

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N E F C S 1 2 0 0 0 0 0 0 0 0 0 0 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 37 CAT 38

CON'T

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 8 5 7 1 2 2 0 7 8 8 0 1 0 2 7 9 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 During plant heatup an operator noted that one pressure channel for the PORV system
0 3 was not responding. Technician troubleshooting the problem pulled recorder fuses
0 4 which caused both PORVs to open. The operator closed both PORV isolation valves to
0 5 terminate the transient. Both PORVs were returned to service 15 minutes later.
0 6
0 7
0 8

0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
C J 11 D 12 Z 13 V A L V E X 14 X 15 B 16
7 8 9 10 11 12 13 18 19 20
17 LER RO EVENT YEAR
REPORT NUMBER 7 8
21 22
23
24 25 26 27
28 29
30 31
32
ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP COMPONENT
TAKEN ACTION ON PLANT METHOD SUBMITTED FORM SUB. SUPPLIER MANUFACTURER
E 18 F 19 Z 20 Z 21 0 0 0 0 22 Y 23 N 24 N 25 D 2 4 3 1 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 Additional administrative controls were in effect during the short time the PORVs were
1 1 isolated to ensure that an overpressure condition would not occur. Additionally, if
1 2 at least one PORV could not have been returned to service the reactor coolant system
1 3 would have been depressurized and vented as required by Technical Specification
1 4 2.1.6(4). Modifications are being considered to prevent recurrence.

1 5 FACILITY STATUS % POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32
C 28 0 0 0 29 NA A 31 Operator Observation
7 8 9 10 12 13 44 45 46 80
1 6 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36
Z 33 Z 34 NA NA
7 8 9 10 11 44 45 80
1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39
0 0 0 37 Z 38 NA
7 8 9 11 12 13 80
1 8 PERSONNEL INJURIES NUMBER DESCRIPTION 41
0 0 0 40 NA
7 8 9 11 12 80
1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43
Z 42 NA
7 8 9 10 80
2 0 PUBLICITY ISSUED DESCRIPTION 45
N 44 NA
7 8 9 10 80

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LER 78-045
Omaha Public Power District
Fort Calhoun Station Unit No. 1
Docket No. 05000285

ATTACHMENT NO. 1

Safety Analysis

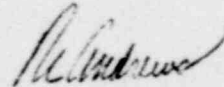
The power operated relief valves (PORV) provide low temperature overpressurization protection for the reactor coolant system.

At the time when the PORVs were isolated at Fort Calhoun Station the high pressure safety injection pump and the reactor coolant pumps were caution tagged in the pull-stop position. The Shift Supervisor stationed an operator full time at the pressure controls of the reactor coolant system during the period of time when both PORVs were isolated.

By taking these additional precautions the probability of an overpressurization event during the time the PORVs were isolated is significantly reduced.

The action of the operator to isolate the PORVs to terminate the depressurization transient and the additional administrative controls implemented are considered to be appropriate and proper.

If at least one PORV had not been promptly returned to an operable status, the reactor coolant system would have been depressurized and vented as required by Technical Specification 2.1.6(4). Since for a period of 15 minutes both PORVs were isolated with the reactor coolant system at low temperature (140°F) the least conservative aspect of Technical Specification 2.1.6(4) was exceeded and the event considered reportable.

A handwritten signature in cursive script, likely of the person responsible for the analysis or report.

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ATTACHMENT NO. 2

Corrective Action

The immediate corrective action was to restore both PORVs to an operable status and complete repairs/calibration of the effected channel as necessary.

Since (1) either one of two redundant low temperature/high pressure channels will actuate both power operated relief valves, and since (2) loss of power to indicating recorders when the reactor coolant temperature is less than 300°F in either channel will also cause a spurious and unnecessary actuation of both PORVs, the following permanent corrective action is being considered:

- (1) Both PORV low temperature/high pressure channels may be modified to ensure that recorder power loss will not result in spurious actuation of the PORVs.
- (2) A more practical means of disabling one channel for maintenance while ensuring that the redundant channel is operable will be investigated.

M. Andrews

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ATTACHMENT NO. 3

Failure Data

This is the first incident involving inoperability of low temperature over-pressurization protection system.

Al Andrews