

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | 1LQAD1 2000 - 090 - 000 341111 45

LICENSEE CODE

LICENSE NUMBER

LICENSE TYPE

CAT 58

CONT

REPORT SOURCE

L605000254 7012479 8021679 9

DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

While performing local leak rate testing, reactor building-to-suppression chamber vacuum breaker 1-1601-31B was found to have a leak rate of 147.7 SCFH. The consequence of this occurrence was minimal because the A0 1-1601-20B valve, which is in-line with the 1-1601-31B valve, would have provided primary containment isolation.

SYSTEM CODE

CAUSE CODE

CAUSE SUBCODE

COMPONENT CODE

COMP. SUBCODE

VALVE SUBCODE

SD 11

E 12

B 13

VALVEX 14

C 15

C 16

LER/RO REPORT NUMBER 17

EVENT YEAR

79

SEQUENTIAL REPORT NO.

003

OCCURRENCE CODE

03

REPORT TYPE

L

REVISION NO.

0

ACTION TAKEN

FUTURE ACTION

EFFECT ON PLANT

SHUTDOWN METHOD

HOURS

ATTACHMENT SUBMITTED

NPRD-4 FORM SUB.

PRIME COMP. SUPPLIER

COMPONENT MANUFACTURER

D18Z19

Z20

Z21

0000

Y23

Y24

A25

A585 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

The cause of this occurrence was component failure. The leakage was coming through the teflon packing around the valve shaft. The valve packing was replaced and a second leak rate test was performed on the volume between valves 1-1601-31B and A0 1-1601-20B. The corrected total leak rate was found to be 0.67 SCFH.

FACILITY STATUS

% POWER

OTHER STATUS

METHOD OF DISCOVERY

DISCOVERY DESCRIPTION

H 28

000 29

NA 30

B 31

Local Leak Rate Testing 32

ACTIVITY RELEASED

CONTENT OF RELEASE

AMOUNT OF ACTIVITY

LOCATION OF RELEASE

Z33Z34

NA 35

NA

36

PERSONNEL EXPOSURES

NUMBER

TYPE

DESCRIPTION

000 37

Z 38

NA 39

PERSONNEL INJURIES

NUMBER

DESCRIPTION

000 40

NA 41

LOSS OF OR DAMAGE TO FACILITY

TYPE

DESCRIPTION

Z 42

NA 43

PUBLICITY

ISSUED

DESCRIPTION

Z 44

NA 45

NRC USE ONLY

NAME OF PREPARER

J. Swales

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7903070340

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- I. LER NUMBER: LER/RO 78-03/03L-0
- II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit One
- IV. DOCKET NUMBER: 050-254
- V. EVENT DESCRIPTION:

On January 24, 1979, a local leak rate test of reactor building-to-suppression chamber vacuum breakers, 1-1601-31B and AO 1-1601-20B, revealed an excessive leak rate of 147.7 SCFH. The test was performed in accordance with procedure QTS 100-29, and the acceptance criteria is 18.36 SCFH according to Technical Specification 4.7.2.i.2.b. Valves AO 1-1601-20B and 1-1601-31B were leak rate tested by pressuring the volume between them to 48 psig and measuring the pressure decay. During the leak test, it was found that virtually all of the leakage was coming through the stem packing on the 1-1601-31B valve. Work request number 448-79 was written to repair the valve.

The last failure of a reactor building to suppression chamber vacuum breaker to pass its leak test occurred on Unit Two, January 18, 1978, and is reported in LER/RO 78-03/03L.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The consequences of this occurrence were minimal because the AO 1-1601-20B valve, which is in-line with the 1-1601-31B valve, would have provided the necessary isolation. In addition, the redundant suppression chamber to reactor building vacuum breakers, AO 1-1601-20A and 1-1601-31A, were fully operable and passed their local leak rate test.

V. CAUSE:

The cause of this occurrence was component failure. During the local leak rate test, it was determined that most of the leakage was coming through the teflon packing around the valve shaft. The 1-1601-31B valve is a 20 inch, 150 pound vacuum breaker check valve, manufactured by the Atwood and Morrill Company.

VIII. CORRECTIVE ACTION:

The teflon packing was replaced on both sides of the valve shaft. The volume was again local leak rate tested and the total leak rate was found to be 0.67 SCFH.