



MISSISSIPPI POWER & LIGHT COMPANY
Helping Build Mississippi
P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

PRODUCTION DEPARTMENT

June 30, 1978

U. S. Nuclear Regulatory Commission
Light Water Reactors Branch No. 1
Division of Project Management
Washington, D. C. 20555

Attention: Mr. John F. Stolz, Chief

Dear Sir:

SUBJECT: Grand Gulf Nuclear Station
Units 1 & 2
Docket Nos. 50/416, 50/417
File 0260/15515/L-835.0/L-813.0
MP&L Position on NUREG-0313 Technical
Report on Material Selection and
Processing Guidelines for BWR
Coolant Pressure Boundary Piping
AECP-78/42

As requested in your letter dated February 28, 1978, please find attached our response on NUREG-0313 "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping." As requested our response addresses each subsection in Sections II and III of NUREG-0313.

This response pertains to Unit 1. Our position on Unit II will be submitted by January, 1979.

If you have any questions on the attached response, please let us know.

Yours truly,

for J. P. McGaughy, Jr.
Director of Power Production

DRB/db

Enclosure: Position on NUREG-0313 for (Grand Gulf Nuclear Station - Unit I)

cc: Mr. Robert McGehee
Mr. Troy Conner
Mr. N. L. Stampley

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Member Middle South Utilities System

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Mississippi Power & Light Company
Position on NUREG - 0313 for
Grand Gulf Nuclear Station - Unit 1

Section II

II.1

stainless steel reactor coolant pressure boundary (RCPB)
fittings for 2 inch and under sizes is of type 304L

stainless steel RCPB piping and fittings greater than 2 inches in
carbon content \geq .035 percent (i.e. type 304). To the extent
these piping spools (including piping, fittings, and shop
supplied in the solution annealed condition.

ly 80 percent of the total number of piping spools greater
in size are in the solution annealed condition. Field welds
solution annealed.

II.2

advanced stage of construction of this Unit, solution annealment of
pipe ends is impractical.

II.3

tion is not applicable.

Section III

III.1

ndations of this subsection will be implemented as applicable.

III.2.A(1)

ndations of this subsection will be implemented for non-service
lines.

III.2.A(2).a

ulf Unit 1 leak detection system meets the position described in
Guide 1.45.

Subsection III.2.A.(2).b

It is requested that the following limiting conditions for plant operation be implemented in lieu of the recommendations specified in this subsection:

With any increase in the rate of unidentified leakage in excess of 2gpm above a baseline leakage rate (to be determined during startup) within a period of 4 hours, reduce the leakage rate below this limit or be in hot shutdown within 12 hours and in cold shutdown within the next 24 hours.

When the total unidentified leakage attains a rate of 5gpm above the baseline leakage (to be determined during startup) reduce the leakage to less than the 5gpm limit within 8 hours or be in hot shutdown within 12 hours and in cold shutdown within the next 24 hours.

The above limiting conditions for operation would allow a reasonable amount of time for evaluation of the problem and implementation of corrective action.

Subsection III.2.A.(2).c

The recommendations of this subsection will be implemented.

Subsection III.2.B.(1)

The recommendations of this subsection will be implemented.

Subsection III.2.B.(2)

This subsection is not applicable. The recirculation system bypass piping as well as the "stub tubes" have been deleted. In addition, the reactor core spray piping is carbon steel.

Subsection III.2.B.(3)

The recommendations of this subsection will be implemented.

Subsection III.2.B.(4)

The recommendations of this subsection will be implemented.