

BEFORE THE UNITED STATES
NUCLEAR REGULATORY COMMISSION

In the Matter of)
POWER AUTHORITY OF THE STATE OF NEW YORK) Docket No. 50-333
James A. FitzPatrick Nuclear Power Plant)


APPLICATION FOR AMENDMENT TO
OPERATING LICENSE

Pursuant to Section 50.90 of the regulations of the Nuclear Regulatory Commission (NRC), the Power Authority of the State of New York, as holder of Facility Operating License No. DPR-59, hereby applies for an Amendment to the Technical Specifications contained in Appendix A of this license.

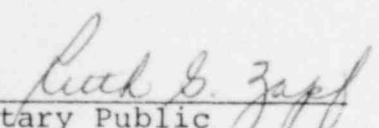
The proposed change to the FitzPatrick Technical Specification seeks revision to Section 4.11.D.1.b for surveillance requirements regarding emergency service water pump test acceptance criteria.

Proposed changes to the Technical Specifications are presented in Attachment I to this Application. The Safety Evaluation is included in Attachment II.

POWER AUTHORITY OF THE STATE
OF NEW YORK

By 
Paul J. Early
Assistant Chief Engineer-Projects

Subscribed and sworn to before
me this 26 day of June, 1979.


Notary Public
RUTH G. ZAPF
Notary Public, State of New York
No. 30-4663428
Qualified in Nassau County
Commission Expires March 30, 1980

2170 149

7906280124 P

ATTACHMENT I

PROPOSED TECHNICAL SPECIFICATION CHANGES
RELATED TO
EMERGENCY SERVICE WATER PUMP TESTING

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333
JUNE 22, 1979

2170 150

3.11 (cont'd)

JAFNPP

4.11 (cont'd)

D. Emergency Service Water System

1. To ensure the adequate equipment and area cooling, operable when the requirements of specification 3.5.A and 3.5.B must be satisfied, except as specified below in specification 3.11.D.2.

D. Emergency Service Water System

1. Surveillance of the ESW system shall be performed as follows:

<u>Item</u>	<u>Frequency</u>
a. Simulated Automatic Actuation Test	Each operating cycle
b. Flow Rate Test - ESW pumps shall deliver at least 3,250 gpm against a system head corresponding to a total pump head of ≥ 80 psi, as determined from the pump certification curve by measuring the pump shutoff head which shall be ≥ 117 psi	Once/ 3 months
c. Pump Operability	Once/month
d. Motor Operated Valves	Once/month

2170 151

ATTACHMENT II

SAFETY EVALUATION

EMERGENCY SERVICE WATER PUMP TESTING

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333
JUNE 22, 1979

2170 152

SAFETY EVALUATION

EMERGENCY SERVICE WATER PUMP TESTING

Section I - Basis for the Modification

Technical Specification Section 4.11.D.1.b has been proposed to be modified in order to allow for normal emergency service water pump wear while still assuring capability to meet system requirements.

Emergency Service Water System flow requirements are 2915 gpm as stated in Table 9.7.1 of the FSAR. The Technical Specification refers to 3700 gpm at 80 psi which is taken off the pump curve at design point.

Recent shutoff head measurements (Ref. 1) represent a reduction of 2.0 psi from the original pump curve. In discussions with the pump vendor a 4% reduction in shutoff head identified as a point where overhaul should reasonably be considered. The resulting criteria would be approximately 117 psi. The pump vendor further indicated that reductions in shutoff head, if applied throughout the head-capacity curve, would provide a conservative indication of pump performance after normal wear. The resulting capacity at 80 psi would be 3250 gpm; i.e., 11.5% above the required capacity of 2915 gpm.

Section II - Impact of the Change

A review of the FSAR and SER Accident Analyses indicate that there are no changes to the parameters used, and that the conclusions reached are unchanged, as a result of this modification.

Section III - Implementation of the Modification

The modifications as proposed will not impact the ALARA or Fire Protection Program at JAF.

Section IV - Conclusions

The incorporation of this modification: a) will not increase the probability nor the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the Safety Analysis Report; b) will not increase the possibility for an accident or malfunction of a different type than any evaluated previously in the Safety Analyses Report; and c) will not reduce the margin of safety as defined in the basis for any Technical Specification.

Section V - References

1. Letter: J.D. Leonard, Jr. (PASNY) to Mr. Boyce H. Grier (NRC) of May 24, 1979.
2. JAF FSAR
3. JAF SER

2170 153