

CONTROL BLOCK: 

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 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

7	8	9	SYSTEM CODE I B (11)		CAUSE CODE E (12)	CAUSE SUBCODE G (13)	COMPONENT CODE I N S T R U (14)			COMP. SUBCODE X (15)	VALVE SUBCODE Z (16)	
0	9		9	10	11	12	13	14	15	16	17	
7	8											
LER/RO REPORT NUMBER (17)			EVENT YEAR 7 9 (21 22)		SEQUENTIAL REPORT NO. 0 9 1 (24 25 26)		OCCURRENCE CODE 0 3 (28 29)		REPORT TYPE L (30)		REVISION NO. 0 (32)	
ACTION TAKEN B (18)			FUTURE ACTION Z (19)		EFFECT ON PLANT Z (20)		SHUTDOWN METHOD Z (21)		HOURS 0 0 0 (22 23 24)		ATTACHMENT SUBMITTED Y (23)	
NPRD-4 FORM SUB. Y (24)			PRIME COMP. SUPPLIER N (25)		COMPONENT MANUFACTURER C 5 6 0 (26 27 28)							
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)												

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	4																	80		
7	8																	9		
FACILITY STATUS		% POWER				OTHER STATUS				(30)	METHOD OF DISCOVERY				DISCOVERY DESCRIPTION				(32)	
1	5	E	(28)	1	0	0	(29)	NA	A	(31)	Operator observation									
2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	80

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35)

1 6 Z (33) Z (34) NA

45 LOCATION OF RELEASE (36) 80

PERSONNEL EXPOSURES

NUMBER			TYPE	DESCRIPTION
1	7	0 0 0	(37) Z (38)	NA

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	8	0	0	0	NA

1		9		Z		42		NA		43	
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8 9 10  
PUBLICITY  
ISSUED DESCRIPTION (45)  
2 0 N (44) NA 7910020 475 NRC USE ONLY

NRC USE ONLY

7910020

PHONE: 419-259-5000, Ext. 233

TOLEDO EDISON COMPANY  
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-33-79-106

DATE OF EVENT: September 1, 1979

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Steam Generator 2 Low Level Trip of Steam and Feedwater Rupture Control System (SFRCS) Channel 3

Conditions Prior to Occurrence: The unit was in Mode 1, with Power (MWT) = 2770, and Load (Gross MWE) = 910.

Description of Occurrence: On September 1, 1979 at 1053 hours, a Steam Generator 2 low level trip occurred on SFRCS Channel 3. The Startup Feedwater Valve SP7B closed as designed. The Main Feedwater Control Valve SP6E opened as designed to compensate for the loss of feedwater flow. The string for Level Transmitter LT-SP9A7 to SFRCS was declared inoperable which placed the station in Action Statement (#13) of Technical Specification 3.3.2.2. The Action Statement requires the inoperable section of the channel to be placed in the tripped condition within one hour.

Designation of Apparent Cause of Occurrence: The cause of the occurrence was an inoperable optical isolator, Part #FCD820 on a Consolidated Controls Corporation Input Buffer Card, Part #6N178.

Analysis of Occurrence: There was no danger to the health and safety of the public or to station personnel. The SFRCS performed as designed; the failure of the component forced a trip condition on SFRCS Channel 3.

Corrective Action: On September 1, 1979 under Maintenance Work Order IC-417-79, it was determined that the Input Buffer Card circuit for LSLSP09A7 was bad. A new Input Buffer Card was taken from stock, functionally tested, and installed. Applicable sections of ST 5031.14, the SFRCS Monthly Test, was performed and the channel was declared operable at 1430 hours on September 1, 1979, removing the unit from the action statement.

Failure Data: On January 7, 1978, an input buffer card failed in SFRCS Channel 1. However, that failure was due to a failed diode.

LER #79-091

1071 150