LICENSEE EVENT REPORT

LER 79-28/3L

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	V T V Y S 1 (2)	0 0 - 0 0 0 0 0 - 0 0 (3)	4 1 1 1 1 (4)	(5)
7 8	9 LICENSEE CODE 14	15 LICENSE NUMBER 25	26 LICENSE TYPE 30	57 CAT 58

CON'T

0 1	REPORT SOURCE L (6)	0 5 0 0 0 2 7 1 (7)	1 0 2 8 7 9 (8)	1 1 1 3 7 9 (9)
7 8	60	61 DOCKET NUMBER 68	69 EVENT DATE 74	75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2

0 3 See attached sheet

0 4

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0 6

0 7

0 8

0 9	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	S D (11)	E (12)	B (13)	V A L V E X (14)	X (15)	D (16)
7 8	9 10	11	12	13 18	19	20

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
7 9	—	0 2 8	/	0 3	L	—	0
21 22	23	24 26	27	28 29	30	31	32

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
B (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	A (25)	X 9 9 9 (26)
33	34	35	36	37 40	41	42	43	44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0

1 1 See attached sheet

1 2

1 3

1 4

1 5	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
	H (28)	0 0 0 (29)	NA	B (31)	Surveillance Tests
7 8	9	10 12	13 44	45	46 80

1 6	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
	Z (33)	Z (34)	NA	NA
7 8	9	10	11 44	45 80

PERSONNEL EXPOSURES

1 7	NUMBER	TYPE	DESCRIPTION (39)
	0 0 0 (37)	Z (38)	NA
7 8	9 11	12	13 80

PERSONNEL INJURIES

1 8	NUMBER	DESCRIPTION (41)
	0 0 0 (40)	NA
7 8	9 11	12 80

1776 304

LOSS OF OR DAMAGE TO FACILITY (43)

1 9	TYPE	DESCRIPTION
	Z (42)	NA
7 8	9	10 80

PUBLICITY

2 0	ISSUED	DESCRIPTION (45)	NRC USE ONLY
	N (44)	NA													
7 8	9	10 68	69 80

NAME OF PREPARER W. F. Conway PHONE: 802-257-7711

7911160 547

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

During the performance of refuel outage related surveillance testing in accordance with Tech. Spec. Section 4.7.A.2.b, two primary containment isolation valves were found to have seat lekages in excess of that permitted by Tech. Spec. Section 4.7.A.4. For each valve found to have seat leakage greater than that allowable, a second valve in the applicable line provided the proper containment isolation capability. It is therefore, concluded that there were no adverse consequences of this event to the health and safety of the public. Similar events were reported as RO 77-26/3L and RO 76-24/3L. The two valves and their as-found leakage rates are as follows:

Valve Description	As-Found Leakage Rate	Test Date
V12-68 (Reactor Cleanup Return Line Isolation Valve)	2.03 lbm/hr	10/17/79
V-16-20-20 (Containment Purge Makeup Isolation Valve)	Indeterminable	9/28/79

Both valves were repaired and satisfactorily retested prior to plant startup as required by Tech. Spec. Section 3.7.A.4.

CAUSE DESCRIPTION AND CORRECTION ACTIONS

The valve failures summarized in this report and their repairs and retests will be reported in accordance with Appendix J, 10 CFR 50, V.B.3.