

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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REPORT SOURCE

L	(6)	0	5	0	0	0	3	2	1	(7)	1	0	0	7	7	9	(8)	1	0	2	2	7	5	(9)
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DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

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02 On 10-7-79, during normal power operation the "A" Reactor Protection System Motor-Gen-

03 erator set tripped due to component failures and the alternate supply for the "A"

04 Reactor protection system essential bus could not be tied on for six hours. The delay

05 in transfer from normal to alternate supply also caused the drywell H₂O₂ analyzers,

06 drywell leakage detection system, and reactor water conductivity monitors and sample

07 station to be inoperable for six hours. The above mentioned analyzers and detection

08 systems are addressed in the Technical Specifications Section (continued)

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP. SUBCODE		VALVE SUBCODE					
I	A	E	F	R	E	L	A	Y	X	A	Z						
9 10		11 12		13 14		15 16 17 18				19 20							
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.									
7	9	0	8	3	0	3	L	0									
21 22		23 24 25 26		27 28 29		30 31		32									
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
E	A	Z	Z	0	0	0	0	Y	Y	N	G	0	8	0			
18 19		20 21		22 23 24 25		26 27		28 29		30 31		32 33 34 35		36 37 38 39			

33 34 35 36 37

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

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1	0	The motor-generator set tripped because the coil of a General Electric CR 120A control
1	1	relay overheated causing another CR 120A relay within the motor-generator set control
1	2	logic to overheat. These two relays provided the start and control signals to the
1	3	motor, thus, with no signals present, the motor-generator set tripped. The inspec-
1	4	tion of the CR 120A relays did not reveal any apparent (continued)

7	8	9	FACILITY STATUS		% POWER		OTHER STATUS		(30)	METHOD OF DISCOVERY		DISCOVERY DESCRIPTION		(32)	
1	5	E	(28)	0	9	9	(29)	NA		Z	(31)	NA			80
7	8	9	ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY		(35)	LOCATION OF RELEASE		(36)			80
1	6	Z	(33)	Z	(34)			NA				NA			80
7	8	9	PERSONNEL EXPOSURES		NUMBER		TYPE		DESCRIPTION		(39)				80
1	7	0	0	0	(37)	Z	(38)			NA					80
7	8	9	PERSONNEL INJURIES		NUMBER		DESCRIPTION		(41)						80
1	8	0	0	0	(40)					NA					80
7	8	9	LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION		(43)						80
1	9	Z	(42)							NA					80
7	8	9	PUBLCITY		ISSUED		DESCRIPTION		(45)						80
2	0	N	(44)							NA					80

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NRC USE ONLY

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Event Description and Probable Consequences (continued)

3.7.A.6.a and 3.6.G. This is a non-repetitive occurrence for the Reactor Protection system motor-generator set control circuitry. This event posed no threat upon public health or safety. The "B" Reactor Protection system motor-generator set was available and operable.

Cause Description and Corrective Actions (continued)

reason for failure. Similar relays in the plant have proved reliable. In the meantime due to the important instrumentation that the "A" Reactor Protection system bus feeds the operations department tried, but unsuccessfully, to transfer power to the alternate supply. The transfer could not be accomplished since the recently installed overvoltage relays for the alternate supply were set at too low a voltage (128 volts) for the actual supply voltage. This was the first time that the alternate supply had been used to feed the bus since the overvoltage relays were added so, the calibration data for the relays was reviewed and the two relays were reset to 130.2 and 130.7 volts which is within the upper limits of their setpoint range, 129 ± 2 volts. The alternate supply was then tied on to the bus and the bus energized. The two damaged CR 120A relays were replaced the next day and the bus power was then transferred to its normal supply, the A motor-generator set.

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