

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01LQAD12000-000-00034111145

REPORT SOURCE L505000254701197980216799

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

While performing local leak rate testing, the Reactor Core Isolation Cooling System (RCIC) turbine steam exhaust check valve CV 1-1301-41 was found to have a leak rate of 396.3 SCFH. In the event of a high suppression chamber pressure, stop check valve 1-1301-41 was available and also could have been manually shut to minimize leakage back to the RCIC turbine. The leakage through CV 1-1301-41 did not render the RCIC incapable of performing its intended function.

SYSTEM CODE CE11 CAUSE CODE E12 CAUSE SUBCODE B13 COMPONENT CODE VALVEX14 COMP SUBCODE C15 VALVE SUBCODE D16

LER/RO REPORT NUMBER 17 EVENT YEAR 79 SEQUENTIAL REPORT NO. 003 OCCURRENCE CODE 03 REPORT TYPE L REVISION NO. 0

ACTION TAKEN D18 FUTURE ACTION Z19 EFFECT ON PLANT Z20 SHUTDOWN METHOD Z21 HOURS 0000 ATTACHMENT SUBMITTED Y23 NRPD-4 FORM SUB. N24 PRIME COMP. SUPPLIER N25 COMPONENT MANUFACTURER C66526

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

The valve internals were found to be dirty, thus preventing a good disc-to-seat seal. The valve internals were cleaned and the valve reassembled. A local leak rate test was then performed and the leak rate was found to be 0.0 SCFH.

FACILITY STATUS H28 % POWER 00029 OTHER STATUS NA30 METHOD OF DISCOVERY B31 DISCOVERY DESCRIPTION Local Leak Rate Testing32

ACTIVITY CONTENT RELEASED OF RELEASE Z33Z34 AMOUNT OF ACTIVITY NA35 LOCATION OF RELEASE NA36

PERSONNEL EXPOSURES NUMBER 00037 TYPE Z38 DESCRIPTION NA39

PERSONNEL INJURIES NUMBER 00040 DESCRIPTION NA41

LOSS OF OR DAMAGE TO FACILITY TYPE Z42 DESCRIPTION NA43

PUBLICITY ISSUED N44 DESCRIPTION NA45

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NRC USE ONLY

NAME OF PREPARER

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- I. LER NUMBER: LER/RO 79-03/03L-0
- II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit One
- IV. DOCKET NUMBER: 050-254
- V. EVENT DESCRIPTION:

On January 19, 1979, while performing Local Leak Rate Testing, the Reactor Core Isolation Cooling System (RCIC) turbine steam exhaust check valve CV 1-1301-41 was found to have a leak rate of 396.3 SCFH. The test was performed according to procedure QTS 100-21 by closing the 1-1301-64 stop check valve and pressurizing the volume between the two valves. The Technical Specification leak rate limit for any one primary containment isolation valve is 18.36 SCFH. Work request 357-79 was written to repair the valve.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The purpose of check valve CV 1-1301-41 is to prevent the back flow of steam from the pressure suppression chamber into the RCIC system. In the event of a high suppression chamber pressure, stop check valve 1-1301-41 was operable. It also could have been manually shut to minimize leakage back to the RCIC turbine. The leakage through CV 1-1301-41 did not render the RCIC system inoperable, nor incapable of performing its intended function when needed.

VII. CAUSE:

Upon disassembly, the valve internals were inspected and the valve disc and seating surfaces were found to be dirty, thus preventing a good disc-to-seat seal. Further inspection revealed no other abnormalities which could have caused the failure. Check valve CV 1-1301-41 is an 8 inch, 150 pound swing check valve manufactured by Crane Company.

VIII. CORRECTIVE ACTION:

The valve internals were cleaned and the valve was reassembled. A local leak rate test was then performed and the leak rate was found to be 0.0 SCFH.