

## LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 A L B R F 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 5 9 7 0 1 1 8 7 9 8 0 1 3 1 7 9 9  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While pulling rods at the end of a refueling outage, the reactor scrambled due to high  
0 3 flux actuating the IRM high-high neutron monitoring circuit. The reactor was approach  
0 4 ing criticality in the B1 number 1 rod sequence. The estimated reactor period  
0 5 approached 5 seconds. Reactivity anomaly is reported in accordance with T.S. 6.7.2.a.4.  
0 6 There have been no previous occurrences. There was no hazard to the public health or  
0 7 safety.

0 8  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

0 9 SYSTEM CODE I D 1 CAUSE CODE A 12 CAUSE SUBCODE A 13 COMPONENT CODE Z Z Z Z Z Z Z 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

17 LER/RO REPORT NUMBER 7 9 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

ACTION TAKEN H 18 FUTURE ACTION H 19 EFFECT ON PLANT C 20 SHUTDOWN METHOD C 21 HOURS 0 0 1 2 22 ATTACHMENT SUBMITTED Y 23 NPD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER Z 25 COMPONENT MANUFACTURER Z Z Z Z Z 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The fast period was caused by withdrawal of a high reactivity worth rod. Normal plant  
1 1 practices were being followed. Administrative controls shall require the first rod  
1 2 in a RWM group to be notched prior to criticality. Precautions have been placed into  
1 3 operating instructions.

1 4  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 5 FACILITY STATUS C 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION NA 32  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 6 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

PUBLICITY ISSUED N 44 DESCRIPTION NA 45  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

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NRC USE ONLY

NAME OF PREPARER

PHONE:

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 791 Technical Specification Involved None

Reported Under Technical Specification 6.7.2.a.4

Date of Occurrence 1-18-79 Time of Occurrence 2328 Unit 1

Identification and Description of Occurrence:

Reactor scrambled due to high flux detected by the IRM neutron monitoring circuit.

Conditions Prior to Occurrence:

Pulling control rods to obtain criticality.

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

None

Apparent Cause of Occurrence:

Too rapid withdrawal of a high reactivity worth control rod.

Analysis of Occurrence:

Withdrawal of high reactivity worth control rods must be slow enough to allow full development of ensuing reactor periods.

Corrective Action:

The fast period was caused by withdrawal of a high reactivity worth control rod. The operator, following normal practice, allowed the rod to continuously withdraw since neutron monitoring instruments indicated the reactor to be far subcritical. Operator action to reinsert the rod failed to reduce the period and resulted in a reactor trip. Administrative controls have been implemented to require the first

Failure Data: rod in a RWM group to be notched before criticality. Precautions pertaining to high rod worths have been inserted into operating instructions.

None

Tennessee Valley Authority - Browns Ferry Nuclear Plant