occurrence:

Mode 2 (Rx Startup); Mode 1 (26% power operation)

Description of Occurrence:

At 1515 on 7/31/79 during routine inspection, RMA-6 vacuum pump was found isolated. Further investigation revealed that RMA-6 discharge isolation valve WSV-6 had failed closed. Maintenance restored WSV-6 and RMA-6 was returned to service at 1740 on 7/31/79.

At 1130 on 8/1/79 it was discovered that RMA-6 discharge isolation valve WSV-5 had failed closed, isolating RMA-6. Maintenance restored WSV-5 and RMA-6 was returned to service at 1440 on 8/1/79.

Designation of Apparent Cause:

The cause of this event is attributed to damaged wires which lead to the control switch for WSV-5 and WSV-6. The damage occurred as a result of modification (79-5-63) being installed in the vicinity of the switch wiring.

Analysis of Occurrence:

No effect upon the plant or general public as the plant remained in a stable condition.

Corrective Action:

The wiring to the control switches for WSV-5 and WSV-6 was repaired and RMA-6 was returned to service. Personnel were instructed of the importance of proper work techniques.

Failure Data:

This is the first occurrence of this type reported.

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