

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 A L B R F 1 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

CON'T
0 1 REPORT SOURCE L 6 0 5 0 0 0 2 5 9 7 0 8 0 9 7 9 8 0 9 0 7 7 9 9
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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Due to G. E. reevaluation of POST-LOCA conditions, GE SIL 299, High Drywell Tempera-
0 3 ture Effect on Reactor Vessel Water Level Instrumentation, the reactor water level in-
0 4 dications can be 29 inches less than indicated by present procedures. There was no
0 5 hazard to the public health or safety. There have been no previous occurrences. Re-
0 6 dundant systems are not applicable.

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7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

0 9 SYSTEM CODE I A 11 CAUSE CODE X 12 CAUSE SUBCODE Z 13 COMPONENT CODE Z Z Z Z Z Z Z 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

17 LER/RO REPORT NUMBER 7 9 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

ACTION TAKEN X 18 FUTURE ACTION X 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 22 ATTACHMENT SUBMITTED Y 23 NPRD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER G 0 8 0 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The reevaluation of high drywell temperature effects on the reactor vessel water level
1 1 instrumentation has resulted in a design change request 1875 submitted requesting a
1 2 setpoint change for LIS-3-58A-D.

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1 4
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 5 FACILITY STATUS E 29 % POWER 0 9 6 29 OTHER STATUS NA 30 METHOD OF DISCOVERY D 31 DISCOVERY DESCRIPTION Notification by NSSS 32
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 6 ACTIVITY CONTENT Z 33 Z 34 NA 35 AMOUNT OF ACTIVITY NA 36 LOCATION OF RELEASE
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

2 0 PUBLICITY ISSUED N 44 DESCRIPTION NA 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

7 909110 419

939334

NAME OF PREPARER

PHONE

NRC USE ONLY

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 7917 Technical Specification Involved Table 3.2.B

Reported Under Technical Specification 6.7.2.b(1)

Date of Occurrence 8/9/79 Time of Occurrence 3:30 p.m. Unit 1

Identification and Description of Occurrence:

Due to GE reevaluation of POST-LOCA conditions, GE SIL 299, High Drywell Temperature Effect on Reactor Water Level Instrumentation, the reactor water level indications can be 29 inches less than present setpoint.

Conditions Prior to Occurrence:

Normal Operation.

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

None

Apparent Cause of Occurrence:

Heating of the water legs of the Reactor Water Level instruments during a LOCA condition could result in a 29-inch error indication in the instruments.

Analysis of Occurrence:

None

Corrective Action:

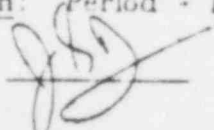
Design change request 1875, has been submitted requesting a setpoint change for LIS-3-58A-D, from ≥ 378 vessel inches to ≥ 407 vessel inches.

Failure Data:

None

*Retention: Period - Lifetime, Responsibility - Administrative Supervisor

*Revision:



939335