

LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	LICENSE TYPE						30	CAT				58

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7 8

REPORT SOURCE L 6 0 5 0 0 0 2 8 0 7 0 9 0 6 7 9 8 1 0 0 5 7 9 9

50 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | A review of last year's reactor coolant activity records revealed that the limits

0 3 | as set forth in Appendix A-1 of the Technical Specification were exceeded. This event

0 4 | is reportable under T.S. 6.9.1. The health and safety of the public were not

0 5 | affected.

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7 8 9 10 11 12 13 14 15 16

LER/RO REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.	
17		7 9		0 2 5		0 3		1		0	
21 22		23		24 25		27		28 29		30 31	
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED	
Y *		18 19		Z		Z		22		Y	
33 34		35 36		37 38		39 40		41 42		43 44	
NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER							
N		Z		Z 9 9 9							
24		25		26							

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The iodine spike resulted from a reactor trip. The specific activity sampling

1 1 program was initiated as required by Appendix A 1. The unit was returned to

1 2 power.

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7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS (28) 1 5 C

% POWER 0 1 9 (29) NA

OTHER STATUS (30)

METHOD OF DISCOVERY (31) A

DISCOVERY DESCRIPTION (32) Chemistry Personnel

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 7 33 7 34 NA NA

PERSONNEL EXPOSURES					
NUMBER		TYPE	DESCRIPTION		
1	7	(37)	Z	(38)	NA

PERSONNEL INJURIES		DESCRIPTION		1124 349	
1	8	40	NA		
1	8	0	0	0	0

7 8 9 11 12 90
LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
1 9 79 (42) 7910000 71

PUBLICITY		(45)		NRC USE ONLY									
ISSUED		(44)											

NAME OF PREPARER W. L. Stewart

PHONE: (804) 357-3184

NRC USE ONLY

978-0-16-0441

Surry Power Station, Unit 1
Docket No: 50-280
Report No: 79-025/03L-0
Event Date: 9/6/79

Attachment: page 1 of 1

Title of Report: Excessive Reactor Coolant Activity

1. Description of Event:

A review of last year's reactor coolant activity revealed that the specific activity of the primary coolant was 1.15 uCi/gram dose equivalent I-131 following a reactor trip at 19% power level. This event is contrary to Appendix A-1 to DPR-32 of the Technical Specifications, and was reportable under T.S. 6.9.1. However, a report was not submitted at that time.

2. Probable Consequences and Status of Redundant Systems:

Appendix A-1 specifies that a sampling program must be initiated within 2 to 6 hours following a thermal power change exceeding 15 percent of rated thermal power (Table 3.1.D-1.4(b)), and sampling repeated every 4 hours whenever the specific activity is greater than 1.0 uCi/gram dose equivalent I-131. This appendix allows continued operation for up to 48 hours provided operation time does not exceed 10% of unit's total yearly operating time.

3. Cause:

The iodine spike resulted from a reactor trip.

4. Immediate Corrective Action:

The immediate corrective action was the initiation of the specific activity sampling program.

5. Subsequent Corrective Action:

No additional corrective action was required, because the sampling program eventually indicated a specific activity less than 1.0 uCi/gram dose equivalent I-131.

6. Actions Taken to Prevent Recurrence:

Future occurrences of this type will be evaluated, and if necessary, reported expeditiously.

7. Generic Implications:

None