

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	Y	J	A	F	1	2	0	0	-	0	0	0	0	-	0	0	0	3	4	1	1	1	1	1	4	5	
7	8	LICENSEE CODE						14	LICENSE NUMBER										25	LICENSE TYPE						30	CAT		58

CON'T

REPORT SOURCE: 0 1 7 8 L 6 0 5 0 0 0 3 3 3 7 0 3 1 5 7 9 8 0 9 1 1 7 9 9
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | Please See Attachment

03 | Page

05 | Page

0	6
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08

7 8 9

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

0 9 11 12 13 14 15 16

9 10 11 12 13 14 15 16 17 18 19 20

(17) LER NO. REPORT N. NUMBER [7 | 9] [—] [0 | 1 | 5] [/] [0 | 3] [X] [—] [1]
 EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION
 21 22 23 24 25 26 27 28 29 30 31 32

ACTION TAKEN: A 18 Z 19
FUTURE ACTION: 34
EFFECT ON PLANT: Z 20
SHUTDOWN METHOD: Z 21
HOURS: 0 0 0 0 22
ATTACHMENT SUBMITTED: Y 23
NPRD-4 FORM SUB.: Y 24
PRIME COMP. SUPPLIER: N 25
COMPONENT MANUFACTURER: G 0 8 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | Please See Attachment

[illegible]

1	2	
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1	3	
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1	4	
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7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

1 5 D (28) 0 2 1 (29) NA B (31) Surveillance Test

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 Z 33 Z 34 NA

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37) Z (38) NA (39)				

PERSONNEL INJURIES		DESCRIPTION		NA		POOR	
NUMBER							
1	2	0	0	0	40		

8 9 11 12
LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
1 9 Z (42) NA ORIGINAL 949328

2		0	ISSUED		N	44	DESCRIPTION		45	NA		NRC USE ONLY									
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NAME OF PREPARER W. Verne Childs

PHONE: 315-342-3840

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POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-015/03X-1

Page 1 of 1

During a load reduction and shutdown to meet the cold shutdown requirements of a show cause order issued on March 13, 1979, the Rod Sequence Control System (RSCS) failed to pass Operations Surveillance Test F-ST-23A titled "Rod Sequence Control Functional Test." This test is required prior to power reduction to less than 20% of rated power in accordance with Technical Specification Appendix A, Paragraph 3.3.B.3.c. Inadequate time was available to troubleshoot and repair the Rod Sequence Control System, the reactor was manually scrammed from approximately 21% of rated power. The event did not represent any hazard to the public health and safety.

Complete troubleshooting, repair and post repair testing would have required the mode switch to be moved from the shutdown position which was contrary to the requirements of the show cause order. Following the lifting of the show cause order the mode switch was moved from the shutdown position as required to complete the test and repair. RSCS was repaired by replacing certain printed circuit boards and proper operation was demonstrated by satisfactory completion of F-ST-23A on August 24, 1979.

NOTE: Revision 1 of this LER is submitted to update the original report following repair and test. This attachment and Items 12, 13, 17 through 19 are also revised.

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