

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK:

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CON'T

REPORT
SOURCE

REPORT SOURCE	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80				
	1	6	0	5	0	0	0	3	3	3	7	1	2	2	7	7	9	8	0	1	1	6	8	0	9
	ROCKET NUMBER										EVENT DATE					REPORT DATE									

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2	SEE ATTACHMENT
0	3	
0	4	
0	5	
0	6	
0	7	
0	8	

SYSTEM CODE 1 B 11		CAUSE CODE E 12		CAUSE SUBCODE A 13		COMPONENT CODE I N S T R U 14		COMP. SUBCODE X 15		VALVE SUBCODE Z 16	
EVENT YEAR 7 9 21 22		SEQUENTIAL REPORT NO. 1 1 6 23 24 25 26		OCCURRENCE CODE 0 3 27 28 29		REPORT TYPE L 30 31		REVISION NO. 0 32			
ACTION TAKEN X 18		FUTURE ACTION Y 19		EFFECT ON PLANT C 20		SHUTDOWN METHOD A 21		HOURS 0 0 4 8 22 37 40		ATTACHMENT SUBMITTED Y 23	
NPRD-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER G 0 8 0 44 47							

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	SEE ATTACHMENT
1	1	
1	2	
1	3	
1	4	

8 9
FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

1 5 C (28) 0 0 0 (29) NA A (31) Operator Observation

1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 2 33 4 34 NA

7 8 9 10 11 44

AMOUNT OF ACTIVITY (35)

LOCATION OF RELEASE (36)

NA 45

PERSONNEL EXPOSURES

NUMBER			TYPE	DESCRIPTION
1	7	0 0 0	17 Z 38	39 NA

PERSONNEL INJURIES		DESCRIPTION		NA	
NUMBER					
1	2	3	4	5	6
0	0	0	40		1783 333

7 9 11 12
LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
1 9 Z (42) NA

7 8 9 10
PUBLICITY
ISSUED DESCRIPTION (45)
2 0 N (44) NA
68 69 NRC USE ONLY

NAME OF PREPARER W. Verne Childs

PHONE: (315) 342-3840

8001220502

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-116/03L-0

Page 1 of 1

Following a reactor scram and during the subsequent plant startup, a number of problems with SRM's were encountered. While some of these difficulties are not considered to be specific instances of operation in a degraded mode permitted by Technical Specifications Appendix A, Table 3.2-3, they are included in this report in an effort to fully describe the series of problems encountered:

- 1) On December 6, 1979 SRM Channel C was observed to be occasionally spiking upscale during normal operation of the plant in the RUN mode with SRM instrumentation fully withdrawn. Investigation did not reveal the cause of the occasional spikes. The problem was not discovered as part of surveillance testing and did not result in operation in a degraded mode.
- 2) On December 10, 1979 SRM Channel A failed upscale during normal operation of the plant in the RUN mode with SRM instrumentation fully withdrawn. Investigation revealed electrical breakdown of the detector, detector cable or cable connector inside primary containment at approximately 250 VDC compared to a normal detector high voltage of 650 VDC. SRM A was bypassed and normal operation continued.
- 3) Following a reactor scram on December 27, 1979, SRM Channel C remained upscale while the remaining SRM channels followed the decay in reactor power. Investigation did not reveal any cause for the upscale reading. Following recalibration, the instrument responded normally.
- 4) On December 27, 1979 and during surveillance testing to permit reactor startup, SRM Channel D was discovered to have a failed 15 volt DC voltage regulator in the instruments power supply. Replacement of the power supply voltage regulator and recalibration restored the instrument to normal.
- 5) On December 28, 1979, following repair and/or testing of SRM C and D, and following the completion of other surveillance testing required for startup, reactor startup was initiated with SRM B, C, and D operable and SRM A inoperable.
During the startup, prior to reaching the point where reactor became critical, SRM Channel C failed upscale. At this point reactor was fully shutdown and cooled down to allow repair of SRM instrumentation components within primary containment. The detector connector for SRM Channel A was found to have some indication of exterior corrosion and the connector was replaced. SRM C detector connector revealed the presence of moisture inside the connector and both the detector and cable connectors were replaced. Following this work, calibration of the instruments restored all four (4) SRM Channels to an operable status.
- 6) Following a reactor startup on December 30, 1979 and during heat up of the plant, SRM Channel A again failed upscale. Corrective action has been deferred until the next primary containment entry.
- 7) Following the units return to service and at approximately 60% rated power, SRM Channel C failed upscale on January 1, 1980. Investigation and repair has been deferred until the next primary containment entry.

A followup report describing the corrective action required to restore SRM Channels A and C to operable status will be submitted at a later date.

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