



Consumers  
Power  
Company

Stephen H. Howell  
Senior Vice President

General Offices: 1945 West Parnall Road, Jackson, Michigan 49201 • (517) 788-0453

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Mr J G Keppler, Regional Director  
Office of Inspection and Enforcement  
US Nuclear Regulatory Commission  
Region III  
799 Roosevelt Road  
Glen Ellyn, IL 60137

MIDLAND NUCLEAR PLANT -  
UNIT NO 1, DOCKET NO 50-329  
UNIT NO 2, DOCKET NO 50-330  
SMALL BREAK/REACTOR COOLANT PUMP OPERATION INTERACTION

In accordance with the requirements of 10 CFR 50.55(e), this letter constitutes an interim report concerning a potentially unsafe situation if the reactor coolant pumps are tripped after high system void fractions develop in the loop. Specifically, for the low probability events involving a range of small reactor coolant system breaks (approximately  $0.025 \text{ ft}^2$  to  $0.2 \text{ ft}^2$ ), analyses show that a high void fraction will develop in the primary system during the course of the accident. 10 CFR 50 Appendix K criteria may be exceeded if reactor coolant pumps are tripped after creation of this high void condition. B&W, Midland's NSSS Supplier, has been working closely with the NRC staff (Dr Rosztozy) concerning this analysis.

B&W reports that analyses have been performed which confirm a safe situation if the reactor coolant pumps are tripped promptly in the transient prior to establishing a high void fraction. B&W made a recommendation to the 177FA operating plants that "Upon receipt of an ESFAS actuation caused by low RC pressure, all operating RC pumps must be tripped immediately." CPCo is considering an automatic/safety grade trip system to accomplish the recommended action for the Midland Plant.

Another report, either interim or final, will be sent on or before October 19, 1979.

*Subscribed by Stephen H. Howell*

CC: Director of Office of Inspection and Enforcement  
Att: Mr Victor Stello, USNRC (15)

Director of Office of Management  
Information and Program Control, USNRC (1)

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