

# LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y J A F 1 2 0 0 - 0 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5  
 7 8 9 14 15 25 26 30 57 CAT 58  
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 3 3 3 7 0 5 2 4 7 9 8 0 6 2 0 7 9 9  
 7 8 60 61 68 69 74 75 80  
 DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 Please See Attachment  
 0 3  
 0 4  
 0 5  
 0 6  
 0 7  
 0 8

0 9 SYSTEM CODE W B 11 CAUSE CODE X 12 CAUSE SUBCODE X 13 COMPONENT CODE Z Z Z Z Z Z 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16  
 7 8 9 10 11 12 13 18 19 20  
 17 LER/RO REPORT NUMBER 7 9 21 22 23 24 0 3 3 26 27 28 0 3 29 30 31 32 0 33  
 ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
 X 18 X 19 Z 20 Z 21 0 0 0 0 22 Y 23 N 24 Z 25 Z 9 9 9 9 26  
 33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 Please See Attachment  
 1 1  
 1 2  
 1 3  
 1 4

1 5 FACILITY STATUS G 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY C 31 DISCOVERY DESCRIPTION Bulletin 79-02 Investigation 32  
 7 8 9 10 12 13 44 45 46 80  
 1 6 RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36  
 7 8 9 10 11 44 45 80  
 1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39  
 7 8 9 11 12 13 80  
 1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41  
 7 8 9 11 12 80  
 1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43  
 7 8 9 10 80  
 PUBLICITY 2238 258  
 ISSUED N 44 DESCRIPTION NA 45  
 7 8 9 10 80  
 7906250277 NRC USE ONLY

NAME OF PREPARER W. Verne Childs

PHONE: 315-342-3840

POWER AUTHORITY OF THE STATE OF NEW YORK  
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-033/03L-0

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While in a cold shutdown condition, during the conduct of a review of plant systems to satisfy the requirement of NRC Bulletin 79-02, it was determined that the Reactor Building Cooling System Supply and Return Lines to the Drywell Equipment Drain Sump did not have a seismic stress analysis performed. At this time it has not been determined, with certainty, if a seismic stress analysis is required for the 1 1/2" Cooling Water Supply and Return Lines associated with the sump cooler. If the investigation indicates that a seismic analysis should have been performed this event will be an example of failure to implement administrative or procedural instructions that are required by Technical Specification Appendix A, Paragraph 6.9.A.4.2.c.

When the investigation has been completed a follow-up to this interim report will be submitted.

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