

# LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y J A F 1 0 0 - 0 0 0 0 - 0 0 0 3 4 1 1 1 1 4 5  
7 8 9 14 15 25 26 30 57 CAT 58

CONT

0 1 L 0 5 0 0 0 3 3 3 1 2 0 6 7 8 1 2 2 7 7 8 9  
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)  
Please See Attachment

0 2  
0 3  
0 4  
0 5  
0 6  
0 7  
0 8

0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE  
X X 11 A 12 D 13 Z Z Z Z Z Z 14 Z 15 Z 16  
7 8 9 10 11 12 13 14 15 16 17 18 19 20  
17 LER RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.  
7 8 7 8 0 9 6 0 3 L 0  
21 22 23 24 25 26 27 28 29 30 31 32  
ACTION FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER  
X 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 N 24 Z 25 7 9 9 9 26  
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Please See Attachment  
1 1  
1 2  
1 3  
1 4

1 5 FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION  
C 28 0 0 1 29 NA B 31 Reactor Water Chemistry Surveillance  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
1 6 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE  
Z 33 Z 34 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION  
0 0 0 37 Z 38 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
1 8 PERSONNEL INJURIES NUMBER DESCRIPTION  
0 0 0 40 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION  
Z 42 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
2 0 PUBLICITY ISSUED DESCRIPTION  
N 44 NA  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

NAME OF PREPARER W. Verne Childs

PHONE: 315-342-3840

7901030148

During a routine reactor startup, at a steaming rate of approximately 60,000 pounds per hour, a reactor coolant sample exceeded the chloride ion limits of Technical Specification Appendix A, Paragraph 3.6.C.2. Sample analysis indicated 0.180 ppm chloride. Operation of the reactor with high concentrations of chloride ions present could cause or accelerate intergranular stress corrosion cracking of certain stainless steel components.

Accidental "contamination" of the sample with some material containing significant chloride ions apparently caused the high chloride ion sample. This conclusion is supported by earlier and later sample data as shown below:

SAMPLE	DATE/TIME	APPROXIMATE STEAMING (Pound/Hour)	CHLORIDE (ppm)	CONDUCTIVITY (Micromhos/CM <sup>2</sup> )
1	12/ 5/78/1445	0	0.055	1.05
2	12/ 5/78/1840	0	0.040	1.13
3	12/ 5/78/2235	60,000	0.180	0.90
4	12/ 5/78/2305	100,000	0.075	0.90
5	12/ 6/78/0235	100,000	0.044	0.98

Sample number 3, above, was the only out of specification sample. Sample 4, obtained only 30 minutes later, indicates a chloride ion reduction to less than one half of that indicated in sample 3. Since the reactor water cleanup system cleanup "half life" is approximately 60 hours, samples 4 and 5 clearly support the conclusion that sample 3 was inadvertently contaminated.