THIS DOCUMENT CONTAINS
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OPERATING DATA REPORT

DOCKET NO. 50-334
DATE 2/5/79
COMPLETED BY F. P. Witkowsky
TELEPHONE 412-643-5023

OPERATING STATUS

1. Unit Name: Beaver Valley Power Station, Unit #1
2. Reporting Period:
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 800

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	744	24144
12. Number Of Hours Reactor Was Critical	366.92	366.92	10562.99
13. Reactor Reserve Shutdown Hours	417.07	417.07	4899.87
14. Hours Generator On-Line	339.16	339.16	9918.86
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	787964.87	787964.87	21656359.82
17. Gross Electrical Energy Generated (MWH)	256400	256400	6595140
18. Net Electrical Energy Generated (MWH)	234579	234579	6026947
19. Unit Service Factor	45.5	45.5	47.2
20. Unit Availability Factor	45.5	45.5	47.2
21. Unit Capacity Factor (Using MDC Net)	39.4	39.4	41.6
22. Unit Capacity Factor (Using DER Net)	36.9	36.9	38.6
23. Unit Forced Outage Rate	55.2	55.2	40.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Weekend outage in March for a transformer change out. Refueling outage on 23 of April, 1979.

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	N/A	N/A
INITIAL ELECTRICITY	N/A	N/A
COMMERCIAL OPERATION	N/A	N/A

7902120062

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-334
UNIT BVPS Unit #1
DATE 2/5/79
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MONTH January, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	801	17	0
2	804	18	381
3	600	19	0
4	0	20	493
5	0	21	721
6	0	22	808
7	0	23	788
8	0	24	829
9	0	25	821
10	0	26	296
11	0	27	285
12	0	28	192
13	0	29	607
14	0	30	816
15	0	31	759
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1979

DOCKET NO. 50-334
UNIT NAME BVPS Unit #1
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No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	790103	F	347.98	A	3	79-4	HP	VALVEX	Reactor trip from 99% F.P. and safety injection due to high steam line flow coincident with low steam pressure in the A & B loops when C main steam stop valve tripped close.
2	780118	F	25.40	A	3	79-5	HC	VALVEX	Reactor trip due to high steam flow coincident with low steam pressure and a safety injection signal. Heater drain valve [LCV-SD106A] failed open tripping main feed pumps and main steam dump valves cycled open. Heater drain valve [LCV-SD106A] was equalized to preventing open.
3	790120	F	3.57	A	3	79-6	EE	CKTBRK	Reactor trip from full power caused by loss of the #3 inverter. Failure of the #3 uninterruptible power supply resulted in loss of 1C reactor coolant pump breaker #3 above P8.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5
Exhibit I - Same Source

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1979

DOCKET NO. 50-334
UNIT NAME BVPS Unit #1
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No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	790126	F	13.14	G	3	None	CH	ZZZZZZ	Reactor trip from 100% F.P. on low pressurizer pressure due to over-feeding the 1B steam generator followed by a feedwater isolation and reactor trip on high steam generator level of 1B steam generator.
5	790128	F	11.20	A	3	None	CJ	VALVEX	Reactor trip after reducing power below 10% (P-7) after losing cooling water to the 1C reactor coolant pump. A blown diaphram on component cooling water trip valve [TV-CCR103C1] was replaced.
6	790129	F	3.55	A	3	None	CH	VALVEX	Reactor trip on Lo-Lo steam generator level due to intermittent failure to feed regulation bypass valve to go close.

1
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(9/77)

DUQUESNE LIGHT COMPANY
Beaver Valley Power Station

Narrative Summary Of Operating Experience - January, 1979

Date	Event
January 1 to January 3	January 1 and January 2, the plant was in Operational Mode 1 at full power. At 1806 hours on January 3, the 1C Main Steam Trip Valve closed, causing a reactor trip and safety injection. Cause of trip valve closure was determined to have been from moisture, causing shorting of Valve Partial Stroke Test Switch. A plant cooldown was started at 2339 hours on January 3.
January 4 to January 12	The plant entered Operational Mode 5 at 1005 hours on January 4. During this outage period, maintenance was performed on the main steam trip valves, 1C Charging Pump, containment personnel air lock and main turbine.
January 13 to January 17	On January 13, at 1009 hours, RCS heatup commenced. The plant entered Operational Mode 4 at 1340 hours on January 14, but startup was placed on hold because of condensate cation conductivity being out of specification. At 0605 hours on January 14, the plant entered Operational Mode 3 and reached operating temperature at 2200 hours. Startup remained on hold for completion of turbine maintenance.
January 18	On 1/18/79, the reactor was taken critical at 0144 hours and the main unit was synchronized at 0430 hours. The reactor tripped on high steam generator level during the synchronizing transient. The reactor was critical at 0507 hours and the main unit synchronized at 0605 hours with power level increased to full power. At 2205 hours, a reactor trip and safety injection occurred from high steam flow when the heater drain tank high level dump valve failed open and load was being removed from the main unit in attempt to balance steam flow to feedwater capability.
January 19	On January 19, the reactor was taken critical at 1725 hours and the main unit synchronized at 2329 hours and power level increased.
January 20	On January 20 at 1733 hours, a reactor trip occurred when No. 3 Vital Bus Inverter failed, giving false indication of a loss of one reactor coolant pump. The reactor was taken critical at 1927 hours and the main unit synchronized at 2107 hours and power level increased.

Narrative Summary Of Operating Experience - January, 1979

Date	Event
January 21 to January 25	The plant was in Operational Mode 1 at full power.
January 26	On January 26, the reactor tripped at 0905 hours on high steam generator level while attempting to repair the 1B Main Feedwater Regulating Valve positioner. The reactor was taken critical at 1909 hours, the main unit synchronized at 2213 hours, and power level increased.
January 27	Plant operated between 20% and 50% power level for system load requirements and steam generator chemistry.
January 28	At 1218 hours with plant at approximately 65% power, the inside containment isolation trip valve for component cooling water to the 1C Reactor Coolant Pump failed shut. While decreasing load prior to securing the pump, the reactor tripped on low low steam generator level at 1237 hours. The reactor was taken critical at 1402 hours for xenon follow. After the failed valve diaphragm was replaced, the main unit was started up and synchronized at 2349 hours but the reactor tripped on low low steam generator water level during the transient.
January 29	The reactor was taken critical at 0154 hours, the main unit synchronized at 0333 hours, and power level increased, reaching full power at 0830 hours.
January 30 to January 31	The plant was in Operational Mode 1 at full power.

Major Safety Related Maintenance - January, 1979

1. Containment air lock door repairs were started on January 5 and completed on January 7.

2. The rotating element of Charging Pump [CH-P-1C] was replaced between January 5 through January 13 because of low discharge pressure.

3. The 1B and 1C main steam trip valves were opened for internal inspection after tripping closed on January 3. The discs and seats were in good condition. The rocker shaft seals and packing were replaced to eliminate leakage on all three of the main steam trip valves.