

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 L Q A D 1 2 0 0 0 - 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5

LICENSEE CODE

LICENSE NUMBER

LICENSE TYPE

C/T 58

DON'T

REPORT

SOURCE

L 0 5 0 0 0 2 5 4 7 0 1 2 0 7 9 8 0 2 1 6 7 9 9

DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

While performing the feedwater check valve local leak rate test, the leakage rate through feedwater check valve 1-220-62B was found to be 2558 SCFH. This is in excess of the 18.36 SCFH allowable limit as specified by Technical Specification 4.7.A.2.i.2.b. The safety implications of this occurrence are minimal due to the fact that the inboard feedwater check valve, 1-220-58B, was tested and found to be within acceptable limits. Thus, any back leakage through the B feedwater line would have been stopped by check valve 1-220-58B.

SYSTEM CODE

CAUSE CODE

CAUSE SUBCODE

COMPONENT CODE

COMP. SUBCODE

VALUE SUBCODE

C H 11 E 12 B 13 V A L V E X 14 C 15 D 16

LER/RO REPORT NUMBER

EVENT YEAR

SEQUENTIAL REPORT NO.

OCCURRENCE CODE

REPORT TYPE

REVISION NO.

7 9 0 0 3 0 3 L 0

ACTION TAKEN

FUTURE ACTION

EFFECT ON PLANT

SHUTDOWN METHOD

HOURS

ATTACHMENT SUBMITTED

NPRD-4 FORM SUB.

PRIME COMP. SUPPLIER

COMPONENT MANUFACTURER

B 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 Y 24 N 25 C 6 6 5 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

The valve seat was found to be dirty preventing the proper seating of the valve disc. The "O" ring was found to be in acceptable condition. The valve seat was cleaned and a new Kalraz "O" ring was installed. After the valve was reassembled a second local leak rate test was performed. The corrected leak rate was found to be 4.4 SCFH.

FACILITY STATUS

% POWER

OTHER STATUS

METHOD OF DISCOVERY

DISCOVERY DESCRIPTION

ACTIVITY CONTENT

AMOUNT OF ACTIVITY

LOCATION OF RELEASE

RELEASED OF RELEASE

PERSONNEL EXPOSURES

NUMBER

TYPE

DESCRIPTION

PERSONNEL INJURIES

NUMBER

DESCRIPTION

LOSS OF OR DAMAGE TO FACILITY

TYPE

DESCRIPTION

PUBLICITY

ISSUED

DESCRIPTION

7903070358

NRC USE ONLY

NAME OF PREPARER

J. Schnitzmeyer

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- I. LER NUMBER: 79-03/03L-0
- II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit One
- IV. DOCKET NUMBER: 050-254
- V. EVENT DESCRIPTION:

On January 20, 1979, local leak rate testing was being performed in accordance with procedure QTS 100-25, Feedwater Check Valve Local Leak Rate Test. The leak rate through feedwater check valve 1-220-62B was found to be 2558 SCFH, which exceeds the 18.36 SCFH limit specified in the Technical Specifications. Work request 365-79 was written to investigate and repair the leakage.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The consequences of this event are minimized by the fact that the inboard feedwater check valve, 1-220-58B, was tested and found to be within acceptable limits. Thus, any back leakage through the B feedwater line would have been stopped by check valve 1-220-58B. Feedwater check valves in the "A" feedwater line 1-220-58A and 1-220-62A each had acceptable leak rate values.

VII. CAUSE:

Check valve 1-220-62B was disassembled under work request 365-79. The valve seat was found to be dirty preventing the proper seating of the valve disc. The "O"-ring was found to be in acceptable condition. The last failure of valve 1-220-62B was on March 22, 1977 during the last Unit One refueling outage. The check valve is an 18 inch swing type check valve manufactured by Crane Company.

VIII. CORRECTIVE ACTION:

The valve seat was cleaned and a new Kalraz "O" ring was installed. The valve was reassembled and successfully bench tested. After reinstalling the valve, a local leak rate test was performed on the valve. The corrected leak rate was found to be 4.4 SCFH.