

**LICENSEE EVENT REPORT**

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	1	4	Q	A	D	1	2	0	0	0	-	0	0	0	-	0	0	0	3	4	1	1	1	1	4			5					
9		LICENSE CODE						14	LICENSE NUMBER												25	LICENSE TYPE						30	57 CAT 58					

SOURCE

REPORT SOURCE 1 6 0 5 0 0 0 2 5 4 7 0 1 3 0 7 9 8 0 2 1 6 7 9 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

While performing the local leak rate test on the drywell floor drain sump isolation

valves, A0 1-2001-3 and 4, it was determined that the 1-2001-4 valve leak rate

was 26.0 SCFH. The safety implications of this occurrence are minimal due to the fact

that the redundant isolation valve in the drywell sump line, A0 1-2001-3, had a

leak rate of only 1.85 SCFH.

SYSTEM CODE S D 11		CAUSE CODE F 12		CAUSE SUBCODE R 13		COMPONENT CODE V A L V E X 14				COMP. SUBCODE E 15		VAL. SUBCODE D 16	
LER/RO REPORT NUMBER 7 9 17		EVENT YEAR 7 9 21 22		SEQUENTIAL REPORT NO. 0 0 3 24 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO. 0 32			
ACTION TAKEN A 18		FUTURE ACTION Z 19		EFFECT ON PLANT Z 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 0 22		ATTACHMENT SUBMITTED Y 23		NPRD-4 FORM SUB. N 24	
PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER C 6 6 5 26											

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The cause of the excessive leakage was due to a worn valve stem packing on the 1-2001-4

valve. The valve packing was replaced and the volume between valves A0 1-2001-3 and

4 was retested. The corrected combined leak rate is 1.85 SCFH.

FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION		
1	5	H	28	0	0	0	29	NA	30	B	31	Local Leak Rate Testing		
ACTIVITY CONTENT			RELEASED OF RELEASE			AMOUNT OF ACTIVITY			LOCATION OF RELEASE					
1	6	Z	33	Z	34	35	NA		36					
PERSONNEL EXPOSURES			PERSONNEL INJURIES			LOSS OF OR DAMAGE TO FACILITY			PUBLICITY			NRC USE ONLY		
NUMBER			NUMBER			TYPE			DESCRIPTION					
1	7	0	0	0	0	37	Z	38	39	NA				
TYPE			DESCRIPTION			TYPE			DESCRIPTION					
1	8	0	0	0	0	40		41	NA					
TYPE			DESCRIPTION			TYPE			DESCRIPTION					
1	9	Z	42			43	NA							
ISSUED			DESCRIPTION			7903070338								
2	0	N	44			45	NA							

G. Tietz

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- I. LER NUMBER: LER/RO 79-03/03L-0
- II. LICENSEE NAME: Commonwealth Edison Company  
Quad-Cities Nuclear Power Station
- III. FACILITY NAME: Unit One
- IV. DOCKET NUMBER: 050-254
- V. EVENT DESCRIPTION:

On January 30, 1979 while performing a local leak rate test on the drywell floor drain sump isolation valves, A0 1-2001-3 and 4, it was determined that the 1-2001-4 valve leak rate was 26 SCFH. This is in excess of the 18.36 SCFH allowable limit as specified by Technical Specification 4.7.A.2.i.2.b. The test was done in accordance with procedure QTS 100-16. While the volume between the two valves was pressurized, application of a soap bubble solution indicated leakage through the stem packing of the 1-2001-4 valve.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The safety implications of this occurrence are minimal due to the fact that the redundant isolation valve in the drywell sump line, A0 1-2001-3, had a leak rate of only 1.85 SCFH. Therefore, had a Group Two isolation occurred, the drywell sump line would have been isolated.

VII. CAUSE:

The cause of this occurrence is attributed to component failure. Worn valve stem packing caused the leakage. The packing was manufactured by Crane Packing Company, style number 1871, size one inch by one and a half inch.

VIII. CORRECTIVE ACTION:

The immediate corrective action was to write a work request to replace the valve packing. After the packing was replaced, the valve was local leak rate tested again. The corrected combined leak rate of the volume between both valves is 1.85 SCFH.