

(7-77)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM CODE S D (11)		CAUSE CODE E (12)		CAUSE SUBCODE B (13)		COMPONENT CODE V A L V O P (14)				COMP. SUBCODE D (15)		VALVE SUBCODE Z (16)	
LER/RO REPORT NUMBER 7 9 (8)		EVENT YEAR 7 9 (21 22)		SEQUENTIAL REPORT NO. 0 0 6 (24 26)		OCCURRENCE CODE 0 3 (28 29)		REPORT TYPE L (30)		REVISION NO. 0 (32)			
ACTION TAKEN E (18)		FUTURE ACTION Z (19)		EFFECT ON PLANT Z (20)		SHUTDOWN METHOD Z (21)		HOURS 0 0 0 (22 23 24)		ATTACHMENT SUBMITTED Y (23)		NPRD-4 FORM SUB. Y (24)	
PRIME COMP. SUPPLIER N (25)		COMPONENT MANUFACTURER V 0 9 5 (26 27 28 29)											

8 9 FACILITY STATUS (28) H 0 0 0 % POWER (29) NA OTHER STATUS (30) METHOD OF DISCOVERY (31) B Routine Test DISCOVERY DESCRIPTION (32)

7 8 9 ACTIVITY CONTENT RELEASED OF RELEASE (33) Z Z (34) AMOUNT OF ACTIVITY (35) NA LOCATION OF RELEASE (36)

		LOSS OF OR DAMAGE TO FACILITY		(43)
		TYPE	DESCRIPTION	
1	9	Z	(42) NA	(43)

PUBLICITY
 ISSUED DESCRIPTION (45) NA 79030700357
 (2) (0) (N) (44) 68 69 80
 7 8 9 10

NAME OF PREPARER Jim Swales

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- I. LER NUMBER: LER/RO 79-06/03L-0
- II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit One
- IV. DOCKET NUMBER: 050-254
- V. EVENT DESCRIPTION:

On January 25, 1979, while performing Primary Containment Isolation (PCI) Valve Closure Timing, procedure QOS 1600-14, the A0 1-1601-21 valve failed to close in the required length of time. Technical Specification Table 3.7-1 requires this valve to close in less than or equal to 10 seconds. The valve actually closed in 12 seconds. Unit One was shutdown for refueling at the time of this occurrence. The last time this valve failed the closure timing test was May 14, 1974 and was reported in incident number 1-4-1-74-18.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The A0 1-1601-21 valve is a Drywell Purge Valve which is normally closed when the unit is operating. In addition all PCI valves downstream of A0 1-1601-21 (A0 1-1601-22, A0 1-1601-55, A0 1-1601-56) had acceptable valve closure times. Valve A0 1-1601-21 had been functionally tested for isolation capability using procedure QOS 1600-12. Safe reactor operation was not affected by this occurrence.

VII. CAUSE:

The cause of this occurrence is component failure. Upon investigation, it was found that the speed control on the valve operator was out of adjustment. The apparent cause of the misadjustment is normal operational wear. The speed control is part of a solenoid-operated 4-way Versa valve model VGS-4422-U.

VIII. CORRECTIVE ACTION:

Corrective action taken was to readjust the 4-way solenoid valve speed control. Valve 1-1601-21 was then retimed on February 6, 1979 and had a closure time of 7 seconds.