

CONTROL BLOCK:

						(1)
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0	1	N	Y	J	A	F	I	(2)	0	0	-	0	0	0	0	-	0	0	0	(3)	4							(4)			(5)
7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE						30	57	CAT	58

CON'T

0	1
7	8

REPORT SOURCE

60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80				
1	6	0	5	0	0	0	3	3	3	7	1	0	0	8	7	9	8	1	1	0	5	7	9	9

DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

08 _____

7 8

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

0 9 11 12 13 14 15 16

9 10 11 12 13 14 15 16 17 18 19 20

(17) LER RO REPORT NUMBER [EVENT YEAR] [21] [7] [22] [23] [—] [24] [0] [25] [8] [26] [3] [27] [—] [28] [0] [29] [3] [30] [L] [31] [—] [32] [0]

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS				ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER			
X	18	Z	19	B	20	Z	21	0	0	0	0	Y	23	N	24		25				
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	4	
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8 9 FACILITY STATUS (28) 1 5 E 2 7 8 9 10 11 12 13 % POWER 1 0 0 (29) NA OTHER STATUS (30) 44

METHOD OF DISCOVERY (31) 45 A 46 OPERATOR OBSERVATION (32) 80

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 2 33 34 NA NA

7 8 9 10 11 44 45 80

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	NA	
7	8	9	11	12	13				90

PERSONNEL INJURIES									
NUMBER			DESCRIPTION (41)						
1	2	3	4	5	6	7	8	9	10
7	8	9	10	11	12				

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NA

LOSS OF OR DAMAGE TO FACILITY					
TYPE		DESCRIPTION			
1	9	Z	(42)	NA	
7	8	9	10		

PUBLICITY
 ISSUED DESCRIPTION (45)
 2 0 N (44) NA
 7911 090 529
 NRC USE ONLY
 68 69 80

NAME OF PREPARER W. Verne Childs

PHONE: (315) 342-3840

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-083/03L-0

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During normal operation, operations personnel noted that a routine computer printout of the core performance log indicated that the Maximum Fraction of Limiting Power Density (MFLPD) was 1.014 and 1.016 for two nodes compared to a Technical Specification limit of equal to or less than 1.000. Reactor power was immediately reduced approximately 3% in accordance with the requirements of Technical Specification, Appendix A, Paragraph 4.1.B. This action restored MFLPD to within the limits of Technical Specifications within the time frame allowed and is considered to be equivalent to operation in a degraded mode.

Following the initial power reduction, the control rod pattern was adjusted to reduce local power and power was returned to approximately 100% of rated power three hours later.

The event did not represent any significant hazard to the public health and safety.

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