BALTIMORE GAS AND ELECTRIC COMPANY

GAS AND ELECTRIC BUILDING

BALTIMORE, MARYLAND 21203

ARTHUR E. LUNDVALL, JR.
VICE PRESIDENT
SUPPLY

November 5, 1979

Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attn: Mr. Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Subject: Calvert Cliffs Nuclear Power Plant
Units Nos. 1 & 2, Docket Nos. 50-317 & 50-318
NRC IE Bulletin Nos. 79-01 and 01A

Reference: a) BG&E letter dated 6/13/79 from A. E. Lundvall to Director DROI, NRC IE Bulletin No. 79-01.
b) BG&E letter dated 6/28/79 from A. E. Lundvall to Director, DROI, same subject.
c) BG&E letter dated 7/9/79 from A. E. Lundvall to Director, DROI, same subject.
d) BG&E letter dated 7/16/79, from A. E. Lundvall to B. H. Grier, same subject.

Gentlemen:

Enclosed are two copies of a report which constitutes a further update of our initial report, reference (a), and update, reference (c), concerning the qualification of electrical equipment required to function under accident conditions.

This report is divided into three sections. The first, Section A, identifies the systems and electrical components required to function under accident conditions. A description of the component is detailed along with the location of the component and a component qualification code. This qualification code and a location code are used as cross reference to the other sections of the report. Section B details the component description and environmental parameters for which the components are qualified with references to supporting documentation and methods of qualification. Section C lists the rooms in which the components are located. The environmental parameters associated with each room under accident conditions are defined. We have used abbreviated codes throughout this report as follows:

N/A - (Not Applicable) - The environmental parameter will not affect the component's design function.

NR - (Not Required) - There is no special environmental condition for the component at the location where it is used.

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NTE1 - (Note 1) - Additional documentation has been requested from the manufacturer.

NTE2 - (Note 2) - In cases where the radiation dose rate may exceed 100 mrem/hr, we refer to the following: "Reactor Shielding Design Manual", Rockwell III, T., New York: McGraw-Hill, 1956; Table 10-1.

ARF - This refers to the Auxiliary Building roof where outdoor ambients are present.

ISPR - This refers to the Intake Structure Pump Room where there are no special environmental conditions as a result of a postulated accident.

In addition to the above, further amplifying information regarding the design function of documented components is detailed below. (The component codes and qualification codes are used for reference to the report.)

Ex-Core Nuclear Instrumentation

This includes the ion chamber, fission chamber, proportional counter and preamplifier which are coded NEPR01, NEWR01, NEPC01, NEPA01 respectively. These are associated with the power range and wide range log power channels. Their design function is for safe start-up and power operation. The electrical connectors associated with this instrumentation are coded CON001 thru CON008. These are enclosed in wiring troughs (codes JB0001 and JB

Hydrogen Recombiner Power and Control Panel (CTL004)

The operating temperature for this equipment was specified at 90°F and the calculated maximum temperature during a LOCA is 125°F. We are reviewing this with Westinghouse and do not believe the specified operating temperature is a maximum temperature. The recombiner system is inspected and tested every 6 months as required per the Technical Specification Section 4.6.5.2. There have been no problems with these power and control panels.

Solenoid Valves

These were reviewed and we reported our findings in reference (b) and provided additional information in a subsequent letter, reference (d). Those solenoid coils that were found to be unqualified will be replaced during the present Unit 2 outage. The corresponding Unit 1 coils will be replaced during the next scheduled Unit 1 outage.

There are several recent updates to this report and they are listed below:

MTR016 - Section B, Page 21 - The maximum temperature is 90C.

SV0018 - Section B, Page 32 - The maximum temperature is 100C.

LOC429 - Section C, Page 7 - The maximum temperature anticipated is 125F.

All of the components listed are documented as indicated previously in this report. The components that require further investigation with the vendor and/or user utilities are referenced with the previously defined code NTE1. As we resolve these items, this report will be updated accordingly. We anticipate completion of the undocumented environmental parameters by the end of this year.

Very truly yours,

for A. E. Lundvall, Jr.

cc: Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
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